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50-270	Oconee Nuclear Station, Unit 2, Duke Power Co.	05000270
50-287	Oconee Nuclear Station, Unit 3, Duke Power Co.	05000287
50-369	William B. McGuire Nuclear Station, Unit 1, Duke Power Co.	05000369
50-370	William B. McGuire Nuclear Station, Unit 2, Duke Power Co.	05000370
50-413	Catawba Nuclear Station, Unit 1, Duke Power Co.	05000413
50-414	Catawba Nuclear Station, Unit 2, Duke Power Co.	05000414

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SUBJECT: Submits response to GL 96-01 re NRC testing of safety related logic circuits.

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DUKE POWER

April 17, 1996

U. S. Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555-0001

SUBJECT: Duke Power Company

Oconee Nuclear Station - Units 1, 2, and 3
Docket Nos. 50-269, 50-270, and 50-287

McGuire Nuclear Station - Units 1 and 2
Docket Nos. 50-369 and 50-370

Catawba Nuclear Station - Units 1 and 2
Docket Nos. 50-413 and 50-414

Response to Generic Letter 96-01

Generic Letter 96-01, dated January 10, 1996, requested holders of operating licenses to supply information to the NRC regarding testing of safety related logic circuits. The information was to include licensee intentions in regards to implementing the requested actions of GL 96-01. A detailed review of this generic letter was conducted and several Duke representatives also attended the March 19 industry/NRC GL 96-01 meeting facilitated by NEI in Atlanta. Based on these efforts, Duke Power will initiate a program that implements the requested actions of GL 96-01. This program will include a comprehensive review of each station's logic diagrams and surveillance test procedures to ensure that all portions of the logic circuitry are tested such that Technical Specifications requirements are fulfilled. The scope of systems to be included in the program will be the reactor protection system, the engineered safety features, the emergency power systems (Oconee) or the emergency diesel generator load shedding and sequencing systems (McGuire and Catawba), the emergency feedwater system (Oconee), and the main steam line break circuitry (Oconee) as described below.

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April 17, 1996

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DUKE POWER SCOPE OF SYSTEMS TO BE INCLUDED IN THE REQUESTED
ACTIONS OF GENERIC LETTER 96-01

Oconee Nuclear Station

Reactor Protection System - As Identified in FSAR Section
7.2, "Reactor Protective System"

Engineered Safety Features - As Identified in FSAR Sections
1.2.2.4, "Engineered Safeguards Systems" and 7.3,
"Engineered Safeguards Protective System"

Emergency Power Systems - As Identified in FSAR Sections
8.3.1.1.1, "Keowee Hydro Station" and 8.3.1.1.3, "4160 Volt
Auxiliary System"

Emergency Feedwater System - As Identified in FSAR Section
7.4.3, "Emergency Feedwater Controls"

Main Steam Line Break Circuitry - Scope to be identified in
the forthcoming Oconee FSAR update.

McGuire Nuclear Station

Reactor Protection System - As identified in FSAR Section
7.2, "Reactor Protection System"

Engineered Safety Features - As identified in FSAR Section
7.3, "Engineered Safety Features Actuation System"

Emergency Diesel Generator Load Shedding and Sequencing
Systems - As identified in FSAR Section 8.3.1.1.7, "Standby
Power Supplies"

Catawba Nuclear Station

Reactor Protection System - As Identified in FSAR Section
7.2, "Reactor Trip System"

Engineered Safety Features - As identified in FSAR Section
7.3, "Engineered Safety Features Actuation System"

Emergency Diesel Generator Load Shedding and Sequencing
Systems - As identified in FSAR Section 8.3.1.1.2, "Class 1E
AC Power Systems"

The schedule for conducting this program and providing a
confirmation report to the NRC is shown below for each of
the Duke nuclear units.

DUKE POWER SCHEDULE FOR COMPLETING THE REQUESTED ACTIONS OF
GENERIC LETTER 96-01:

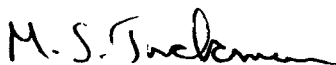
Station-Unit	Outage Number	Outage Begin	Outage End	NRC Response Date
MNS-1	EOC11	2/4/97	5/15/97	6/14/97
CNS-2	EOC8	3/18/97	5/2/97	6/1/97
ONS-1	EOC17	4/11/97	5/21/97	6/20/97
MNS-2	EOC11	8/13/97	11/21/97	12/21/97
ONS-2	EOC16	9/18/97	10/28/97	11/27/97
CNS-1	EOC10	11/23/97	1/7/98	2/6/98
ONS-3	EOC17	4/5/98	5/20/98	6/19/98

Note that the dates shown above are correlated to the units' upcoming outage schedules, consistent with the provisions of GL 96-01, and are therefore subject to change based upon operational considerations. The GL 96-01 review program for each unit is scheduled for completion in conjunction with the end of the respective outage for the applicable unit, as shown above.

Please direct questions on this matter to J. S. Warren at (704) 382-4986.

I declare, under penalty of perjury, that the statements set forth herein are true and correct to the best of my knowledge.

Very truly yours,



M. S. Tuckman

MST/JSW

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Duke Nuclear Sites