

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

Item 1

Page: Various, as listed in Attachment 2.

Section: Various, as listed in Attachment 2.

Description of Change

Additional details for the administrative controls for: 1) the Site's Safety Groups, 2) Technical Review and Control, 3) the Nuclear Safety Review Board, 4) Audits, 5) Safety Limits, 6) Post-maintenance/modification Testing, 7) Reportable Events 8) Procedures, and 9) Records Retention are being added to the Topical Report.

Reason for Change

This change is being made to support implementation of the Improved Technical Specifications (ITS) at Oconee, McGuire, and Catawba Nuclear Stations.

Basis for Change

This change is consistent with NUREG 1430, Standard Technical Specifications for Babcock and Wilcox Plants; and NUREG-1431, Standard Technical Specifications for Westinghouse Plants. Based on these NUREGs, Duke has proposed Technical Specifications amendments for the conversion of the current Oconee, McGuire, and Catawba Technical Specifications to the new ITS. These amendments were submitted to the NRC by Duke letters dated May 27, 1997 for McGuire and Catawba; and October 28, 1997 for Oconee. Each of these amendments was submitted pursuant to 10CFR50.90 and contained a description and justification for several changes to each station's current Technical Specifications. These changes involved the relocation of numerous existing Technical Specifications requirements to the Topical Report. Amendment 23 includes those Technical Specifications requirements that were identified for relocation to the Topical Report.

This change does not reduce any commitments now contained in the Duke Quality Assurance Program, since it involves only the addition of current Technical Specifications requirements. Duke is presently operating Oconee, McGuire, and Catawba in compliance with these current Technical Specifications. There are no Topical

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

Report deletions or Quality Assurance Program reductions resulting from the inclusion of these existing Technical Specifications requirements.

Section 17.3.3.2.4 of Amendment 23 contains the relocated requirements for the Safety Review Group (SRG) at each nuclear site. In a letter to the NRC dated March 3, 1998, Duke submitted a license amendment request (LAR) for Catawba Nuclear Station that revised the current requirements for the composition of the SRG. The contents of Section 17.3.3.2.4 are based upon the March 3, 1998 LAR. With the implementation of Amendment 23, Duke is proposing to adopt SRG requirements that are consistent among Oconee, McGuire, and Catawba Nuclear Stations.

The current Technical Specifications being relocated to the Topical Report, and the location in Amendment 23, are listed in Attachment 2 and shown on the marked copies of the current Technical Specifications contained in Attachment 3.

Item 2

Page: 17-3, 17-16, 17-21, and 17-38

Section: 17. Quality Assurance, Definitions; 17.3.1.2.3, Electric System Support Department; Figure 17-3, Off-site Organization; and 17.3.2.14, Document Control

Description of Change

Administrative changes are being made in Amendment 23 to address a recent reorganization of the Electric System Support Department. This reorganization was previously described to the NRC in a Duke letter dated January 8, 1998.

Reason for Change

This change is being made in order that the Topical Report will accurately describe the current Duke nuclear organization.

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

Basis for Change

This change is solely administrative in nature and does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 3

Page: 17-16

Section: 17.3.1.2.4, Duke Power Group Human Resources Department

Description of Change

An administrative change is are being made in Amendment 23 to add the Nuclear Access Program as a responsibility of the Duke Power Group Human Resources Department.

Reason for Change

This change is being made in order that the Topical Report will be more inclusive of the nuclear quality related activities performed by the Duke Power Group Human Resources Department.

Basis for Change

This change is being made to more accurately state the nuclear quality related responsibilities of the Duke Power Group Human Resources Department. This change is solely administrative in nature and does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 4

Page: 17-16

Section: 17.3.1.2.5, Duke Power Group Environmental, Health and Safety Department

Description of Change

An administrative change is are being made in Amendment 23 to add soils testing and environmental services as responsibilities of the Duke Power Group Environmental, Health and Safety Department.

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

These responsibilities are being relocated from the Shared Services Department as currently described in Section 17.3.1.2.6.

Reason for Change

This change is being made in order that the Topical Report will accurately describe the responsibilities of the corporate departments performing nuclear quality related activities.

Basis for Change

This change is being made to more accurately state the nuclear quality related responsibilities of the Duke Power Group Environmental, Health and Safety and the Shared Services Departments. This change is based upon a further clarification of departmental responsibilities following the Duke Power Company/Pan Energy Corp. merger that was described to the NRC in a Duke letter dated July 15, 1998. This change is solely administrative in nature and reflects an organizational change only. This change does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 5

Page: 17-16

Section: 17.3.1.2.6, Shared Services Department

Description of Change

An administrative change is are being made in Amendment 23 to delete soils testing and environmental services as responsibilities of the Shared Services Department. These responsibilities are being relocated to Section 17.3.1.2.5, Duke Power Group Environmental, Health and Safety Department as described in Item 4. The description of the remaining responsibilities of the Shared Services Department has been rewritten for clarity.

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

Reason for Change

This change is being made in order that the Topical Report will accurately describe the responsibilities of the corporate departments performing nuclear quality related activities.

Basis for Change

This change is being made to more accurately state the nuclear quality related responsibilities of the Shared Services Department. This change is based upon a further clarification of departmental responsibilities following the Duke Power Company/Pan Energy Corp. merger that was described to the NRC in a Duke letter dated July 15, 1998. This change is solely administrative in nature and reflects an organizational change only. This change does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 6

Page: 17-17

Section: 17.3.1.2.7, Information Management Department

Description of Change

An administrative change is are being made in Amendment 23 to revise the wording of the description of the responsibilities of the Information Management Department. This change replaces the wording, "mainframe computer software and data ..." with, "selected information technology services and support for the Nuclear Generation Department, some of... ."

Reason for Change

This change is being made in order that the Topical Report will more accurately describe the responsibilities of the nuclear quality related activities performed by the Information Management Department.

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

Basis for Change

This change is being made to update the description of nuclear quality related responsibilities of the Information Management Department. This change is solely administrative in nature and reflects a change in wording only. This change does not affect the work performed by the Information Management Department in support of the Duke nuclear stations, nor does it reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 7

Page: 17-30

Section: 17.3.2.8, Test Control

Description of Change

Additional clarification is being added to state that functional verifications are used as a means to verify the satisfactory performance of affected items following maintenance or modification.

Reason for Change

By letter dated November 4, 1996, the NRC issued Inspection Report Nos. 50-269/96-13, 50-270/96-13, and 50-287/96-13. Violation A, as described in these inspection reports, involved a situation in which a testing procedure did not sufficiently include information as required by the Topical Report. In its December 4, 1996 response to this violation, Duke committed to perform a comprehensive review of the Topical Report, and other documents, to assess the adequacy of these documents as they pertain to non Code required post modification functional verifications.

Basis for Change

The change contained in Amendment 23 implements the results of the Duke review and is intended to provide a better correlation between post modification verification implementing procedures and the Topical Report. An addition is being made to Section 17.3.2.8 to include functional verifications as a means of verifying post maintenance/modification performance. This change only clarifies a

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

discontinuity between the Topical Report and Duke's implementing procedures. This change does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 8

Page: 17-38

Section: 17.3.2.14, Document Control

Description of Change

An administrative change is being made in Amendment 23 to make an addition to the description of the content of the Information Management Quality Manual. This change adds the wording, "and the information technology infrastructure"

Reason for Change

This change is being made in order that the Topical Report will more accurately describe the content of the Information Management Department Quality Manual.

Basis for Change

This change is being made to update the description of nuclear quality related work activities contained in the Information Management Quality Manual. This change is solely administrative in nature and reflects an addition to this section of the Duke Quality Assurance Program. This change provides additional detail on the content of the Information Management Quality Manual and it does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 9

Page: 17-38

Section: 17.3.2.14, Document Control

Description of Change

An administrative change is being made in Amendment 23 to revise the paragraph that describes the location of the policies and

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

procedures that govern the nuclear quality related activities of the Duke Power Group Human Resources Department. These policies and procedures are now contained within the Nuclear Policy Manual (already discussed in Section 17.3.2.14) and the applicable approved Human Resources Procedures. Currently, Topical Report Amendment 22 describes these policies and procedures as being located in the Duke Power Group Human Resources Department Manual.

Reason for Change

This change is being made in order that the Topical Report will accurately describe the location of the policies and procedures of the Duke Power Group Human Resources Department.

