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PARKER, W.O.	Duke Power Co.		
RECIP. NAME	RECIPIENT AFFILIATION		
STOLZ, J.F.	NRC - No Detailed Affiliation Given		
DENTON, H.R.	Office of Nuclear Reactor Regulation, Director		

SUBJECT: Responds to NRC 810224 ltr. Only marginal benefit could be realized from further evaluation of L3-1 test. Util will provide info resulting from review of earlier pre-test predictions by 810601.

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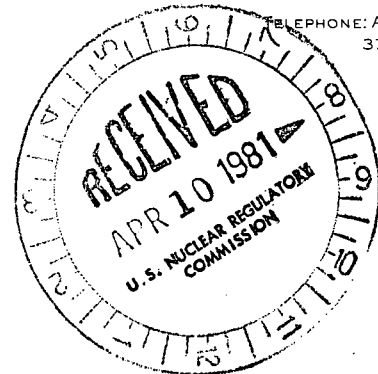
DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

April 3, 1981



Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Attention: Mr. J. F. Stolz

Reference: Letter from R. W. Reid to All Babcock & Wilcox Licensees,
February 24, 1981
Docket Nos. 50-269, -270, -287

Dear Sir:

Duke Power Company and Babcock & Wilcox have reviewed the referenced letter, evaluated the stated objectives, and provides the following response.

The B&W prediction of the L3-1 test compared very favorably with the other Vendor predictions. All of the predictions showed the clearing of liquid from the pump loop seal. However, the test did not produce this result due to a bypass flow path which existed between the vessel upper head and the downcomer annulus, as well as another bypass between the hot- and cold-leg pipes due to leakage path in LOFT is approximately 3 percent of the core flow, or comparable to prototype valves. However, the actual leakage path cannot be measured directly but only indirectly inferred by assuming a value which leads to the prediction agreement with the test. Therefore, the leakage flow from L3-1 must be further evaluated before additional analytical work could be justified. In addition, on page 40 of EGG-CAAP-5255 (LOFT L3-1 Preliminary Comparison Report) it is stated that B&W was the only Vendor who accurately calculated the behavior of the secondary side of the steam generator. For these reasons, Duke considers that only a marginal benefit at best could be realized from further detailed evaluation of the L3-1 test.

Regarding the S-07-1DD test, the situation is a little different. None of the Vendor predictions characterized the test very well. However, we feel this is due in large measure to insufficient information to model the steam side of the steam generator, as well as insufficient data on the valve and associated piping. There is also insufficient information to adequately model the steam separator. Based on these reasons, we are of the opinion that our current results are not unreasonable considering the conservative features of the model B&W used to predict the experiment.

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Mr. Harold R. Denton, Director

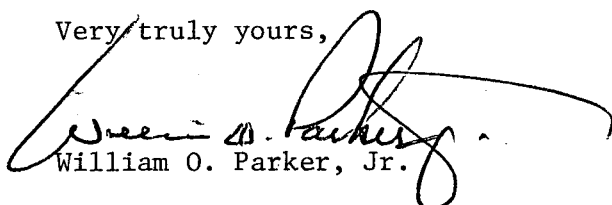
April 3, 1981

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Duke is considering a further review of the earlier pre-test predictions in light of the identified test uncertainties and model conservatisms to confirm the validity of the existing evaluation model.

Due to the extensive manpower commitment on the analysis of the L3-6 test, which was just completed and submitted to Dr. Sheron on March 23, as well as a continuing effort to respond to the requirements of NUREG-0737 and others, Duke will provide any pertinent information resulting from this review by June 1, 1981.

Very truly yours,

A handwritten signature in black ink, appearing to read "William O. Parker, Jr.", is written over the typed name. The signature is fluid and cursive, with a long horizontal stroke extending to the right.

William O. Parker, Jr.

RLG:pw