

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8211290681 DUC DATE: 82/11/19 NOTARIZED: NO DOCKET #  
 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co. 05000269  
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270  
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287  
 AUTH. NAME AUTHUR AFFILIATION  
 TUCKER, H.B. Duke Power Co.  
 RECIP. NAME RECIPIENT AFFILIATION  
 DENTON, H.R. Office of Nuclear Reactor Regulation, Director  
 STOLZ, J.F. Operating Reactors Branch 4

SUBJECT: Forwards table summarizing changes to revised inservice testing program. Changes will become effective when associated sys declared operational. Valves added that perform pressure isolation function will be leak tested.

DISTRIBUTION CODE: A047S COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5-----  
 TITLE: OR Submittal: Inservice Inspection/Testing

NOTES: AEOD/Ornstein:1cy. 05000269  
 AEOD/Ornstein:1cy. 05000270  
 AEOD/Ornstein:1cy. 05000287

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NRR/DE/MTEB 14	1 1	REG FILE	04 1 1
RGN2	1 1		
EXTERNAL: ACRS	16 10 10	LPDR	03 1 1
NRC PDR	02 1 1	NSIC	05 1 1
NTIS	1 1		

NOTES: 1 1

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November 19, 1982

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Mr. John F. Stolz, Chief  
Operating Reactors Branch No. 4

Subject: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287

Dear Sir:

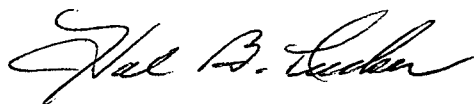
By letter dated March 25, 1982, the NRC Staff revised the Oconee Technical Specifications to incorporate the provisions of the Staff approved inservice testing program. The purpose of this letter is to advise the Staff of the revised inservice testing program for Oconee.

This revised program became effective July 1, 1982 and includes system changes that have been made subsequent to the last program submittal, including modifications made as a result of the TMI Action Plan and the installation of the Standby Shutdown Facility. The testing associated with these changes will become effective when the associated systems are declared operational. The applicable ASME Boiler and Pressure Vessel Code is the 1980 Edition.

Attached please find a table that summarizes the changes from the previous program. Those valves added that perform a pressure isolation function will be leak tested. All check valves added will be full stroke tested quarterly except as noted in the attached request for relief.

This information has been provided pursuant to an NRC request for information. As such, no license fees are deemed necessary.

Very truly yours,



Hal B. Tucker

RLG/php  
Attachment

8211290681 821119  
PDR ADOCK 05000269  
P PDR

A047

Mr. Harold R. Denton, Director  
November 19, 1982  
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cc: Mr. James P. O'Reilly, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
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Mr. Philip C. Wagner  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Mr. W. T. Orders  
NRC Resident Inspector  
Oconee Nuclear Station

DUKE POWER COMPANY  
OCONEE NUCLEAR STATION  
Changes to Inservice Testing Program  
July 1982

Applicable Units	Valve Number	LRT	Stroke	Check	Locked	System
1	CCW-109				0	Standby Shutdown Facility (SSF)
2	CCW-117				0	SSF
3	CCW-125				0	SSF
1	CCW-266		X			SSF
1 2 3	CCW-268		X			SSF
1 2 3	CCW-269		X			SSF
1	CCW-283		X			SSF
1	CCW-284			X		SSF
1	CCW-285		X			SSF
1	CCW-286		X			SSF
1 2 3	CCW-287		X			SSF
1	CCW-289		X			SSF
1 2 3	FDW-345			X		SSF
1 2 3	FDW-346			X		SSF
1 2 3	FDW-347		X			SSF
1 2 3	HP-398		X			SSF
1 2 3	HP-399			X		SSF
1 2 3	HP-400			X		SSF
1 2 3	HP-401			X		SSF
1 2 3	HP-402					SSF
1 2 3	HP-405	X	X			SSF
1 2 3	HP-409		X			HPI Cross-Connect
1 2 3	HP-410		X			HPI Cross-Connect
1 2 3	HP-417	X	X			SSF
1 2 3	HP-426	X	X			SSF
1 2 3	HPSW-184		X			LPSW to TDEFWP Oil Coolers
1 2 3	LPSW-138		X			RB Additional Cooling
1 2 3	LPSW-565		X			RB Additional Cooling
1 2 3	LPSW-566		X			RB Additional Cooling
1 2 3	PR-59	X	X			Dedicated H <sub>2</sub> Penetration
1 2 3	PR-60	X	X			Dedicated H <sub>2</sub> Penetration
1 2 3	PR-71		X			Hydrogen Sample System (RB) (HSS (RB))
1 2 3	PR-72		X			HSS (RB)
1 2 3	PR-73		X			HSS (RB)
1 2 3	PR-74		X			HSS (RB)
1 2 3	PR-75		X			HSS (RB)
1 2 3	PR-76		X			HSS (RB)
1 2 3	PR-77		X			HSS (RB)
1 2 3	PR-78		X			HSS (RB)
1 2 3	PR-79		X			HSS (RB)
1 2 3	PR-80		X			HSS (RB)
1 2 3	PR-81	X	X			HSS (RB)
1 2 3	PR-83		X			HSS (RB)
1 2 3	PR-84	X	X			HSS (RB)