Basis for Change

This change is being made to update the description of the nuclear quality related policies and procedures of the Duke Power Group Human Resources Department. This change is solely administrative in nature and reflects a relocation of the policies and procedures that govern Duke's Fitness-for-Duty and Nuclear Access Programs. This change describes an administrative relocation only. There are no programmatic changes in the implementation of the programs governed by the affected policies and procedures. This change does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Item 10

Page: 17-43

Section: 17.3.2.15, Records

Description of Change

Records retention requirements for the Duke Power Group Environmental, Health and Safety Department.

Reason for Change

This change is being made in order that the Topical Report will contain records retention requirements for the Duke Power Group Environmental, Health and Safety Department.

ATTACHMENT 1
Duke Energy Corporation Topical Report, Duke-1-A,
Quality Assurance Program, Amendment 23
Listing and Discussion of Amendment 23 Contents

Basis for Change

This change is being made to update the description of the records retention requirements contained in Section 17.3.2.15 of the Topical Report. This change adds requirements for the Duke Power Group Environmental, Health and Safety Department to retain records related to nuclear quality related activities. This addition is consistent with similar requirements currently contained in the Topical Report for other Duke Energy Corporation Departments performing nuclear quality related activities. This change describes an addition to the Duke Quality Assurance Program. This change does not reduce any commitments currently contained in the Duke Quality Assurance Program.

Attachment 2

Duke Energy Corporation Oconee, McGuire, and Catawba Nuclear Stations

Relocation of Current Technical Specifications into the Quality Assurance Program Topical Report (Amendment 23)

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Oconee</u>			
1	6.3.2 6.3.3	Safety Limit Violation & Reporting	17.3.2.13, Pages 17-35, 36
2	4.5.1.2.2 4.7.1b	Post Maintenance Action for Pressure Isolation Valves/ Control Rods	17.3.2.8, Page 17-32
3	4.20.3.a.1	Post Maintenance Action for SSF Diesels	17.3.2.8, Page 17-32
4	6.1.1.9	OEP Provided by by SRG	17.3.3.2.4, Page 17-48
5	6.1.2.1	Technical Review and Control (TR&C)	17.3.2.14, Pages 17-37, 38, 39, 40
6	6.1.2.1a	TR&C Procedures and Programs	17.3.2.14, Pages 17-38, 39, 40
7	6.1.2.1b	TR&C Modifications	17.3.2.2, Pages 17-23, 24
8	6.1.2.1c	TR&C Qualified Reviewers	17.3.2.14, Page 17-39
9	6.1.2.1d	TR&C FOL/TS Changes	17.3.2.14, Page 17-37

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Oconee</u>			
10	6.1.2.1e	TR&C Tests and Experiments	17.3.2.10, Page 17-33
11	6.1.2.1f	TR&C Reportable Incidents TS Violations	17.3.2.13, Pages 17-35, 36
12	6.1.2.1g	TR&C Special Reviews and Investigations	17.3.2.13, Pages 17-35, 36
13	6.1.2.1h	TR&C Unplanned Onsite Releases	17.3.2.13, Page 17-36
14	6.1.2.1i	TR&C PCP, ODCM, Radwaste Systems	17.3.2.14, Page 17-39
15	6.1.2.1j	TR&C Fire Protection	17.3.2.14, Page 17-39
16	6.1.2.1k	TR&C Records	17.3.2.15, Pages 17-40 thru 17-44
17	6.1.3.1a-h 6.1.3.2a-g 6.1.3.3a-i	NSRB	17.3.3.2.1, Pages 17-45, 46
18	6.1.3.4a-1	Audits	17.3.3.2.3, Pages 17-47, 48
19	6.1.3.5a	NSRB Authority	17.3.3.2.1, Page 17-46
20	6.1.3.5b	NSRB Minutes	17.3.3.2.1, Page 17-46
21	6.1.3.5c	NSRB Records	17.3.2.15, Page 17-42

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Oconee</u>			
22	6.1.3.5d	Audit Reports Forwarded in 30 days	17.3.3.2.3, Page 17-48
23	6.2	Reportable Events	17.3.2.13, Pages 17-35, 36
24	6.4.1n	PCP Implementing Procedures	17.3.2.14, Page 17-39
25	6.4.1o	TR&C Implementing Procedures	17.3.2.14, Pages 17-37, 38, 39
26	6.4.1p	PORC Implementing Procedures	17.3.2.14, Page 17-39
27	6.4.2	Procedures Review/Approval	17.3.2.14, Pages 17-39, 40
28	6.4.3	Temporary Changes to Procedures	17.3.2.14, Pages 17-39, 40

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Oconee</u>			
29	6.5.1	Records	17.3.2.15, Pages 17-40 thru 17-44
	6.5.1a		Item b, Page 17-40
	6.5.1b		Item f, Page 17-41
	6.5.1c		Item f, Page 17-41
	6.5.1d		Item c, Page 17-40
	6.5.1e		Item y, Page 17-41
	6.5.1f		Item d, Page 17-41
	6.5.1g		Item e, Page 17-41
	6.5.1h		Item u, Page 17-41
	6.5.1i		Item a, Page 17-40
	6.5.1j		Item z, Page 17-41
	6.5.1k		Item p, Page 17-41
	6.5.1l		Item aa, Page 17-41
	6.5.1m		Item ab, Page 17-41
	6.5.1n		Item ac, Page 17-42
	6.5.1o		Items a,b,c; Page 17-42
30	6.5.2	Records	17.3.2.15, Pages 17-40 thru 17-44
	6.5.2a		Item m, Page 17-41
	6.5.2b		Item m, Page 17-41
	6.5.2c		Item m, Page 17-41
	6.5.2d		Item j, Page 17-41
	6.5.2e		Items o, Page 17-41; a-f, Page 17-42
	6.5.2f		Items r, t; Page 17-41
	6.5.2g		Item n, Page 17-41
	6.5.2h		Item ad, page 17-42
	6.5.2i		Item g, Page 17-41
	6.5.2j		Item af, Page 42
	6.5.2k		Item ae, Page 17-42

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>McGuire</u>			
31	6.2.3.1 6.2.3.2 6.2.3.3a-e 6.2.3.4	SRG	17.3.3.2.4, Pages 17-48, 49
32	6.2.3.5	SRG Records	17.3.2.15, Pages 17-42, 43
33	6.5.1	Technical Review and Control (TR&C)	17.3.2.14, Pages 17-37, 38, 39, 40
34	6.5.1.1	TR&C Procedures and Programs	17.3.2.14, Pages 17-38, 39, 40
35	6.5.1.2	TR&C Modifications	17.3.2.2, Pages 17-23, 24
36	6.5.1.3	TR&C Qualified Reviewers	17.3.2.14, Page 17-39
37	6.5.1.4	TR&C FOL/TS Changes	17.3.2.14, Page 17-37
38	6.5.1.5	TR&C Tests and Experiments	17.3.2.10, Page 17-33
39	6.5.1.6	TR&C Reportable Events TS Violations	17.3.2.13, Pages 17-35, 36
40	6.5.1.7	TR&C Special Reviews and Investigations	17.3.2.13, Pages 17-35, 36
41	6.5.1.8	TR&C Unplanned Onsite Releases	17.3.2.13, Page 17-36

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>McGuire</u>			
42	6.5.1.9	TR&C PCP, ODCM, Radwaste Systems	17.3.2.14, Page 17-39
43	6.5.1.10	TR&C Fire Protection	17.3.2.14, Page 17-39
44	6.5.1.11	TR&C Records	17.3.2.15, Pages 17-40 thru 17-44
45	6.5.2.1 6.5.2.2 6.5.2.3 6.5.2.4 6.5.2.5 6.5.2.6 6.5.2.7 6.5.2.8a-i	NSRB	17.3.3.2.1, Pages 17-45, 46
46	6.5.2.9 Items a-m	Audits	17.3.3.2.3, Pages 17-47, 48
47	6.5.2.10	NSRB Authority	17.3.3.2.1, Page 17-46
48	6.5.2.11a	NSRB Minutes	17.3.3.2.1, Page 17-46
49	6.5.2.11b	NSRB Reports of Reviews 14 Days	17.3.3.2.1, Page 17-46
50	6.5.2.11c	Audit Reports Forwarded in 30 Days	17.3.3.2.3, Page 17-48
51	6.6.1.b	Reportable Events	17.3.2.13, Pages 17-35, 36
52	6.7.1c	Safety Limit Violation	17.3.2.13, Pages 17-35, 36

