

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9612030315 DOC. DATE: 96/11/26 NOTARIZED: NO
 FACIL: 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.
 AUTH. NAME AUTHOR AFFILIATION
 WILKIE, L.V. Duke Power Co.
 HAMPTON, J.W. Duke Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

DOCKET #
05000270

SUBJECT: LER 96-005-00: on 961030, MSR valves technically declared inoperable due to improper assembly of component. W/961126 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

	RECIPIENT		COPIES			RECIPIENT		COPIES	
	ID CODE/NAME		LTTR	ENCL		ID CODE/NAME		LTTR	ENCL
	PD2-2 PD		1	1		LABARGE, D		1	1
INTERNAL:	ACRS		1	1		AEOD/SPD/RAB		2	2
	AEOD/SPD/RRAB		2	2		<u>FILE CENTER</u>		1	1
	NRR/DE/ECGB		1	1		NRR/DE/EELB		1	1
	NRR/DE/EMEB		1	1		NRR/DRCH/HHFB		1	1
	NRR/DRCH/HICB		1	1		NRR/DRCH/HOLB		1	1
	NRR/DRCH/HQMB		1	1		NRR/DRPM/PECB		1	1
	NRR/DSSA/SPLB		1	1		NRR/DSSA/SRXB		1	1
	RES/DET/EIB		1	1		RGN2 FILE 01		1	1
EXTERNAL:	L ST LOBBY WARD		1	1		LITCO BRYCE, J H		1	1
	NOAC MURPHY, G.A		1	1		NOAC POORE, W.		1	1
	NRC PDR		1	1		NUDOCS FULL TXT		1	1

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM OWFN 5D-5 (EXT. 415-2083) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

FULL TEXT CONVERSION REQUIRED
 TOTAL NUMBER OF COPIES REQUIRED: LTTR 26 ENCL 26

AP

C
A
T
E
G
O
R
Y
1
D
O
C
U
M
E
N
T

Duke Power Company
Oconee Nuclear Site
P.O. Box 1439
Seneca, SC 29679

J. W. HAMPTON
Vice President
(864) 885-3198 Office
(864) 885-3564 Fax



DUKE POWER

November 26, 1996

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station Unit
Docket Nos. 50-269, -270, -287
Licensee Event Report 270/96-05
Problem Investigation Process No.: 2-096-2031

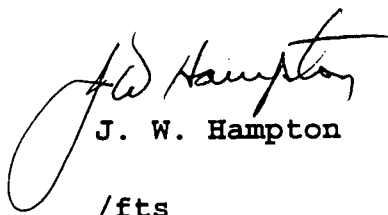
Gentlemen:

Pursuant to 10 CFR 50.73 Sections (a) (1) and (d),
attached is Licensee Event Report 270/96-05, concerning
the technical inoperability of the Main Steam Safety
Valves.

This report is being submitted in accordance with 10 CFR
50.73 (a) (2) (v) (D). This event is considered to be of
no significance with respect to the health and safety of
the public.

A supplemental report will be submitted by January 30,
1997.

Very truly yours,


J. W. Hampton
/fts

Attachment

9612030315 961126
PDR ADOCK 05000270
S PDR

IT 6021

Document Control Desk
November 26, 1996

cc: Mr. S.D. Ebnetter
Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta St., NW, Suite 2900wazzu
Atlanta, GA 30323

Mr. D. E. LaBarge
U.S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, D.C. 20555

INPO Records Center
700 Galleria Parkway
Atlanta, GA 30339-5957

Mr. M. A. Scott
NRC Resident Inspector
Oconee Nuclear Station

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Oconee Nuclear Station, Unit Two										DOCKET NUMBER (2) 05000 270		PAGE (3) 1 Of 1		
TITLE (4) Main Steam Relief Valves Technically Inoperable Due To Improper Assembly Of Component														
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)				
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		DOCKET NUMBER(S)			
									Oconee, Unit Three		05000 287			
10	30	96	96	05	00	11	26	96			05000			
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11)											
POWER LEVEL (10)		0	20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)		
			20.405(a)(1)(i)			50.36(c)(1)			X 50.73(a)(2)(v) (D)			73.71(c)		
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in		
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)			Abstract below and		
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)			in Text, NRC Form		
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)			366A)		
LICENSEE CONTACT FOR THIS LER (12)														
NAME										TELEPHONE NUMBER				
Lanny V. Wilkie, Safety Review Manager										AREA CODE				
										(864)		885-3518		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)														
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS				
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE)								NO		01	30	97	

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

On October 14, 1996, Units 1 and 2 were at cold shutdown and Unit 3 was in a refueling outage. As a result of industry events, an inspection of the Unit 2 Main Steam Safety Valves (MSSV) was conducted. This inspection indicated that the spindle nut cotter pin was not properly installed on four of the sixteen valves. On October 24, 1996, an inspection of the Unit 3 MSSVs indicated that the spindle nut cotter pins were missing from two of the sixteen valves. All MSSV spindle nut cotter pins on Unit 1 were found to be properly installed. On October 29, 1996, at 1000 hours, an engineering past operability review concluded that a reasonable doubt existed as to whether the MSSVs could have mitigated the consequences of certain accidents. Further analysis on past operability is ongoing. The root cause and corrective actions will be provided in the supplemental report.