

1. Energy Inc, "Palisades Plant PCS Overpressurization Subsystem Description" Report, October 1977
2. CEN-5, "Palisades Reactor Internal Wear Report," April 1974
3. WCAP-15353, Revision 0, "Palisades Reactor Pressure Vessel Neutron Fluence Evaluation," Westinghouse Electric Company, LLC, Roberts, G.K., et al., January 2000
4. Deleted |
5. Hood (NRC) to Haskell (Consumers), "Palisades Plant - Reactor Vessel Neutron Fluence Evaluation and Revised Schedule for Reaching Pressurized Thermal Shock Screening Criteria (TAC No. MA8250)," November 14, 2000
6. WASH-1400 (NUREG-75/014), Appendix V, October 1975
7. Combustion Engineering Report, "Input for Response to NRC Lessons Learned Requirements for Combustion Engineering Nuclear Steam Supply Systems," CEN-125, December 1979
8. CE-NPSD-154, "Natural Circulation Cooldown," October 1981
9. Technical Paper - WAPD-BT18 Bettis Technical Review, Reactor Technology Section, "Application of Stress Concentration Factors" by B F Langer, April 1960
10. US Nuclear Regulatory Commission, Regulatory Guide 1.99, "Effects of Residual Elements on Predicted Radiation Damage to Reactor Vessel Materials," Revision 2, May 1988
11. US NRC Standard Review Plan, Directorate of Licensing, Section 5.3.2, "Pressure-Temperature Limits"
12. ASME B&PV Code, Section III, Appendix G, "Protection Against Non-Ductile Failure," 1974 Edition
13. C-PAL-98-1924 Code Analytical Report Fatigue Analysis and Specification for the RV Missing 500 Reactor Trips Listed in FSAR Section 4.2.2
14. 10 CFR 50, Appendix G, "Fracture Toughness Requirements," May 31, 1983 as Amended November 6, 1986
15. Battelle Report, "Palisades Nuclear Plant Reactor Vessel Surveillance Program: Capsule A-240," March 13, 1979

16. Deleted
17. Consumers Power Company Letter, From Brian D Johnson to Harold R Denton, US Nuclear Regulatory Commission, Dated October 31, 1984
18. Kunka, M K and Cheney, C A, "Analysis of Capsules T-330 and W-290 From the Consumers Power Company Palisades Reactor Vessel Radiation - Surveillance Program," WCAP-10637, September 1984
19. Peter, Lippincott, Wrights and Madeyski (Westinghouse), "Analysis of Capsule W-110 from the Consumers Power Company Palisades Reactor Vessel Surveillance Program," May 1994
20. Deleted
21. SOER 82-7, "Reactor Vessel Pressurized Thermal Shock"
22. CEN-152, "Combustion Engineering Emergency Procedure Guideline," Dated May 8, 1984
23. Palisades Technical Specifications Basis B3.4.3
24. Palisades Systems Operation Procedure SOP-1, "Primary Coolant System"
25. Deleted
26. Deleted
27. CPCo to NRC dated November 22, 1993, "Palisades Plant - Bulletin 88-11: Pressurizer Surge Line Thermal Stratification - Additional Information"
28. CPCo to NRC dated April 30, 1994, "Palisades Plant - Bulletin 88-11: Pressurizer Surge Line Thermal Stratification - Additional Information"
29. CEN-387-P (Also Numbered CE NPSD-546-P), "Pressurizer Surge Line Flow Stratification Evaluation," July 1988
30. Engineering Analysis EA-FC-809-13
31. Engineering Analysis EA-PTS-87010, DOT Benchmarking Model
32. Holian, Brian, NRC Project Manager, to Gerald B. Slade, Palisades Plant General Manager, "Amendment No. 135 to Provisional Operating License No. DPR-20," February 11, 1991
33. CE Owner's Group Asymmetric Loads Program Report, "Reactor Coolant System Asymmetric Loads Evaluation Program Final Report," Volumes 1, 2 and 3, dated June 30, 1980