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>McGuire</u>			
53	6.8.1e	PCP Implementing Procedures	17.3.2.14, Page 17-39
54	6.8.1h	TR&C Implementing Procedures	17.3.2.14, Pages 17-37, 38, 39
55	6.8.1j	PORC Implementing Procedures	17.3.2.14, Page 17-39
56	6.8.2	Procedures Review and Approval	17.3.2.14, Pages 17-39, 40
57	6.8.3	Temporary Changes to Procedures	17.3.2.14, Pages 17-39, 40
58	6.10.1	Records	17.3.2.15, Pages 17-40 thru 44
	6.10.1a		Item m, Page 17-41
	6.10.1b		Item j,o,v; Page 17-41
	6.10.1c		Items r, t; Page 17-41
	6.10.1d		Items h,n,o,v; Page 17-41,
	6.10.1e		Item p, Page 17-41
	6.10.1f		Item e, Page 17-41
	6.10.1g		Item af, Page 17-42
	6.10.1h		Item ag, Page 17-42

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>McGuire</u>			
59	6.10.2	Records	17.3.2.15, Pages 17-40 thru 17-44
	6.10.2a		Item b, Page 17-40
	6.10.2b		Item ah, Page 17-42
	6.10.2c		Item d, Page 17-41
	6.10.2d		Item e, Page 17-41
	6.10.2e		Item al, Page 17-42
	6.10.2f		Item z, Page 17-41
	6.10.2g		Item g, Page 17-41
	6.10.2h		Item u, Page 17-41
	6.10.2i		Item ai, Page 17-42
	6.10.2j		Items a,b,c; Page 17-42
	6.10.2k		Item ab, Page 17-41
	6.10.2l		Item aj, Page 17-42
	6.10.2m		Item ak, Page 17-42
	6.10.2n		Item ac, Page 17-42
60	6.10.3	Records required by QA Topical Report	17.3.2.15, Pages 17-40 thru 44

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Catawba</u>			
61	6.2.3.1 6.2.3.2 6.2.3.3 6.2.3.4	SRG	17.3.3.2.4, Pages 17-48, 49
62	6.2.3.5	SRG Records	17.3.2.15, Pages 17-42, 43
63	6.5.1	Technical Review and Control (TR&C)	17.3.2.14, Pages 17-37, 38, 39, 40
64	6.5.1.1	TR&C Procedures and Programs	17.3.2.14, Pages 17-38, 39, 40
65	6.5.1.2	TR&C Modifications	17.3.2.2, Pages 17-23, 24
66	6.5.1.3	TR&C Qualified Reviewers	17.3.2.14, Page 17-39
67	6.5.1.4	TR&C FOL/TS Changes	17.3.2.14, Page 17-37
68	6.5.1.5	TR&C Tests and Experiments	17.3.2.10, Page 17-33
69	6.5.1.6	TR&C Reportable Events TS Violations	17.3.2.13, Pages 17-35, 36
70	6.5.1.7	TR&C Special Reviews and Investigations	17.3.2.13, Pages 17-35, 36
71	6.5.1.8	TR&C Unplanned Onsite Releases	17.3.2.13, Page 17-36

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Catawba</u>			
72	6.5.1.9	TR&C PCP, ODCM, Radwaste Systems	17.3.2.14, Page 17-39
73	6.5.1.10	TR&C Fire Protection	17.3.2.14, Page 17-39
74	6.5.1.11	TR&C Records	17.3.2.15, Pages 17-40 thru 17-44
75	6.5.2.1 6.5.2.2 6.5.2.3 6.5.2.4 6.5.2.5 6.5.2.6 6.5.2.7 6.5.2.8a-i	NSRB	17.3.3.2.1, Pages 17-45, 46
76	6.5.2.9 Items a-m	Audits	17.3.3.2.3, Pages 17-47, 48
77	6.5.2.10a	NSRB Minutes	17.3.3.2.1, Page 17-46
78	6.5.2.10b	NSRB Reports of Reviews 14 Days	17.3.3.2.1, Page 17-46
79	6.5.2.10c	Audit Reports Forwarded in 30 Days	17.3.3.2.3, Page 17-48
80	6.6.1.b	Reportable Events	17.3.2.13, Pages 17-35, 36
81	6.7.1c	Safety Limit Violation	17.3.2.13, Pages 17-35, 36

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Catawba</u>			
82	6.8.1e	PCP Implementing Procedures	17.3.2.14, Page 17-39
83	6.8.1h	TR&C Implementing Procedures	17.3.2.14, Pages 17-37, 38, 39
84	6.8.1j	PORC Implementing Procedures	17.3.2.14, Page 17-39
85	6.8.2	Procedures Review and Approval	17.3.2.14, Pages 17-39, 40
86	6.8.3	Temporary Changes to Procedures	17.3.2.14, Pages 17-39, 40
87	6.10.1	Records	17.3.2.15, Pages 17-40 thru 44
	6.10.1a		Item m, Page 17-41
	6.10.1b		Item j,o,v; Page 17-41
	6.10.1c		Item r, t; Page 17-41
	6.10.1d		Item h,n,o,v; Page 17-41
	6.10.1e		Item p, Page 17-41
	6.10.1f		Item e, Page 17-41
	6.10.1g		Item af, Page 17-42
	6.10.1h		Item ag, Page 17-42

Attachment 2

<u>Item #</u>	<u>Current TS #</u>	<u>Subject</u>	<u>Location in Amendment 23 (Section/Page Number)</u>
<u>Catawba</u>			
88	6.10.2	Records	17.3.2.15, Pages 17-40 thru 17-44
	6.10.2a		Item b, Page 17-40
	6.10.2b		Item ah, Page 17-42
	6.10.2c		Item d, Page 17-41
	6.10.2d		Item e, Page 17-41
	6.10.2e		Item al, Page 17-42
	6.10.2f		Item z, Page 17-41
	6.10.2g		Item g, Page 17-41
	6.10.2h		Item u, Page 17-41
	6.10.2i		Item ai, Page 17-42
	6.10.2j		Items a,b,c; Page 17-42
	6.10.2k		Item ab, Page 17-41
	6.10.2l		Item aj, Page 17-42
	6.10.2m		Item ak, Page 17-42
	6.10.2n		Item am, Page 17-42
89	6.10.3	Records required by QA Topical Report	17.3.2.15, Pages 17-40 thru 44
90	6.14.2a.3	Documentation of Review of Changes to ODCM	17.3.2.14, Page 17-39
91	6.16.2b	Review by Qualified Individual of Changes to Radiological Effluent Controls	17.3.2.14, Page 17-39

Attachment 3

Current Technical Specifications for
Oconee, McGuire, and Catawba Nuclear Stations

Marked Pages Showing Items for Relocation to the
Topical Report



Indicates Item for Relocation to the Topical Report

Oconee Nuclear Station: Items 1-30
McGuire Nuclear Station: Items 31-60
Catawba Nuclear Station: Items 61-91

(A1) (except as marked) Specification 2.0

(M2)

restore to within limits and

in MODE 3 within 1 hour

(L3)

2.2 ~~6.3~~

ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED

2.2.1 ~~6.3.1~~

2.2.2

If a safety limit is exceeded, the reactor shall be shut down immediately and maintained in a safe shutdown condition until the Commission authorizes resumption of operation.

(L3)

(A5)

6.3.2

The violation of a safety limit shall be reported to the Commission, the Site Vice President and the Director of the Nuclear Safety Review Board.

(LA2)

6.3.3

A report shall be prepared which describes (1) applicable circumstances preceding the violation, (2) effects of the violation upon structures, systems, or components, and (3) corrective action taken to prevent recurrence. The report shall be reviewed by the Operations Superintendent and the Station Manager. The report shall be submitted to the Site Vice President and the Director of the Nuclear Safety Review Board.

6.3.4

A report of the violation, with appropriate analyses and corrective action to prevent recurrence shall be submitted to the Commission within 30 days of the violation.

(L2)

Add 2.2.3

(M3)

Add Note to SR 3.4.14.1

M39

4.5.1.1.3 Core Flooding System

- a. During each refueling outage, a system test shall be conducted to demonstrate proper operation of the system. During pressurization of the Reactor Coolant System, verification shall be made that the check and isolation valves in the core flooding tank discharge lines operate properly.
- b. The test will be considered satisfactory if control board indication of core flood tank level verifies that all valves have opened.

