UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 26, 2015

Mr. Mano Nazar
President and Chief Nuclear Officer
Nuclear Division
NextEra Energy
P. O. Box 14000
Juno Beach, FL 33408-0420

SUBJECT: TURKEY POINT NUCLEAR GENERATING UNIT NOS. 3 AND 4 - CORRECTIONS TO AMENDMENTS NOS. 263 AND 258 (TAC NOS. MF6560 AND MF6561)

Dear Mr. Nazar:

The U.S. Nuclear Regulatory Commission (NRC) staff identified typographical errors in the Technical Specifications (TSs) for Turkey Point Nuclear Generating Unit Nos. 3 and 4 (Turkey Point). Two errors were inadvertently introduced in Amendment Nos. 263 and 258 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15166A320), dated July 16, 2015, for Units 3 and 4, respectively. The NRC is issuing this letter to correct the errors in accordance with the guidance in the January 16, 1997, NRC memorandum (ADAMS Accession No. ML103260096), which is derived from the December 17, 1996, Staff Requirements Memorandum (ADAMS Accession No. ML003754054) for SECY-96-238 (ADAMS Legacy Library Accession No. 9611250030).

By license amendment application dated April 9, 2014 (ADAMS Accession No. ML14105A042), as supplemented by letters dated August 29, 2014, and February 20, April 3, and July 7, 2015 (ADAMS Accession Nos. ML14252A228, ML15069A153, ML15113A311, and ML15194A120, respectively), Florida Power & Light Company (the licensee) requested to incorporate a Surveillance Frequency Control Program to numerous TS pages. The NRC approved the licensee's request with issuance of Amendment Nos. 263 and 258. The NRC staff made typographical errors on TS pages 3/4 5-7 and 3/4 8-15. In the letter dated April 3, 2015, the licensee proposed to retain "during shutdown" in item f of TS page 3/4 5-7. The NRC staff inadvertently omitted "during shutdown" in item f of TS page 3/4 5-7 when Amendment Nos. 263 and 258 were issued. In the letter dated April 9, 2014, the licensee proposed deletion of the revision bar on item 3) on TS page 3/4 8-15. The NRC staff inadvertently left the revision bar on item 3) on TS page 3/4 8-15 when Amendment Nos. 263 and 258 were issued.

Enclosed are the corrected TS pages 3/4 5-7, and 3/4 8-15 for Amendment Nos. 263 and 258.

The NRC staff concludes that the errors were introduced during the preparation of the license amendments and are entirely typographical in nature. The corrections do not change any of the conclusions in the safety evaluation associated with the amendments and do not affect the associated notice to the public.

Please contact me at (301) 415-0489 if you have any questions on this issue.

Sincerely,

Farideh E. Saba

for Audrey L. Klett, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosure:
Corrections to Technical Specifications

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CORRECTIONS TO TECHNICAL SPECIFICATIONS

FOR TURKEY POINT NUCLEAR GENERATING UNIT NOS. 3 AND 4

RENEWED FACILITY OPERATING LICENSE NOS. DPR-31 AND DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise the Appendix A Technical Specifications as follows.

Remove pages	Insert pages
3/4 5-7	3/4 5-7
3/4 8-15	3/4 8-15

Enclosure

d. By a visual inspection which verifies that no loose debris (rags, trash, clothing, etc.) is present in the containment which could be transported to the containment sump and cause restriction of the pump suctions during LOCA conditions. The visual inspection shall be performed:

1) For all accessible areas of the containment prior to establishing CONTAINMENT INTEGRITY, and

2) At least once daily of the areas affected within containment by containment entry and during the final entry when CONTAINMENT INTEGRITY is established.

e. In accordance with the Surveillance Frequency Control Program by:

1) Verifying automatic isolation and interlock action of the RHR system from the Reactor Coolant System by ensuring that with a simulated or actual Reactor Coolant System pressure signal greater than or equal to 525 psig the interlocks cause the valves to automatically close and prevent the valves from being opened, and

2) Verifying correct interlock action to ensure that the RWST is isolated from the RHR System during RHR System operation and to ensure that the RHR System cannot be pressurized from the Reactor Coolant System unless the above RWST Isolation Valves are closed.

3) A visual inspection of the containment sump and verifying that the suction inlets are not restricted by debris and that the sump components (trash racks, screens, etc.) show no evidence of structural distress or abnormal corrosion.

f. In accordance with the Surveillance Frequency Control Program, during shutdown, by:

1) Verifying that each automatic valve in the flow path actuates to its correct position on Safety Injection actuation test signal, and

2) Verifying that each of the following pumps start automatically upon receipt of a Safety Injection actuation test signal:

a) Safety Injection pump, and

b) RHR pump.

SURVEILLANCE REQUIREMENTS (Continued)

2) The cell-to-cell and terminal connections are clean, tight, and coated with anticorrosion material,

3) Each 400 amp battery charger (associated with Battery Banks 3A and 4B) will supply at least 400 amperes at ≥ 129 volts for at least 8 hours, and each 300 amp battery charger (associated with Battery Banks 3B and 4A) will supply at least 300 amperes at ≥ 129 volts for at least 8 hours, and

4) Battery Connection resistance is:

Battery	Connection	Limit (Micro-Ohms)
3B, 4A	inter-cell / termination	≤ 29
	inter-cell (brace locations)	≤ 30
	transition cables	≤ 125
	or	
	total battery connections	≤ 1958
Battery	Connection	Limit (Micro-Ohms)
3A, 4B, D-52	inter-cell / termination	≤ 35
	inter-cell (brace locations)	≤ 40
	transition cables	≤ 125
	or	
	total battery connections	≤ 2463

d. In accordance with the Surveillance Frequency Control Program, during shutdown**, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual or simulated emergency loads for the design duty cycle when the battery is subjected to a battery service test.

e. At least once per 12 months, during shutdown**, by giving performance discharge tests of battery capacity to any battery that shows signs of degradation or has reached 85% [75% for Batteries 4B and D52 (Spare) when used in place of Battery 4B] of service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% [7% for Batteries 4B and D52 (Spare) when used in place of Battery 4B] of rated capacity from its average on previous performance tests, or is below 90% [93% for Batteries 4B and D52 (Spare) when used in place of Battery 4B] of the manufacturer's rating.

f. In accordance with the Surveillance Frequency Control Program, during shutdown**, by verifying that the battery capacity is at least 80% [87% for Batteries 4B and D52 (Spare) when used in place of Battery 4B] of the manufacturer's rating when subjected to a performance discharge test. Once per 60-month interval this performance discharge test may be performed in lieu of the battery service test required by Specification 4.8.2.1.d.

**Except that the spare battery bank D-52, and any other battery out of service when spare battery bank D-52 is in service may be tested with simulated loads during operation.

Please contact me at (301) 415-0489 if you have any questions on this issue.

Sincerely,

/RA by FSaba for

Audrey L. Klett, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosure:
Corrections to Technical Specification Pages 3/4 5-7 and 3/4 8-15

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ADAMS Accession No.: ML15225A305

OFFICE	LPLII-2/PM	LPLII-2/LA	OGC - NLO	LPLII-2/BC	LPLII-2/PM
NAME	AKlett	BClayton	AGhosh	SHelton	AKlett (FSaba for)
DATE	8/17/15	8/17/15	8/24/15	8/26/15	8/26/15

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