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ACCESSION NBR: 8003070361 DOC. DATE: 80/02/29 NOTARIZED: NO
 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co.
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co.
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 RECIP. NAME: DENTON, H.R. RECIPIENT AFFILIATION: Office of Nuclear Reactor Regulation

DOCKET #
 05000269
 05000270
 05000287

SUBJECT: Forwards supplemental info re NUREG-0578 items in response to NRC 800222 request. Direct valve position indication equipment inside containment has been seismically & environmentally qualified to withstand accident environ.

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DUKE POWER COMPANY

POWER BUILDING

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WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

February 29, 1980

TELEPHONE: AREA 704
373-4083

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. R. W. Reid, Chief
Operating Reactors Branch No. 4

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

On February 22, 1980, the NRC Staff requested supplemental information on several NUREG-0578 items. This letter provides supplemental information in response to the Staff request as well as supplemental information on other items.

Responses to certain items for which the Staff requested additional information are still in preparation and will be provided as soon as possible and in any event by March 7, 1980.

Very truly yours,

William O. Parker, Jr.

William O. Parker, Jr.

By [Signature]

RLG:scs

Attachment

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DUKE POWER COMPANY
OCONEE NUCLEAR STATION

Supplemental Response to NUREG-0578
February 29, 1980

2.1.1 No additional response.

2.1.2 No additional response.

2.1.3 My letter of November 21, 1979 stated that the components "will be" seismically and environmentally qualified. This terminology reflected the qualification of equipment that had not yet been received.

The Direct Valve Position Indication equipment located within the containment has been qualified to withstand the accident environment. It has also been seismically qualified.

Duke is continuing to evaluate various methods of determining RV levels. Presently there are no methods available that fully meet the requirements of this item. We will continue our efforts in this area. In the interim Duke considers that existing instrumentation and operator guidelines are adequate to detect inadequate core cooling.

2.1.4 My letter of January 31, 1980 provided a list of those systems which are isolated upon ES actuation on either high Reactor Building pressure (4 psig) or low reactor coolant pressure (1500 psig). This list is correct and complete.

2.1.5 Duke is in the process of locating valves and operators suitable for use as containment isolation valves inside the Reactor Building. The initial estimate of one year was based on placing an order and having a valve built to specifications. However, Duke is attempting to locate valves already manufactured and qualified and suitable for use. We expect to have more information in this regard by March 31, 1980.

2.1.6 The following system integrity leakage tests have been recently performed at Oconee:

<u>Unit 1</u>	<u>Results</u>
LPI	.23 gph
Coolant Storage	.16 gph
HPI	1.43 gph
LWD	ZERO
GW	3.5 scfm

<u>Unit 2</u>	
LPI (last performed 09/29/79)	.08 gph
All system integrity leakage tests will be performed prior to the completion of the upcoming refueling outage.	

2.1.6 Continued

Unit 3

Results

LPI (last performed 05/12/79)

.04 gph

All system integrity leakage tests will be performed prior to the completion of the upcoming forced outage to complete other NUREG-0578 items.

Additional supplemental information requested on this item is still in preparation and will be provided as soon as possible and in any event by March 7, 1980.

2.1.7 No additional response.

2.1.8 My letter of January 2, 1980 provided a detailed description of a noble gas vent monitor. Further review of the requirements of this item has prompted revision of our initial response.

This item requires a wide range noble gas monitor that covers the range of 10^{-7} $\mu\text{c/cc}$ to 10^{+5} $\mu\text{c/cc}$. The presently installed noble gas monitor will cover the range of 10^{-7} to 10^{+2} . To cover the range up to 10^{+5} $\mu\text{c/cc}$, a gross gamma detector will be added. This gross gamma detector will be attached to the outside of the unit vent and shielded to minimize count rate contribution from other possible sources. The gross gamma detector will be sensitive to the 80Kev energy range of noble gases. The gross gamma detector will have a minimum of one decade overlap with the noble gas monitor. If an event were to occur to cause the activity being released to be in the range of this gross gamma monitor, then the noble gas monitor sample will be isolated. This will prevent the noble gas monitor from becoming contaminated and rendering erroneous indication when activity starts decreasing.

Using a gross gamma detector has many significant advantages over a noble gas monitor to cover this extended range. Standard equipment can be directly applied. No new technology is needed which means shorter delivery time and lower costs. The system is passive in that no pumping systems, moisture remover systems or filtration is required. Also the gross gamma detector will not be exposed directly to the high temperatures that would be present during this postulated event. This will eliminate supporting equipment to cool the sample to a noble gas monitor.

It is considered that this proposed modification meets the intent of noble gas monitoring. We have delayed purchase of any equipment in this area until NRC approval of the design is received. Your prompt attention to this matter will facilitate completion of this effort.

Additional supplemental information requested on this item is still in preparation and will be provided as soon as possible and in any event by March 7, 1980.

2.1.9 My letter dated January 2, 1980 contained specific design information related to containment pressure, containment water level, and containment hydrogen indication systems.

Rather than meeting the explicit qualification requirements specified in that letter, (e.g. RG 1.97, RG 1.89) the qualification of the instrumentation which will be installed will meet or exceed the instrument qualification requirements on which the plant was licensed.