SC 3.5

4.5.1.2 Component Tests

4.5.1.2.1 Valves - Power Operated

- a. Valves LP-17, -18, shall only be tested every cold shutdown unless previously tested during the current quarter.
- b. During each refueling outage the following LPI system valves shall be cycled manually to verify the manual operability of these power operated valves:

- (1) LPI pump discharge (ES) LP-17,-18
- (2) LPI discharge throttling LP-12,-14
- (3) LPI discharge header crossover LP-9,-10
- (4) LPI discharge to HPI/RBS LP-15,-16

M39

M40

L3

4.5.1.2.2 Check Valves

PLUS

Periodic individual leakage testing* of ~~valves LP-12, LP-14, LP-17 and LP-18~~ shall be accomplished prior to ~~power operation~~ after every time the plant is placed in the cold shutdown condition for refueling, after each time the plant is placed in a cold shutdown condition for ~~72 hours~~ if testing has not been accomplished in the preceding 9 months, and prior to returning the valve to service after maintenance, repair or replacement work is performed. Whenever integrity of these valves cannot be demonstrated, the integrity of the remaining valve in each high pressure line having a leaking valve shall be determined and recorded daily. In addition, the position of the other closed valve located in the high pressure piping shall be recorded daily. ~~For the allowable leakage rates and limiting conditions for operation, see Technical Specification 3.1.6.10.~~

7 days

LA13

entering Mode 2

SR 3.4.14.1

2

Bases

The Emergency Core Cooling Systems are the principle reactor safety features in the event of loss of coolant accident. The removal of heat from the core provided by these systems is designed to limit core damage.

A2

The High Pressure Injection System under normal operating conditions has one pump operating. The HPI system test required by Specification 4.5.1.1.1 verifies that the HPI system responds as required to actuation of ES channels 1 and 2.

LA7

- (a) To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.

Oconee 1, 2, and 3

Amendment No. 217(Unit 1)
Amendment No. 217(Unit 2)
Amendment No. 214(Unit 3)

4.5-2

3 of 3

(A1) <Except as marked>

~~4.7 REACTOR CONTROL ROD SYSTEM TESTS~~~~4.7.1 Control Rod Trip Insertion Time Test~~Applicability

Applies to the surveillance of the control rod trip insertion time.

Objective

To assure the control rod trip insertion time is within that used in the safety analyses.

Specification

<SR 3.1.4.3>

Add SR 3.1.4.3 Note

(L6)

The control rod insertion time shall be measured at either full flow or no flow conditions as follows:

- a. For all rods following each removal of the reactor vessel head.
- b. For specifically affected individual rods following any maintenance on or modification to the control rod drive system which could affect the drop time of those specific rods, and
- c. For all rods at least once following each refueling outage.

prior to reactor criticality

(M11)

prior to reactor criticality

(M11)

The maximum control rod trip insertion time for an operable control rod drive mechanism, except for the Axial Power Shaping Rods (APSRs), from the fully withdrawn position to 3/4 insertion (104 inches travel) shall not exceed 1.66 seconds at reactor coolant ~~(full)~~ flow conditions ~~or 1.40 seconds for no flow conditions~~. For the APSRs it shall be demonstrated that loss of power will not cause rod movement.

If the trip insertion time above is not met, the rod shall be declared inoperable.

For Unit 1 Cycle 15, Group 1, Rod 8 and Group 2, Rod 5 may be considered operable with an insertion time ≤ 3.00 sec provided:

- 1) the average insertion time for the remaining rods in Groups 1 and 2 is ≤ 1.50 sec. and
- 2) the core average negative reactivity insertion rate is within the assumptions of the safety analysis.

Bases

The control rod trip insertion time is the total elapsed time from power interruption at the control rod drive breakers until the control rod has completed 104 inches of travel from the fully withdrawn position. The specified trip time is based upon the safety analysis in FSAR Chapter 15.

A rod is considered inoperable if the trip insertion time is greater than the specified allowable time or the core average negative reactivity insertion rate is less than the assumptions of the safety analysis.

REFERENCES

- (1) FSAR, Section 15
- (2) Technical Specification 3.5.2

(A1) (Except as marked)

4.20.3 SSF Electrical Power Systems

a. Diesel Generator

Frequency for SR 3.10.1.3, 4+5

3

1. Monthly, or after maintenance or modification that could affect its operability the SSF diesel generator shall be verified operable by:

LA5

SR 3.10.1.3

- a. Verifying the fuel inventory in the day tank is greater than or equal to 200 gallons and,

SR 3.10.1.4

- b. Verifying the fuel inventory in the underground oil storage tank is greater than or equal to 25,000 gallons and,

LI

Add SR 3.10.1.5 Note

SR 3.10.1.5

- c. Verifying the diesel starts from standby condition and runs according to the procedures and requirements recommended by the manufacturer.

LA4

MI

Add SR 3.10.1.6

2. ~~Quarterly~~ ^{Every 92 days} verify that:

A7

SR 3.10.1.9

SR 3.10.1.9 Note 3

A9

Add SR 3.10.1.9 Notes 1+2

- a. The SSF diesel generator can be operated for a least 60 minutes with a load of greater than or equal to 3000 KW. This test may be preceded by an engine prelube period and/or other warm-up procedures recommended by the manufacturer.

synchronized and

M5

SR 3.10.1.7

- b. The fuel oil transfer pump starts and transfers fuel from the storage system to the day tank. This test will be performed per Specification 4.20.1.a

Automatically

A10

SR 3.10.1.8

3. ~~Quarterly~~ ^{Every 92 days} diesel fuel from the day tank and the underground storage tank shall be sampled and analyzed for viscosity, water and sediment in accordance with applicable ASTM Specifications for Diesel Fuel Oil.

Verify the fuel oil properties of

A8

LA3

SEE 5.0

4. Annually, the SSF diesel generator shall be demonstrated operable by subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.

OCONEE 1, 2, and 3

4.20-2

Amendment No. 195 (Unit 1)
Amendment No. 195 (Unit 2)
Amendment No. 192 (Unit 3)

A8

As tested in accordance with, and maintained within the limits of the Diesel Fuel Oil Testing Program

(A1) (except as marked)

General Manager, Nuclear Services

(A27)

[5.3.1]

the ~~Nuclear Technical Services Manager~~ and as approved by the Site Vice President, Nuclear Operations.

The Operations Superintendent shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two years of the remaining five years of experience may be fulfilled by academic training, or related technical training, on a one-for-one time basis. ~~The Operations Superintendent shall hold or have held a Senior Reactor Operator license.~~

[5.2.2.F]

[5.3.1]

The Shift Operations Manager shall have a minimum of eight years of responsible nuclear or fossil station experience, of which a minimum of three years shall be nuclear station experience. A maximum of two years of the remaining five years of experience may be fulfilled by academic training, or related technical training on a one-for-one time basis. The Shift Operations Manager shall hold a Senior Reactor Operator license.

(L3)

[5.2.2.F]

[5.2.1.e] 6.1.1.5

The individuals who train the operating staff and those who carry out radiation protection and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.1.1.6 Minimum operating shift crew requirements shall be as specified in Table 6.1-1.

(A1)

6.1.1.7 Retraining and replacement of station personnel shall be in accordance with Section 5.5 of the ANSI/ANS-3.1-1978, "Selection and Training of Nuclear Power Plant Personnel."

6.1.1.8 A training program for the fire brigade shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except that training sessions may be held quarterly.

(LAS)

6.1.1.9

[5.2.2.g]

The Shift Manager is an experienced SRO who has been instructed in additional academic subjects, and will be assigned on-shift to provide the accident assessment capability.

(LA7)

The operating experience assessment function will be provided by the Station Safety Review Group.

(LA6)

4

Insert
5.2.2.g from ITS

(M10)

(A1) <except as marked>

Specification S.0

6.1.2 TECHNICAL REVIEW AND CONTROL

5 6.1.2.1 Activities

LA 7
Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.1.2.1a through 6.1.2.1k. Approvals shall be by the head of the appropriate site organization, the head of the appropriate station organization, the head of the appropriate site engineering organization, the head of the environmental organization, or an alternate as specified in other applicable regulatory documents or administrative controls..

