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ACCESSION NBR:9608270268 DOC.DATE: 96/08/08 NOTARIZED: NO  
 FACIL:50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.  
 AUTH.NAME AUTHOR AFFILIATION  
 WILKIE,L.V. Duke Power Co.  
 HAMPTON,J.W. Duke Power Co.  
 RECIP.NAME RECIPIENT AFFILIATION

DOCKET #  
 05000270

SUBJECT: LER 96-003-00:on 960426,pressurizer relief valve had been determined inoperable,due to failure as found set pressure testing criteria.Valve was disassembled,reassembled tested W/acceptable results.W/960819 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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**DUKE POWER**

**DATE: August 19, 1996**

**U.S. Nuclear Regulatory Commission  
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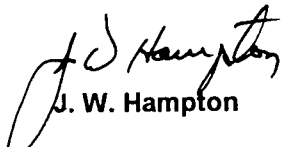
**Subject: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287  
Licensee Event Report 270/96-03**

**Gentlemen:**

**Pursuant to 10 CFR 50.73 Sections (a) (1) and (d), attached is Licensee Event Report 270/96-03, concerning a past inoperable Pressurizer Relief Valve .**

**This report is being submitted in accordance with 10 CFR 50.73 (a) (2) (v) (D). This event is considered to be of no significance with respect to the health and safety of the public.**

**Very truly yours,**

  
J. W. Hampton  
/fts

**Attachment**

**cc: Mr. S.D. Ebner  
Administrator, Region II  
U.S. Nuclear Regulatory Commission  
101 Marietta St., NW, Suite 2900  
Atlanta, GA 30323**

**INPO Records Center  
700 Galleria Parkway  
Atlanta, GA 30339-5957**

**Mr. D. E. LaBarge  
U.S. Nuclear Regulatory Commission  
Office of Nuclear Reactor Regulation  
Washington, D.C. 20555**

**Mr. M. Scott  
NRC Resident Inspector  
Oconee Nuclear Station**

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ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1)

Oconee Nuclear Station

DOCKET NUMBER (2)

05000270

PAGE (3)

1 of 1

TITLE (4) Pressurizer Relief Valve Technically Inoperable

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)
04	26	96	96	03	00	08	08	96		05000
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11)							
N			20.402(b)			20.405(c)			50.73(a)(2)(iv)	
POWER LEVEL (10)			20.405(a)(1)(i)			50.36(c)(1)			x 50.73(a)(2)(v)	
0			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)	
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	
									73.71(b)	
									73.71(c)	
									OTHER (Specify in Abstract below and in Text, NRC Form 366A)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

L. V. Wilkie, Safety Review Manager

TELEPHONE NUMBER

AREA CODE

(864)

885-3518

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

X

YES (if yes, complete EXPECTED SUBMISSION DATE)

NO

EXPECTED SUBMISSION DATE (15)

MONTH

10

DAY

30

YEAR

96

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

On April 26, 1996, during a refueling outage on Unit 2, a Vendor notified Oconee Engineering that 2RC-67 (Pressurizer Relief Valve) failed to meet its as found set pressure testing criteria. The valve lifted at a pressure which was outside the set pressure band. The Vendor disassembled and inspected the valve and no problems were noted. The valve was reassembled and tested with acceptable results. The past operability of 2RC-67 is being evaluated for the period between November 1994 and March 1996 while the valve was in service. However, the analysis model must include overall Reactor Coolant System response to several design basis accidents and is still under development by off-site personnel. On July 31, 1996, site personnel learned that some preliminary calculations indicated unacceptable results, thus increasing doubt that the valve was past operable. Therefore, the decision was made to report this condition pending final resolution.