



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

August 31, 2015

Mr. David A. Heacock  
President and Chief Nuclear Officer  
Virginia Electric and Power Company  
Innsbrook Technical Center  
5000 Dominion Blvd.  
Glenn Allen, VA 23060

SUBJECT: SURRY POWER STATION, UNIT 2 - REVIEW OF THE SPRING 2014 STEAM  
GENERATOR TUBE INSERVICE INSPECTIONS DURING REFUELING  
OUTAGE 25 (TAC NO. MF5189)

Dear Mr. Heacock:

By letters dated October 16, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14294A449) and April 20, 2015 (ADAMS Accession No. ML15126A042), Virginia Electric and Power Company (Dominion), the licensee, submitted information summarizing the results of the spring 2014 steam generator tube inspections performed at Surry Power Station, Unit 2. These inspections were performed during refueling outage 25.

The NRC staff has completed its review of the reports and concludes that the licensee provided the information required by their technical specifications and that no additional follow-up is required at this time. The NRC staff's review of the reports is enclosed.

If you have any questions, please contact me at (301) 415-1438 or via e-mail at [karen.cotton@nrc.gov](mailto:karen.cotton@nrc.gov).

Sincerely,

A handwritten signature in black ink, reading "Karen Cotton", is positioned above the typed name.

Karen Cotton, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-281

Enclosure:  
As stated

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REVIEW OF THE SPRING 2014  
STEAM GENERATOR TUBE INSERVICE INSPECTIONS  
SURRY POWER STATION, UNIT 2  
DOCKET NO. 50-281  
TAC NO. MF5189

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Surry Power Station, Unit 2, has three Westinghouse model 51F steam generators (SGs). Each SG has 3,342 thermally treated Alloy 600 tubes with an outside diameter of 0.875 inches and a nominal wall thickness of 0.050 inches. The tubes are hydraulically expanded at each end for the full depth of the tubesheet. The tubes are supported by stainless steel support plates with quatrefoil-shaped holes. The U-bend region of the tubes installed in rows 1-8 was thermally treated after bending in order to reduce stress.

The licensee provided the scope, extent, methods, and results of their SG tube inspections in the documents referenced above. In addition, the licensee described corrective actions (i.e., tube plugging) taken in response to the inspection findings.

Based on its review of the information submitted by the licensee, the NRC staff has the following observation:

- No cracking or pitting was detected during RFO 25.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by their technical specifications. In addition, the NRC staff concludes that there are no technical issues that warrant follow-up action at this time since the inspections appear to be consistent with the objective of detecting potential tube degradation and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.

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Sincerely,

/RA/

Karen Cotton, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

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