- 6 a. Each procedure and program required by Specification 6.4 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a knowledgeable individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/organization other than the individual/organization which prepared the procedure, or changes thereto. Procedures, or changes thereto, shall be approved in accordance with Specifications 6.4.2 and 6.4.3.
- 7 b. Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a knowledgeable individual/organization. Each modification shall be reviewed by an individual/organization other than the individual/organization which designed the modification.
- 8 c. Individuals responsible for reviews performed in accordance with Specifications 6.1.2.1.a and 6.1.2.1.b shall be members of the supervisory staff assigned to the site, and previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel..
- 9 d. Proposed changes to the Technical Specifications shall be prepared by a knowledgeable individual/organization. The preparation of each proposed Technical Specification change shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed change.

6.1-2

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

43069

(A1) <except as marked>

Specification 5.0

The station manager or his designee shall approve

A8

+ modifications

A8

5.1.1

prior to implementation

e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the PSR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.1.2.1.a. These proposed tests and experiments shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed tests and experiments.

10

LA7

MZ

11

f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.

12

g. Special reviews and investigations, and the preparation and submittal of reports thereon, shall be performed by a knowledgeable individual/organization.

13

h. A knowledgeable individual/organization shall review every unplanned onsite release of radioactive material to the environs, and prepare reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence.

LA7

14

i. A knowledgeable individual/organization shall review changes to the Process Control Program, Offsite Dose Calculation Manual (ODCM), and Radwaste Treatment Systems.

15

j. A knowledgeable individual/organization shall review the Fire Protection Program and implementing procedures.

16

k. Reports documenting each of the activities performed under Specifications 6.1.2.1.a through 6.1.2.1.j shall be maintained.

6.1-3

OCOMEX UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

44 of 68

6.1.3 Nuclear Safety Review Board (NSRB)

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

6.1.3.2 Organization

- a. The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.1.3.1.
- b. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.1.3.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.
- c. The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Site staff.

6.1-4

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
amendment No. 208 (Unit 3)

45 of 68

- 17
- d. Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.
 - e. Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.
 - f. The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least twice per year thereafter.
 - g. The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.

6.1.3.3 Review

The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
 - b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
 - c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
 - d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
 - e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- LAB

6.1-4a

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
amendment No. 208 (Unit 3)

46 of 68

Specifications 5.0

17

- f. ALL REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i. Reports of activities performed under the provisions of Specifications 6.1.2.1a through 6.1.2.1j.

LAB

6.1-4b

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

47 of 68

18

6.1.3.4 Audits

Audits of Site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of station operation to provisions contained within the Technical Specifications and applicable facility operating license conditions.
- b. The performance, training and qualifications of the station staff.
- c. The results of actions taken to correct deficiencies occurring in equipment, structures, systems or methods of operation that affect nuclear safety.
- d. The performance of activities required by the quality assurance program to meet the criteria of Appendix B to 10 CFR 50.
- e. The station emergency plan and implementing procedures.
- f. The station security plan and implementing procedures.
- g. Any other area of station operation considered appropriate by the NSRB or the Senior Vice President, Nuclear Generation.
- h. The station fire protection program and implementing procedures.
- i. The Offsite Dose Calculation Manual and implementing procedures.
- j. The Radiological Environmental Monitoring Program and the results thereof.
- k. The Process Control Program and implementing procedures for solidification of radioactive wastes.
- l. The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 1.21 Revision 1, June 1974 and Regulatory Guide 4.1 Revision 1, April 1975.

LA8

6.1.3.5 Responsibilities and Authorities

- 19 a. The NSRB shall report to and advise the Senior Vice President, Nuclear Generation on those areas of responsibility specified in Specifications 6.1.3.3 and 6.1.3.4.
- 20 b. Minutes shall be prepared and forwarded to the Senior Vice President, Nuclear Generation, and to the Site Vice President within 14 days following each formal meeting of the NSRB.
- 21 c. Records of activities performed in accordance with Specifications 6.1.3.3 and 6.1.3.4 shall be maintained.
- 22 d. Audit reports encompassed by Section 6.1.3.4 shall be forwarded to the Senior Vice President, Nuclear Generation and to the Site Vice President and to the management position responsible for the areas audited within 30 days of completion of each audit.

LA8

Specification 5.0

(A1) <except as marked>

23

6.2 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE OCCURRENCE

LA9

6.2.1 The Manager, Safety Assurance shall assure that any reportable event is promptly investigated.

6.2.2 The Site Vice President shall be notified of any reportable event. A written report shall be prepared which describes the circumstances leading up to and resulting from the incident and shall recommend appropriate action to prevent or minimize the probability of a recurrence. The report shall be submitted to the Site Vice President, and the Director of the Nuclear Safety Review Board.

6.2.3 The Commission shall be notified and/or a report submitted pursuant to the requirements of specification 6.6.2 and 10 CFR 50.73.

A25

10 CFR 50.36

10 CFR 50.73

OCONEE UNITS 1,2,3

6.2-1

Amendment No. 193 (Unit 1)

Amendment No. 193 (Unit 2)

Amendment No. 190 (Unit 3)

52 OF 68

(A1) <except as marked>

5.4.1.a (k). Long-term emergency core cooling systems. Procedures shall include provision for remote or local operation of system components necessary to establish high and low pressure injection within 15 minutes after a line break.

(A9)

LA12

5.4.1.a (l). Fire Protection Program implementation.

(A9)

5.4.1.e (m). Offsite Dose Calculation Manual implementation.

(A24)

24 (n). Process Control Program implementation.

LA11

25 (o). Technical Review and Control Program implementation.

26 (p). Plant Operations Review Committee implementation.

27

6.4.2 Each procedure of specification 6.4.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

LA7

28

6.4.3 Temporary changes to procedures of Specification 6.4.1 above may be made provided:

- The intent of the original procedure is not altered;
- The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operators license on the affected unit; and
- The change is approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports within 14 days of implementation.

Add 5.4.1.d

(M11)

Add 5.4.1.c

6.4-2

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

(M4)

(4) (except as marked)

6.5 STATION OPERATING RECORDS

Specification

29

6.5.1 The following records shall be prepared and permanently retained in a manner convenient for review.

- a. Records of modifications to the station as described in the FSAR.
- b. Special nuclear material physical inventory records.
- c. Special nuclear material isotopic inventory records.
- d. Radiation monitoring records, including records of radiation and contamination surveys.
- e. Records of off-site environmental surveys.
- f. Personnel radiation exposure records as required by 10CFR20.
- g. Records of radioactive releases and waste disposal.
- h. Records of reactor coolant system in-service inspections.
- i. Preoperational testing records.
- j. Records of special reactor tests or experiments.
- k. Records of changes safety-related operating procedures.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.7.
- m. Records of the seal service lives of hydraulic snubbers.
- n. Records of reviews performed for changes made to the ODCM and Process Control Program.
- o. Records of meetings of the NSRB and reports required by Specification 6.1.2.1k.

(LAIS)

6.5-1

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
 Amendment No. 211 (Unit 2)
 Amendment No. 208 (Unit 3)

59668

(A) Except as marked

30

6.5.2

The following records shall be prepared and retained for a minimum of six (6) years in a manner convenient for review:

- a. Switchboard Record.
- b. Reactor Operations Logbook.
- c. Shift Supervisor Logbook.
- d. Maintenance histories for station safety-related structures, systems and components.
- e. Record of safety-related inspections, other than reactor coolant system in-service inspections.
- f. Records of reportable events.
- g. Periodic testing records and records of other periodic checks, calibrations, etc., performed in accordance with surveillance requirements for safety-related parameters, structures, systems and components.
- h. By-product material inventory records.
- i. Training records.
- j. Test results, in units of microcuries, for leak tests performed pursuant to Specification 4.16.
- k. Radioactive liquid effluent, gaseous effluent, and gaseous process monitoring instrumentation alarm/trip setpoints.

LAIS

6.5-2

OCONEE UNITS 1,2,3

Amendment No. 211 (Unit 1)
Amendment No. 211 (Unit 2)
Amendment No. 208 (Unit 3)

60068

LA3

ADMINISTRATIVE CONTROLS

6.2.3 MCGUIRE SAFETY REVIEW GROUP

FUNCTION

31 6.2.3.1 The McGuire Safety Review Group (SRG) shall function to provide review of plant design and operating experience for potential opportunities to improve plant safety; evaluation of plant operations and maintenance activities; and, to advise management on the overall quality and safety of plant operations. The SRG shall make recommendations for revised procedures, equipment modifications, or other means of improving plant safety to appropriate station/corporate management.