Applicable Units	Valve Number	LRT	Stroke	Check	Locked	System
1 2 3	PR-86		X			Hydrogen Sample System (RB) (HSS (RB))
1 2 3	PR-87	X	X			HSS (RB)
1 2 3	PR-89		X			HSS (RB)
1 2 3	PR-90	X	X			HSS (RB)
1 2 3	PR-92		X			HSS (RB)
1 2 3	PR-93		X			Post-Accident Sample Panel
1 2 3	PR-94		X			Post-Accident Sample Panel
1 2 3	PR-95		X			Post-Accident Sample Panel
1 2 3	PR-96		X			Post-Accident Sample Panel
1 2 3	RC-155		X			RC High Point Vent System
1 2 3	RC-156		X			RC High Point Vent System
1 2 3	RC-157		X			RC High Point Vent System
1 2 3	RC-158		X			RC High Point Vent System
1 2 3	RC-159		X			RC High Point Vent System
1 2 3	RC-160		X			RC High Point Vent System
1 2 3	RC-162		X			Post-Accident Sample (RCS)
1 2 3	RC-163		X			Post-Accident Sample (RCS)
1 2 3	RC-164	X	X			Post-Accident Sample (RCS)
1 2 3	RC-165	X	X			Post-Accident Sample (RCS)
1 2 3	SF-82	X	X			SSF

Pumps:

Motor-Driven Emergency Feedwater (1A, 1B, 2A, 2B, 3A, 3B)

SSF RC Makeup

SSF Auxiliary Service Water

Spent Fuel Cooling (1C)

Reactor Coolant Bleed Transfer Pumps

### RELIEF REQUEST BASES

Testing of the FDW valves on the original emergency feedwater lines were approved by the NRC (SER Item 1.10.2.1). However, the line was modified following the TMI accident. The Oconee Valve Test Program now includes the new valves listed below:

Valve(s): 1FDW-311, 312, 317, 318, 373, 383  
2FDW-311, 312, 317, 318, 373, 383  
3FDW-311, 312, 317, 318, 373, 383

Category: C

Drawing Number/Coordinates: PO-121D-1/J-11, I-10, K-13, F-12, K-11, H-11  
PO-121D-2/K-10, I-11, K-13, F-12, K-11, H-11  
PO-121D-3/J-10, I-10, K-13, F-12, K-11, H-11

Function: These valves normally prevent backflow from the feedwater line to the emergency feedwater pump. In an emergency they open to allow flow from the emergency feedwater pump to the normal and emergency feedwater nozzles.

Test Requirement: IWV-3520 Check Valve Exercise Test

Bases for Relief: The emergency feedwater pump supplies unheated condensate to the steam generators. Therefore, exercising these valves at power would create undue thermal stresses on the steam generator tubes. In addition, the introduction of oxygen saturated water into the steam generators during a cold shutdown would delay startup.

Alternate Testing: These valves will be full-stroke exercised at refueling outages.