



Tennessee Valley Authority, Post Office Box 2000, Soddy Daisy, Tennessee 37384-2000

August 13, 2015

10 CFR 50.4
10 CFR 50.55a

ATTN: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Sequoyah Nuclear Plant, Unit 1
Facility Operating License No. DPR-77
NRC Docket No. 50-327

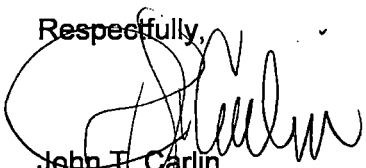
Subject: Unit 1 Refueling Outage 20 - 90-Day Inservice Inspection Summary Report

In accordance with the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, Article IWA-6230 and Code Case N-532-5, the Tennessee Valley Authority is providing the Sequoyah Nuclear Plant (SQN), Unit 1, Inservice Inspection (ISI) Summary Report within 90 days from completion of the Unit 1 Refueling Outage 20 (U1R20). The U1R20 refueling outage ended on May 16, 2015. Accordingly, this report is required to be submitted by August 14, 2015.

The Enclosure to this letter contains the Owner's Activity Report, a list of items with flaws or relevant conditions that required evaluation for continued service (Table 1), an abstract of repair and replacement activities required for continued service due to a flaw or relevant condition (Table 2), and the evaluation of acceptability of inaccessible areas of the steel containment vessel in accordance with 10 CFR 50.55a(b)(2)(ix)(A).

There are no regulatory commitments associated with this submittal. Should you have any questions, please contact Erin Henderson at (423) 843-7170.

Respectfully,


John T. Carlin
Site Vice President
Sequoyah Nuclear Plant

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U.S. Nuclear Regulatory Commission
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Enclosure: ASME Section XI Inservice Inspection Summary Report,
Unit 1 Refueling Outage 20

cc (Enclosure):

NRC Regional Administrator – Region II
NRC Senior Resident Inspector – Sequoyah Nuclear Plant
NRC Project Manager – Sequoyah Nuclear Plant

ENCLOSURE

**TENNESSEE VALLEY AUTHORITY
SEQUOYAH NUCLEAR PLANT, UNIT 1**

**ASME SECTION XI
INSERVICE INSPECTION SUMMARY REPORT
UNIT 1 REFUELING OUTAGE 20**

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number U1C20

Plant SEQUOYAH NUCLEAR PLANT, P.O. Box 2000, Soddy-Daisy, TN 37384-2000

Unit No. 1 Commercial service date July 1, 1981 Refueling outage no. 20
(if applicable)

Current inspection interval Third Inspection Interval
(1st, 2nd, 3rd, 4th, other)

Current inspection period Third Inspection Period
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plans 2001 Edition, 2003 Addenda

Date and revision of inspection plans March 27, 2015 - 0-SI-DXI-000-114.3, Revision 27

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans Not Applicable

Code Cases used for inspection and evaluation: N-460, N-513-3, N-532-4, N-532-5, N-566-2, N-586-1, N-686-1, N-706-1, N-716-1, N-722-1, N-729-1, N-770-1
(if applicable, including cases modified by Case N-532 and later revisions)

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code XI and (c) the repair/replacement activities and evaluations supporting the completion of U1C20 conform to the requirements of Section XI.
(refueling outage number)

Signed *James M. Hays* Date 8/11/15
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and and employed by HSB Global Standards of Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer make any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in manner for personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commission 13912 A.M.I
Inspector's Signature (National Board Number and Endorsement)

Date 8/11/2015

ASME Code Section XI Exception- Reference: TVA Condition Report 1024964, Subject: ASME Category B-G-1 (RCP hydrastuds)
 Description: Using the EPRI guidance, the performance demonstration completed for these exams was not performed as a "blind test" as required by ASME Section XI, Appendix VIII, Supplement 8, paragraph 2.0. This CR documents the issue and references the NRC issued EGM 14-003 as sufficient technical basis for accepting the examinations.

**ASME SECTION XI
INSERVICE INSPECTION SUMMARY REPORT
SEQUOYAH NUCLEAR PLANT UNIT 1 REFUELING OUTAGE 20**

TABLE 1

**ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT
REQUIRED EVALUATION FOR CONTINUED SERVICE**

Examination Category and Item Number	Item Description	Evaluation Description
F-A, F1.10D	Missing outer hex nut from support 1-RCH-114, Report R0039, NOI 1-SQ-516.	Engineering evaluation, accept as is. The support remains functional. This condition was previously identified in change document FCR #7927 and incorporated into calculation 1RCH0114, Attachment I.
F-A, F1.40	Improper clearance between pads and the seismic plates on support 1-CRDH-1, Report R0036, NOI 1-SQ-515	Engineering evaluation, accept as is. A zero gap is acceptable as long as the total gap along the array does not exceed the limits.
C-H, C7.10	Packing leak on 1-FCV-063-174, WO 116770561	Boric Acid Evaluation, No additional action is necessary. No visual evidence of corrosion.
C-H, C7.10	Packing leak on 1-FCV-063-0024, WO 116770560	Boric Acid Evaluation, Cleaned and observed, no active leak.

**ASME SECTION XI
INSERVICE INSPECTION SUMMARY REPORT
SEQUOYAH NUCLEAR PLANT UNIT 1 REFUELING OUTAGE 20**

TABLE 2

**ABSTRACT OF REPAIRS, REPLACEMENTS, OR CORRECTIVE MEASURES
REQUIRED FOR CONTINUED SERVICE**

Code Class	Item Description	Description of Work	Date Completed	Repair/Replacement Plan Number
No Applicable Repair/Replacement activities occurred during this period.				

**REPORTING REQUIRED BY 10 CFR 50.55a(b)(2)(ix)(A)
ASME Section XI, Subsection IWE
Steel Containment Vessel Inspection Program**

10 CFR 50.55a(b)(2)(ix)(A) requires reporting of the degradation assessment for inaccessible areas when conditions are identified in accessible areas during the performance of the ASME Section XI, Subsection IWE Steel Containment Vessel (SCV) Inspection Program that could indicate the presence of or result in degradation to such inaccessible areas.

The conditions identified during the SQN Unit 1 Cycle 20 refueling outage on the SCV in an accessible area were determined to be acceptable and do not affect the degradation assessment for inaccessible areas.