COMPOSITION

31 (LA.3) 6.2.3.2 The SRG shall be composed of at least five individuals and at least three of these shall have a bachelor's degree in engineering or related science and at least 2 years professional level experience in his/her field, at least 1 year of which experience shall be in the nuclear field.

The remaining individuals in the SRG shall have either (1) at least 5 years of nuclear experience and hold or have held a Senior Reactor Operator license; or (2) at least 8 years of professional level experience in his/her field, at least 5 years of which experience shall be in the nuclear field.

RESPONSIBILITIES

31 6.2.3.3 The SRG shall be responsible for:

- a. Review of selected plant operating characteristics and other appropriate sources of plant design and operating experience information for awareness and incorporation into the performance of other duties
- b. Review of the effectiveness of corrective actions taken as a result of the evaluation of selected plant operating characteristics and other appropriate sources of plant design and operating experience information.
- c. Review of selected programs, procedures, and plant activities, including maintenance, modification, operational problems, and operational analysis.
- d. Surveillance of selected plant operations and maintenance activities to provide independent verification* that they are performed correctly and that human errors are reduced to as low as practicable.
- e. Investigation of selected unusual events and other occurrences as assigned by Station Management or the Manager of Safety Assurance.

*Not responsible for sign-off function.

ADMINISTRATIVE CONTROLS

6.2.3 MCGUIRE SAFETY REVIEW GROUP (Continued)

AUTHORITY

6.2.3.4 The SRG shall report to and advise the Manager of Safety as those areas of responsibility specified in Section 6.2.3.

RECORDS

6.2.3.5 Records of activities performed by the SRG shall be prepared and maintained for the life of the station. Summary reports of activities performed by the SRG shall be forwarded each calendar month to the Manager of Safety Assurance.

6.2.4 SHIFT MANAGER

6.2.4.1 The Shift Manager, whose functions include those of a Shift Technical Advisor, shall serve in an advisory capacity to the Shift Supervisor.

5.3 UNIT STAFF QUALIFICATIONS

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, except for the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the SRG.

ADMINISTRATIVE CONTROLS

Sp

6.5 REVIEW AND AUDIT

6.5.1 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.5.1.1 through 6.5.1.11. Approvals shall be by the head of the appropriate site organization, the head of the appropriate station organization, the head of the appropriate site engineering organization, the head of the environmental organization, or an alternate as specified in other applicable regulatory documents or administrative controls.

6.5.1.1 Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a knowledgeable individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/organization other than the individual/organization which prepared the procedure, or changes thereto. Procedures, or changes thereto, shall be approved in accordance with Specifications 6.8.2 and 6.8.3.

6.5.1.2 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a knowledgeable individual/organization. Each such modification shall be reviewed by an individual/organization other than the individual/organization which designed the modification.

6.5.1.3 Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1 and 6.5.1.2 shall be members of the supervisory staff assigned to the site, previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel.

6.5.1.4 Proposed changes to the Appendix A Technical Specifications shall be prepared by a knowledgeable individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed change.

6.5.1.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.5.1.1. These proposed tests and experiments shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the proposed tests and experiments.

ADMINISTRATIVE CONTROLS

ACTIVITIES (Continued)

39 6.5.1.6 ALL REPORTABLE EVENTS and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence which provides recommendations to prevent recurrence. Such reports reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.

40 6.5.1.7 Special reviews and investigations, and the preparation of reports thereon, shall be performed by a knowledgeable individual/organization.

LA. 6 41 6.5.1.8 A knowledgeable individual/organization shall review every unplanned onsite release of radioactive material to the environs and prepare reports covering evaluation, recommendations, and disposition of the corrective ACTION to prevent recurrence.

42 6.5.1.9 A knowledgeable individual/organization shall review changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems.

43 6.5.1.10 A knowledgeable individual/organization shall review the Fire Protection Program and implementing procedures.

44 6.5.1.11 Reports documenting each of the activities performed under Specifications 6.5.1.1 through 6.5.1.10 shall be maintained.

6.5.2 NUCLEAR SAFETY REVIEW BOARD (NSRB)

FUNCTION

LA. 7 45 6.5.2.1 The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,

ADMINISTRATIVE CONTROLSFUNCTION (Continued)

- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

ORGANIZATION

45

LA.7

45

45

45

45

45

6.5.2.2 The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.5.2.1. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.5.2.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.

6.5.2.3 The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the McGuire Nuclear Site staff.

6.5.2.4 Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.

6.5.2.5 Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.

6.5.2.6 The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least twice per year thereafter.

6.5.2.7 The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of McGuire Nuclear Station.

ADMINISTRATIVE CONTROLS

LA-7

REVIEW

45

6.5.2.8 The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, systems, and (2) tests or experiments completed under the provisions of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- f. ALL REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i. Reports of activities performed under the provisions of Specifications 6.5.1.1 through 6.5.1.10.

LA-7

AUDITS

46

6.5.2.9 Audits of site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions;
- b. The performance, training, and qualifications of the entire station staff;

ADMINISTRATIVE CONTROLS

LA7

AUDITS (Continued)

- c. The results of actions taken to correct deficiencies occur unit equipment, structures, systems, or method of operation affect nuclear safety;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50;
- e. The Emergency Plan and implementing procedures;
- f. The Security Plan and implementing procedures;
- g. The Facility Fire Protection programmatic controls including the implementing procedures;
- h. The fire protection equipment and program implementation utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- i. The Radiological Environmental Monitoring Program and the results thereof;
- j. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures;
- k. The PROCESS CONTROL PROGRAM and implementing procedures for SOLIDIFICATION of radioactive wastes;
- l. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring and;
- m. Any other area of site operation considered appropriate by the NSRB or the Senior Vice President, Nuclear Generation.

LA7

47

AUTHORITY

6.5.2.10 The NSRB shall report to and advise the Senior Vice President, Nuclear Generation on those areas of responsibility specified in Specifications 6.5.2.8 and 6.5.2.9.

ADMINISTRATIVE CONTROLS

LA7

RECORDS

6.5.2.11 Records of NSRB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each NSRB meeting shall be prepared, approved, and forwarded to the Senior Vice President, Nuclear Generation and to the Site Vice President, within 14 days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.2.8 above, shall be prepared, approved and forwarded to the Senior Vice President, Nuclear Generation, and to the Site Vice President, within 14 days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the Senior Vice President, Nuclear Generation and to the Site Vice President, and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the Station Manager; or for the Station Manager by: (1) the Operations Superintendent, (2) the Maintenance Superintendents, or (3) the Work Control Superintendent, as previously designated by the Station Manager, and the results of the review shall be submitted to the NSRB and the Site Vice President.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Site Vice President, and the NSRB shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Operations Superintendent and the Station Manager. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence;

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

LAB

52

LA.9

- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRB and the Site Vice President, within 14 days of violation; and
- d. Critical operation of the unit shall not be resumed until approved by the Commission.

5.4 (6.8) PROCEDURES AND PROGRAMS

5.4 (6.8.1) Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The applicable ^{Emergency Operating} procedures required to implement the requirements of NUREG-0737; and to NUREG-0737, Supplement 1, as stated in Generic Letter 82-33

- c. Deleted
- d. Deleted

LA.10

e. PROCESS CONTROL PROGRAM implementation;

53

f. OFFSITE DOSE CALCULATION MANUAL implementation; and

g. Quality Assurance Program for effluent and environmental monitoring.

h. Technical Review and Control Program implementation.

54

i. Fire Protection Program implementation.

A.1

j. Plant Operations Review Committee implementation.

55

k. Commitments contained in SAR Chapter 16.0

M.1
All programs
specified in
Specification 5.5.

56

LA.6

6.8.2 Each procedure of Specification 6.8.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

57

LA.6

6.8.3 Temporary changes to procedures of Specification 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and

ADMINISTRATIVE CONTROLSSPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Regional Administrative NRC Regional Office within the time period specified for each report.

