

## 6.6.2 Non-Routine Reports

### 6.6.2.1 Reportable Occurrences

#### a. Prompt Notification with Written Followup

The types of events listed below shall be reported within 24 hours of discovery by telephone, and confirmed by telegraph, mailgram or facsimile no later than the first normal working day following the event to the Director, Office of Inspection and Enforcement, Region II, or his designate with a written followup report within two weeks to the Director, Office of Inspection and Enforcement, Region II (copy to the Director, Office of Management Information and Program Control).

- (1) Failure of the Reactor Protective System to trip, as required, when a monitored parameter reaches the setpoint specified as the limiting safety system setting in the Technical Specifications.
- (2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications.
- (3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary or primary containment.
- (4) Reactivity anomalies involving disagreement with predicted value of reactivity balance under steady-state conditions greater than or equal to 1%  $\Delta k/k$ ; a calculated reactivity balance indicating shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds, or if subcritical, an unplanned reactivity insertion of more than 0.5%  $\Delta k/k$ ; or any unplanned criticality.
- (5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the Safety Analysis Report.
- (6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the Safety Analysis Report.
- (7) Conditions arising from natural or man-made events that, as a direct result of the event, require unit shutdown, operation of safety systems, or other protective measures required by Technical Specifications.
- (8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the Safety Analysis Report or in the bases for the Technical Specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

- (9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during unit life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

b. Thirty-Day Written Reports

The types of events listed below shall be the subject of written reports to the Director, Office of Inspection and Enforcement, Region II, within 30 days of discovery of the event. (Copy to the Director, Office of Management Information and Program Control)

- (1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- (2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or shutdown required by a limiting condition for operation.
- (3) Observed inadequacies in the implementation of administrative or procedural controls during operation of a unit which could cause reduction of degree of redundancy provided in the Reactor Protective System or Engineered Safety Feature Systems.

6.6.2.2 Environmental Monitoring

- a. If individual milk samples show I-131 concentrations of 10 picocuries per liter or greater, a plan shall be submitted within one week advising the NRC of the proposed action to ensure the plant related annual doses will be within the design objective of 15 mrem/yr to the thyroid of any individual.
- b. If milk samples collected over a calendar quarter show average concentrations of 4.8 picocuries per liter or greater, a plan shall be submitted within 30 days advising the NRC of the proposed action to ensure the plant related annual doses will be within the design objective of 15 mrem/yr to the thyroid of any individual.
- c. If, during any annual report period, a measured level of radioactivity in any environmental medium other than those associated with gaseous radioiodine releases or liquid effluent releases exceeds ten times the control station value, a written notification will be submitted within one week advising the NRC of this condition. This notification should include any evaluation of any release conditions, environmental factors, or other aspects necessary to explain the anomalous result.

- d. If, during any annual report period, a measured level of radioactivity in any environmental medium associated with liquid effluent releases exceeds fifty times the control station value for sampling points at or upstream of location 000.7 or ten times the control station value for sampling points downstream of location 000.7, a written notification will be submitted within one week advising the NRC of this condition. This notification should include an evaluation of any release conditions, environmental factors, or other aspects necessary to explain the anomalous result.

#### 6.6.3 Special Reports

Special reports shall be submitted to the Director, Office of Inspection and Enforcement, Region II, within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Electrical System Degradation, Specification 3.7.
- b. Excessive Liquid Waste Releases, Specification 3.9.
- c. Excessive Gaseous Waste Releases, Specification 3.10.
- d. Inservice Inspection, Specification 4.2.4.
- e. Reactor Vessel Specimen Surveillance, Specification 4.2.8.
- f. Containment Integrated Leak Rate Test, Specification 4.4.1.1.7.
- g. Reactor Building Annual Inspection Report, Specification 4.4.1.4.
- h. Tendon Stress Surveillance, Specification 4.4.2.2.
- i. End Anchorage Concrete Surveillance, Specification 4.4.2.3.
- j. Liner Plate Surveillance, Specification 4.4.2.4.
- k. Single Loop Operation, Specification 3.1.8.
- l. Fuel Surveillance Program, Specification 4.13.