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 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co. 05000269  
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270  
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287  
 50-369 William B. McGuire Nuclear Station, Unit 1, Duke Power Co. 05000369  
 50-370 William B. McGuire Nuclear Station, Unit 2, Duke Power Co. 05000370  
 50-413 Catawba Nuclear Station, Unit 1, Duke Power Co. 05000413  
 50-414 Catawba Nuclear Station, Unit 2, Duke Power Co. 05000414

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 RECIP. NAME Document Control Branch (Document Control Desk)

SUBJECT: Forwards listing & projected schedule for completion & submittal of new topical repts requiring NRC approval by util. New Topical Rept DPC-NE-3002 will describe methodology to address impact of replacement SGs.

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**DUKE POWER**

April 26, 1993

U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Document Control Desk

Subject: Submittal Schedule for Duke Power Company Topical Reports  
Oconee Nuclear Station; McGuire Nuclear Station; Catawba Nuclear Station  
Docket Nos: 50-269, -270, -287  
50-369, -370  
50-413, -414

Attached is a listing and projected schedule for the completion and submittal of new topical reports requiring NRC approval by Duke Power Company. This listing is provided to assist the NRC in planning and scheduling future resources. The anticipated submittal dates and NRC approval need dates are indicated.

Duke Power has recently selected a replacement steam generator design for McGuire Units 1 and 2 and Catawba Unit 1. The new steam generators will be of the feedring design, whereas the current steam generators are of the preheater design. Consequently, the Duke Power approved safety analysis methods topical reports may need to be revised to reflect revised modeling of the steam generators with the RETRAN code. It is expected that there will be a relatively minor impact on most of the FSAR Chapter 15 transients. Revisions to DPC-NE-3000 (RETRAN models), DPC-NE-3001 (rod ejection, steam line break, dropped rod, and key physics parameters), and DPC-NE-3002 (Chapter 15 methods and assumptions) will be revised accordingly. Also, a new topical report DPC-NE-3004 (mass and energy release and containment response) will describe methodology to address the impact of the replacement steam generators. This work will be completed and submitted by April 1, 1994 and will require approval to support the McGuire Unit 1 steam generator replacement outage currently scheduled for August 1, 1995.

If you have any questions concerning this information please contact K. S. Canady at 704-382-4712.

H. B. Tucker  
H. B. Tucker

KSC/pkh

Attachment

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cc: R. C. Jones, Branch Chief  
V. Nerses, Project Manager  
R. E. Martin, Project Manager  
L. A. Wiens, Project Manager

<u>Number</u>	<u>Title</u>	<u>Submittal Date</u>	<u>NRC Approval Need Date</u>	<u>Comments</u>
DPC-NE-3000	Oconee Nuclear Station - Thermal-Hydraulic Transient Analysis Methodology	9/87	1/94	RETRAN and VIPRE transient analysis methods. McGuire and Catawba scope approved by NRC 11/91. Oconee scope under NRC review since April 1989. Last Duke response to NRC questions was March 11, 1992.
DPC-NE-3000 (Revision 1)	McGuire and Catawba Nuclear Stations - Thermal-Hydraulic Transient Analysis Methodology	4/1/94	8/1/95	Revise RETRAN models for replacement steam generators.
DPC-NE-3001 (Revision 1)	McGuire and Catawba - Multidimensional Reactor Transients Methodology	4/1/94	8/1/95	Revise steam line break and dropped rod analysis for replacement steam generators.
DPC-NE-3002 (Revision 1)	McGuire and Catawba - FSAR Chapter 15 System Transients Analysis Methodology	4/1/94	8/1/95	Revise FSAR methods for replacement steam generators.
DPC-NE-3003	Oconee Nuclear Station - Mass and Energy Release and Containment Response Methodology	7/1/93	1/95	Modernize containment pressures and temperature calculation; incorporate recent lake temperature increases and low pressure injection/reactor building cooler fouling phenomena.
DPC-NE-3004	McGuire/Catawba Nuclear Stations - Mass and Energy Release and Containment Response Methodology	4/1/94	8/1/95	Support MNS 1, cycle 11 startup (Dec. 1995) after steam generator replacement. New steam generators will impact existing analyses.
DPC-NE-3005	Oconee Nuclear Station - FSAR Chapter 15 System Transient Analysis Methodology	6/95	1/97	Modernize ONS non-LOCA Chapter 15 analysis. Determine the thermal margin for reload design.
DPC-NE-2005	Duke Power Company Thermal/Hydraulic Statistical Core Design Methodology	10/1/92	4/94	Provide SCD methodology for Oconee/McGuire/Catawba and other PWR plants using mixing vane fuel. Provide additional thermal margin for reload design and transition to SCD for ONS. (McGuire 1, cycle 11 startup; Oconee 3, cycle 16 startup.)

DPC-NE-2006	DCHF-3 Correlation for Predicting Critical Heat Flux in Mixing Vane Grid Fuel Assemblies	6/1/94	12/95	Provide additional thermal margin for reload design. (McGuire 2, cycle 11 startup). Will also submit addendum to DPC-NE-2005 to incorporate DCHF-3 correlation.
DPC-NE-2007	Oconee, McGuire, and Catawba Nuclear Stations Fuel Reconstitution Analysis Methodology	12/1/93	6/95	Provide justification for replacement cold rods and vacancies for fuel reconstitution.

### Duke Power Topical Reports

<u>NRC Budget Year</u>	<u>Duke Priority</u>	<u>Topical Report</u>	<u>Comments</u>
1993	1	DPC-NE-2005	New topical
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1994	1	DPC-NE-2007	New topical
1994	2	DPC-NE-3000	Review in progress, small scope remaining
1994	2	DPC-NE-3003	New topical
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1995	2	DPC-NE-2006	New topical
1995	1	DPC-NE-3000, Rev. 1	Small scope
1995	1	DPC-NE-3001, Rev. 1	Small scope
1995	1	DPC-NE-3002, Rev. 1	Small scope
1995	1	DPC-NE-3004	New topical
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1996	None		
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1997	1	DPC-NE-3005	New topical

Priority 1 - Essential need

Priority 2 - Intermediate need