6.10 RECORD RETENTION

In addition to the applicable record retention requirements of Title of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.1 The following records shall be retained for at least 5 years:

- a. Records and logs of unit operation covering time interval at each power level;
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety;
- c. ALL REPORTABLE EVENTS;
- d. Records of surveillance activities; inspections and calibrations required by these Technical Specifications;
- e. Records of changes made to the procedures required by Specification 6.8.1;
- f. Records of radioactive shipments;
- g. Records of sealed source and fission detector leak tests and results; and
- h. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report;
- b. Records of new and irradiated fuel inventory, fuel transfers, and assembly burnup histories;
- c. Records of radiation exposure for all individuals entering radiation control areas;
- d. Records of gaseous and liquid radioactive material released to the environs;
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1;
- f. Records of reactor tests and experiments;

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- g. Records of training and qualification for current members staff;
- h. Records of inservice inspections performed pursuant to the Technical Specifications;
- i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- LA.18 j. Records of meetings of the NSRB and reports required by Specification 6.5.1.11;
- k. Records of the service lives of all snubbers including the date at which the service life commences and associated installation and maintenance records;
- l. Records of secondary water sampling and water quality; and
- m. Records of analyses required by the Radiological Environmental Monitoring Program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.
- n. Records of reviews performed for changes made to the ODCM and the PCP.

60 6.10.3 Records of quality assurance activities required by the QA Manual shall be retained for a period of time required by ANSI N45.2.9-1974.

6.11 RADIATION PROTECTION PROGRAM

LA.19 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

A.1

Pursuant to 10CFR20, paragraph 20.1601 (c),

5.7 6.12 HIGH RADIATION AREA

A1

requirements of 10CFR 20.1601

5.7.1 6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mrem/hr at 65 CM (26 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Radiation Protection Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mrem/hr provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

A.24 30 12

ADMINISTRATIVE CONTROLS6.2.3 CATAWBA SAFETY REVIEW GROUPFUNCTION

6.2.3.1 The Catawba Safety Review Group (SRG) shall function to provide the review of plant design and operating experience for potential opportunities to improve plant safety; evaluation of plant operations and maintenance activities; and, to advise management on the overall quality and safety of plant operations. The SRG shall make recommendations for revised procedures, equipment modifications, or other means of improving plant safety to appropriate station/corporate management.

COMPOSITION

6.2.3.2 The SRG shall be composed of at least five individuals and at least three of these shall have a bachelor's degree in engineering or related science and at least 2 years professional level experience in his/her field, at least 1 year of which experience shall be in the nuclear field.

The remaining individuals in the SRG shall have either (1) at least 5 years of nuclear experience and hold or have held a Senior Reactor Operator license; or (2) at least 8 years of professional level experience in his/her field, at least 5 years of which experience shall be in the nuclear field.

RESPONSIBILITIES

6.2.3.3 The SRG shall be responsible for:

- a. Review of selected plant operating characteristics and other appropriate sources of plant design and operating experience information for awareness and incorporation into the performance of other duties.
- b. Review of the effectiveness of corrective actions taken as a result of the evaluation of selected plant operating characteristics and other appropriate sources of plant design and operating experience information.
- c. Review of selected programs, procedures, and plant activities, including maintenance, modification, operational problems, and operational analysis.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (continued)

- d. Surveillance of selected plant operations and maintenance activities to provide independent verification* that they are performed correctly and that human errors are reduced to as low as practicable.
- e. Investigation of selected unusual events and other occurrences as assigned by Station Management or the Manager of Safety Assurance.

AUTHORITY

6.2.3.4 The SRG shall report to and advise the Manager of Safety Assurance on those areas of responsibility specified in Section 6.2.3.

RECORDS

6.2.3.5 Records of activities performed by the SRG shall be prepared and maintained for the life of the station. Summary reports of activities performed by the SRG shall be forwarded each calendar month to the Manager of Safety Assurance.

6.2.4 SHIFT MANAGER

6.2.4.1 The Shift Manager, whose functions include those of a Shift Technical Advisor, shall serve in an advisory capacity to the Shift Supervisor.

5.8.3 UNIT STAFF QUALIFICATIONS

5.8.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Manager, who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

*Not responsible for sign-off function

Except that the experience and other considerations described in Duke Power Company's letters dated August 28, 1985, and July 8, 1986, are acceptable for the six and two applicants for SRO licenses identified therein, respectively.

ADMINISTRATIVE CONTROLS6.5 REVIEW AND AUDIT6.5.1 TECHNICAL REVIEW AND CONTROL ACTIVITIES

63 Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.5.1.1. through 6.5.1.11. Approvals shall be by the head of the appropriate site organization, the head of the appropriate station organization, the head of the appropriate site engineering organization, the head of the environmental organization, or an alternate as specified in other applicable regulatory documents or administrative controls.

64 6.5.1.1 Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a knowledgeable individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/organization other than the individual/organization which prepared the procedure, or changes thereto. Procedures, or changes thereto, shall be approved in accordance with Specifications 6.8.2 and 6.8.3.

65 6.5.1.2 Proposed modifications to unit nuclear safety-related structures, systems, and components shall be designed by a knowledgeable individual/organization. Each such modification shall be reviewed by an individual/organization other than the individual/organization which designed the modification. LA-6

66 6.5.1.3 Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1 and 6.5.1.2 shall be members of the supervisory staff assigned to the site, previously designated by the Site Vice President to perform such reviews. Review of environmental radiological analysis procedures shall be performed by the General Manager, Environmental Services or his designee. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel.

67 6.5.1.4 Proposed changes to the Appendix A Technical Specifications shall be prepared by a knowledgeable individual/organization. The preparation of each proposed Technical Specification change shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed change.

68 6.5.1.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.5.1.1. These proposed tests and experiments shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the proposed tests and experiments.

ADMINISTRATIVE CONTROLSTECHNICAL REVIEW AND CONTROL ACTIVITIES (Continued)

69

6.5.1.6 All REPORTABLE EVENTS and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.

70

6.5.1.7 Special reviews and investigations, and the preparation of reports thereon, shall be performed by a knowledgeable individual/organization.

71

LA.6

6.5.1.8 A knowledgeable individual/organization shall review every unplanned onsite release of radioactive material to the environs and prepare reports covering evaluation, recommendations, and disposition of the corrective ACTION to prevent recurrence.

72

6.5.1.9 A knowledgeable individual/organization shall review changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems.

73

6.5.1.10 A knowledgeable individual/organization shall review the Fire Protection Program and implementing procedures.

74

6.5.1.11 Reports documenting each of the activities performed under Specifications 6.5.1.1 through 6.5.1.10 shall be maintained.

6.5.2 NUCLEAR SAFETY REVIEW BOARD (NSRB)

75

LA.7

FUNCTION

6.5.2.1 The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,

ADMINISTRATIVE CONTROLSFUNCTION (Continued)

- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

The NSRB shall report to and advise the Senior Vice President, Nuclear Generation on those areas of responsibility specified in Specifications 6.5.2.8 and 6.5.2.9.

ORGANIZATION

75
LA.7
75
6.5.2.2 The Director, members, and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.5.2.1. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.5.2.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.

75
6.5.2.3 The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the Catawba Nuclear Site staff.

75
6.5.2.4 Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.

75
6.5.2.5 Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.

75
6.5.2.6 The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least twice per year thereafter.

75
6.5.2.7 The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of Catawba Nuclear Station.

ADMINISTRATIVE CONTROLSREVIEW

75 6.5.2.8 The NSRB shall be responsible for the review of:

- LA.7
- a. The safety evaluation for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question.
 - b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
 - c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
 - d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
 - e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
 - f. ALL REPORTABLE EVENTS;
 - g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety;
 - h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
 - i. Reports of activities performed under the provisions of Specifications 6.5.1.1 through 6.5.1.10.

AUDITS

76 6.5.2.9 Audits of site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions;
- b. The performance, training, and qualifications of the entire station staff;

ADMINISTRATIVE CONTROLSAUDITS (Continued)

76

- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50;
- e. The Emergency Plan and implementing procedures;
- f. The Security Plan and implementing procedures;
- g. The Facility Fire Protection programmatic controls including the implementing procedures;
- h. The fire protection equipment and program implementation utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- i. The Radiological Environmental Monitoring Program and the results thereof;
- j. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures;
- k. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes;
- l. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring; and
- m. Any other area of site operation considered appropriate by the NSRB or the Senior Vice President, Nuclear Generation.