34. Combustion Engineering Report, "Response to Questions on the Reactor Coolant System Asymmetric Loads Evaluation Program Final Report," Submitted to the NRC on July 31, 1981
35. Combustion Engineering Owner's Group, "Leak-Before-Break Evaluation of Primary Coolant Loop Piping in Combustion Engineering Designed Nuclear Steam Supply Systems," CEN-367, November 1987
36. DeAgazio, Albert, USNRC, "Safety Evaluation on Asymmetric LOCA Loads - MPA D-010 - Palisades Plant (Tac No MO8621)" to K W Berry, October 27, 1989
37. Deleted
38. Deleted
39. Hsia (NRC) to Slade (CPCo), September 13, 1993, "Safety Evaluation for Combustion Engineering Owners Group Report CEN-387-P, Revision 1, 'Pressurizer Surge Line Flow Stratification' (Bulletin 88-11) (TAC No. M72151)"
40. ESS Specification SP-MP-8304-002(Q), Revision 1 of Conformed Palisades Specification M1-LBA, Data Sheet 1 for PRV-1042B
41. C.E. Book No. 70277, Instruction Manual-Steam Generators- Palisades Plant Consumers Power Company, Combustion Engineering Inc., Nuclear Components Department, Chattanooga, Tennessee, January 1982
42. Combustion Engineering Calculation 82688-STS-602, "Steam Generator Inventory", Contract 82688-Palisades Replacement Steam Generator, J.C.Lowry, December 20, 1989
43. Combustion Engineering Report, "Specification for Steam Generator Assemblies for Consumers Power Company", Specification No. 19377-PE-120 dated June 24, 1982
44. Combustion Engineering Calculation Number PSEC-36, "Sizing of Pressurizer Spray Nozzle and Piping," dated 2/14/67
45. Hannon (NRC) to Bordine (Consumers), December 20, 1996, "Palisades: Evaluation of Updated Reactor Pressure Vessel Fluence Values (TAC No. M95134)"
46. Specification Change 95-038, Revision of LTOP Program
47. Amendment No. 163 to Facility Operating License No. DPR-20 for the Palisades Plant. SUBJECT: PALISADES PLANT - ISSUANCE OF AMENDMENT RE: PRESSURE-TEMPERATURE LIMITS (TAC NO. M90650)

48. Combustion Engineering, Engineering Specification for a Pressurizer Assembly, Specification No 70P-001, Revision 3 (F679/1631)
49. NRC, SER "Approval of ASME Code Case N-474-2 to Use Forged Alloy 690 (SB-564) Material," May 4, 1995
50. Hood (NRC) to Cooper (NMC), October 11, 2001, "Palisades Plant – Revised Withdrawal Schedule for Reactor for Vessel Surveillance Capsule T-150 (TAC No. MB2862)"
51. BWXT Report, "Analysis of Capsule W-100 from the Nuclear Management Company Palisades Reactor Vessel Material Surveillance Program," February 2004
52. NRC Order EA-03-009, "Issuance of First Revised NRC Order (EA-03-009) Establishing Interim Inspection Requirements for Reactor Pressure Vessel Heads at Pressurized Water Reactors"
53. EA-RCH-01-05, "Calculation of Chapter 14 Safety Analysis Parameter Changes Due to FC-977 Power Uprate"
54. Appendix G to the 1998 through the 2000 Addenda Edition of the ASME Boiler and Pressure Vessel Code, Section XI, Division 1, "Fracture Toughness Criteria for Protection Against Failure"
55. Engineering Analysis EA-EC27959-01, "Palisades Pressure-Temperature Limit Curves and Upper-Shelf Energy Evaluation," February 2012.
56. WCAP-15353 Supplement 1-NP, Revision 0, "Palisades Reactor Pressure Vessel Fluence Evaluation," Westinghouse Electric Company, LLC, May 2010 (filed in Palisades Engineering Report No. PLP-RPT-10-00058).
57. Engineering Analysis EA-EC26115-01, Revision 0, "Pressurized Thermal Shock PTS Evaluation for the Palisades Reactor Pressure Vessel," February 2012.
58. Letter from MChawla (NRC) to Vice President Operations Entergy-Palisades, "Updated Reactor Pressure Vessel Pressurized Thermal Shock Evaluation for Palisades Nuclear Plant (TAC No. ME5263)," December 7, 2011 (contains the Safety Evaluation for the updated PTS evaluation).
59. Letter from MChawla (NRC) to Vice President, Entergy Nuclear Operations, "Palisades Plant - Evaluation of Relief Request to Extend the Third 10-Year Inservice Inspection Interval for Reactor Vessel Weld Examination (TAC No. MD9265)," February 11, 2009.