LA.7

RECORDS

77

6.5.2.10 Records of NSRB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each NSRB meeting shall be prepared, approved, and forwarded to the Senior Vice President, Nuclear Generation and to the Site Vice President, within 14 days following each meeting;

ADMINISTRATIVE CONTROLSRECORDS (Continued)

78

LA.7

79

- b. Reports of reviews encompassed by Specification 6.5.2.8 above, shall be prepared, approved, and forwarded to the Senior Vice President, Nuclear Generation, and to the Site Vice President, within 14 days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.2.9 above, shall be forwarded to the Senior Vice President, Nuclear Generation, and to the Site Vice President, and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

LA.8

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

80

LA.6

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50 and
- b. Each REPORTABLE EVENT shall be reviewed by the Station Manager; or for the Station Manager by the (1) Operations Superintendent; (2) Work Control Superintendent; (3) Mechanical Maintenance Superintendent; or (4) I and E/Maintenance Support Superintendent, as previously designated by the Station Manager, and the results of this review shall be submitted to the NSRB and the Site Vice President.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

LA.9

81

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Site Vice President and the NSRB shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Operations Superintendent and Station Manager. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRB and the Site Vice President within 14 days of the violation; and
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

ADMINISTRATIVE CONTROLS

A.1 5.4 ~~6.8~~ PROCEDURES ~~AND PROGRAMS~~

5.4 (6.8.1) Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement No. 1 to NUREG-0737 as stated in Generic Letter No. 82-33;

A.1 c. Deleted
d. Deleted

M.1

E. All programs specified in Specification 5.5.

82

LA.10 e. ~~PROCESS CONTROL PROGRAM implementation;~~

f. ~~OFFSITE DOSE CALCULATION MANUAL implementation;~~

A.1 C.8. Quality Assurance Program implementation for effluent and environmental monitoring;

83

LA.6 h. ~~Technical Review and Control Program implementation.~~

A.35 i. ~~Fire Protection Program implementation;~~

84

LA.6 j. ~~Plant Operations Review Committee implementation.~~

A.1 k. Commitments contained in ~~USAR~~ Chapter 16.0. ~~and~~

85

6.8.2 Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

LA.6

86

6.8.3 Temporary changes to procedures of Specification 6.8.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and
- c. The change is approved by an appropriate division manager, superintendent/manager or one of their designated direct reports within 14 days of implementation.

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

14. BAW-10162P-A, TAC03 Fuel Pin Thermal Analysis Computer Code, B&W Fuel Company, November 1989.

A.27

(Methodology used for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints)

15. BAW-10183P, Fuel Rod Gas Pressure Criterion, B&W Fuel Company, May 1994.

A.27

(Used for Specification 2.2.1, Reactor Trip System Instrumentation Setpoints)

The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC in accordance with 10 CFR 50.4.

A.20

INSERT 10

INSERT 11

A.21

A.30

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the NRC in accordance with 10 CFR 50.4 within the time period specified for each report.

87

6.10 RECORD RETENTION

6.10.1 In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

The following records shall be retained for at least 5 years:

- Records and logs of unit operation covering time interval at each power level;
- Records and logs of principal maintenance activities, inspections, repair, and replacement of principal items of equipment related to nuclear safety;
- ALL REPORTABLE EVENTS;
- Records of surveillance activities, inspections, and calibrations required by these Technical Specifications;
- Records of changes made to the procedures required by Specification 6.8.1;
- Records of radioactive shipments;
- Records of sealed source and fission detector leak tests and results; and

LA.19

ADMINISTRATIVE CONTROLSRECORD RETENTION (Continued)

87

- h. Records of annual physical inventory of all sealed source material of record.

88

6.10.2 The following records shall be retained for the duration of the unit Operating License:

LA.18

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report;
- b. Records of new and irradiated fuel inventory, fuel transfers, and assembly burnup histories;
- c. Records of radiation exposure for all individuals entering radiation control areas;
- d. Records of gaseous and liquid radioactive material released to the environs;
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1;
- f. Records of reactor tests and experiments;
- g. Records of training and qualification for current members of the unit staff;
- h. Records of inservice inspections performed pursuant to these Technical Specifications;
- i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- j. Records of meetings of the NSRB and reports required by Specification 6.5.1.11;
- k. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.8 including the date at which the service life commences and associated installation and maintenance records;
- l. Records of secondary water sampling and water quality;
- m. Records of analyses required by the Radiological Environmental Monitoring Program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed; and
- n. Records of reviews performed for changes made to Section 16.11 (Radiological Effluent Controls) of the FSAR.

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

6.10.3 Records of quality assurance activities required by the Operational Quality Assurance Manual shall be retained for a period of time as recommended by ANSI N.45.2.9-1974.

6.11 RADIATION PROTECTION PROGRAM

6.11 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

5.7.2 6.12 HIGH RADIATION AREA

requirements of
10CFR 20.1601

(S.7.1) 6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 30 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Radiation Protection Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- A radiation monitoring device which continuously indicates the radiation dose rate in the area; or
- A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them; or
- An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Radiation Protection Manager in the RWP.

(S.7.2) 6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 30 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Supervisor on duty and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work areas and the

or continuously
guarded

A.25

A.1

radiation
protection

A.32

Personnel

ADMINISTRATIVE CONTROLS

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM) (Continued)

A.14 ~~page numbered, dated, and containing the revision number, together with the appropriate analyses or evaluations justifying the change(s);~~

A.1

move to
ITS S.S.1

a.2.2) A determination that the change will not reduce the accuracy or reliability of dose calculations of effluent, dose, or Setpoint determinations; and

A.14

A.14

INSERT
12C

90

3) Documentation of the fact that the change has been reviewed and found acceptable by the Station Manager or the Radiation Protection Manager.

b. Shall become effective upon review and acceptance by a qualified individual/organization.

LA.6

Station
manager

A.1

6.15 MAJOR CHANGES TO LIQUID, GASEOUS, AND SOLID RADWASTE TREATMENT SYSTEMS*

6.15 Licensee-initiated major changes to the Radwaste Treatment Systems (liquid, gaseous, and solid):

a. Shall be reported to the Commission in the Annual Radioactive Effluent Release Report for the period in which the evaluation was reviewed by the Station Manager. The discussion of each change shall contain:

- 1) A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR 50.59;
- 2) Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information;
- 3) A detailed description of the equipment, components, and processes involved and the interfaces with other plant systems;
- 4) An evaluation of the change, which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the License application and amendments thereto;
- 5) An evaluation of the change, which shows the expected maximum exposures to a MEMBER OF THE PUBLIC in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the License application and amendments thereto;
- 6) A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period prior to when the changes are to be made;

LA.2D

*Licensees may choose to submit the information called for in this Specification as part of the annual FSAR update.

ADMINISTRATIVE CONTROLS

6.15 MAJOR CHANGES TO LIQUID, GASEOUS, AND SOLID RADWASTE TREATMENT SYSTEMS
(Continued)

- 7) An estimate of the exposure to plant operating personnel as a result of the change; and
- 8) Documentation of the fact that the change was reviewed and found acceptable by the Station Manager or the Chemistry Manager.
- b. Shall become effective upon review and acceptance by a qualified individual/organization.

6.16 SECTION 16.11 (RADIOLOGICAL EFFLUENT CONTROLS) OF THE FSAR

6.16.1 Section 16.11 (Radiological Effluent Controls) of the FSAR shall be approved by the Commission prior to implementation.

6.16.2 Licensee - initiated changes to ~~Section 16.11~~ ^{the} Radiological Effluent Controls of the FSAR.

- a. Shall be documented and records of reviews performed shall be retained ~~as required by Specification 6.16.2.b~~. This documentation shall contain:

1) Sufficient information to support the change ^(S) together with the appropriate analyses or evaluations justifying the change(s) and

2) A determination that the change ^(S) will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations or a determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.146, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

- b. Shall become effective ^{after approval of} ~~upon review and acceptance by~~ the Station Manager and a qualified individual/organization per the ~~Qualified~~ ^{QA.6} Reviewer Program.

- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire Section 16.11 of the ⁴FSAR as a part of or concurrent with the ^{Annual} Radioactive Effluent Release Report for the period of the report in which any changes to Section 16.11 of the FSAR was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date ^{(e.g.,} month/year) the change was implemented. ^{i.e.)}

Attachment 4

Duke Energy Corporation
Quality Assurance Program

Topical Report Duke-1

Amendment 23