WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47 and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 28th day of March, 1980.

Notary Public

My Commission Expires:

February 15, 1982

PROPOSED AMENDMENT
TO
OCONEE NUCLEAR STATION
TECHNICAL SPECIFICATIONS

Table 4.1-3
Section 6.6.1.4(2)(a)
Table 6.6-1

March 1980

TABLE 4.1-3 Cont.

MINIMUM SAMPLING FREQUENCY

Item	Check	Frequency	Sensitivity Limits of Lab Analysis for Waste
7. Low Activity Waste Tank, Condensate Test Tank, Condensate Monitoring Tank, Laundry-Hot Shower Tank	a. Principal Gamma Emitters(5) including Dissolved Noble Gases	a. Prior to release of each batch	a. Gamma Nuclides $<5 \times 10^{-7}$ µCi/ml Dissolved Gases $<10^{-5}$ µCi/ml
	b. Radiochemical Analysis Sr 89,90	b. Monthly	b. $<10^{-8}$ µCi/ml
	c. Tritium	c. Monthly	c. $<10^{-5}$ µCi/ml
	d. Gross Alpha Activity	d. Monthly	d. $<10^{-7}$ µCi/ml
8. Waste Gas Decay Tank	a. Principal Gamma Emitters(6)	a. Prior to release of each batch	a. $<10^{-4}$ µCi/cc (gases) $<10^{-10}$ µCi/cc (particulates and iodines)
	b. Tritium	b. Prior to release of each batch	b. $<10^{-6}$ µCi/cc
9. Unit Vent Sampling	a. Iodine Spectrum(4)	a. Weekly	a. $<10^{-10}$ µCi/cc
	b. Particulates (4)		
	1) Principal Gamma Emitters(6)	1) Weekly Composite	1) $<10^{-10}$ µCi/cc
	2) Gross Alpha Activity	2) Quarterly on a sample of one week duration	2) $<10^{-11}$ µCi/cc
	3) Radiochemical Analysis Sr 89,90	3) Quarterly Composite	3) $<10^{-11}$ µCi/cc

TABLE 4.1-3 Cont.

MINIMUM SAMPLING FREQUENCY

Item	Check	Frequency	Sensitivity Limits of Lab Analysis for Waste
	c. Gases by Principal Gamma Emitters(6)	c. Weekly	c. $<10^{-4}$ µCi/cc
10. Keowee Hydro Dam Dilution Flow	Measure Leakage Flow Rate	Annually	
11. Condenser Air Ejector Partition Factor	Measure Iodine Partition Factor in Condenser	One time if and when primary to secondary leaks develop	
12. Reactor Building	a. Principal Gamma Emitters(6)	a. Each Purge	a. $<10^{-4}$ µCi/cc (gases) $<10^{-10}$ µCi/cc (particulates and iodines)
	b. Tritium	b. Each Purge	b. $<10^{-6}$ µCi/cc

(1) When radioactivity level is greater than 10 percent of the limits of Specification 3.1.4, the sampling frequency shall be increased to a minimum of once each day.

(2) $\bar{E}$ determination will be started when gross beta-gamma activity analysis indicates greater than 10 µCi/ml and will be redetermined for each 10 µCi/ml increase in gross beta-gamma activity analysis thereafter. A radiochemical analysis for this purpose shall consist of a quantitative measurement of 95 percent of the radionuclides in the reactor coolant with half lives greater than 30 minutes. This is expected to consist of gamma isotopic analysis of the primary coolant, including dissolved gaseous activities, radiochemical analysis for Sr-89 and Sr-90, and tritium analysis.

MINIMUM SAMPLING FREQUENCY

(3) When gross activity increases by a factor of two above background, an iodine analysis will be made and performed thereafter when the gross beta-gamma activity increases by 10 percent.

(4) When the activity level exceeds 10 percent of the limits of Specification 3.9, the sampling frequency shall be increased to a minimum of once each day. This can be done by RIA-44 (Unit Vent Iodine Monitor). When the gross activity release rate exceeds one percent of the maximum release rate and the average gross activity release rate increased by 50 percent over the previous day, an analysis shall be performed for iodines and particulates. This can be done by RIA-44 (Unit Vent Iodine Monitor) and RIA-43 (Unit Vent Particulate Monitor).

(5) For certain radionuclides with low gamma yield or low energies, or for certain radionuclides mixtures, it may not be possible to measure radionuclides in concentrations near the lower limit of detection (LLD). Under these circumstances, the LLD may be increased inversely proportionately to the magnitude of the gamma yield (i.e., $5 \times 10^{-7}/I$, where I is the photon abundance expressed as a decimal fraction), but in no case shall the LLD, as calculated in this manner for a specific radionuclide, be greater than 10% of the MPC value specified in 10 CFR 20, Appendix B, Table II, Column 2.

(6) The principal gamma emitters for which the LLD specification will apply are exclusively the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141, and Ce-144. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD for the analyses should not be reported as being present at the LLD level. When unusual circumstances result in LLD's higher than required, the reason shall be documented in the semiannual Radioactive Effluent Release Report.

(d) Percentage applicable limits released.

(2) Iodine Releases

(a) Total I-131, I-133, radioactivity (in curies) released.

(b) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(c) Percentage of limit.

(3) Particulate Releases

(a) Gross radioactivity (β-γ) released (in curies) excluding background radioactivity.

(b) Gross alpha radioactivity released (in curies) excluding background radioactivity.

(c) Total radioactivity released (in curies) of nuclides with half-lives greater than eight days.

(d) Percentage of limit.

(4) Liquid Releases

(a) Gross radioactivity (β-γ) released (in curies) excluding tritium and average concentration released to the unrestricted area at the Keowee Hydro unit.

(b) The maximum concentration of gross radioactivity (β-γ) released to the unrestricted area (averaged over the period of release).

(c) Total tritium and alpha radioactivity (in curies) released and average concentration released to the unrestricted area at the Keowee Hydro unit.

(d) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area at the Keowee Hydro unit.

(e) Total volume (in liters) of Keowee Hydro liquid waste released.

(f) Total volume (in liters) of dilution water used prior to release from the restricted area.

(g) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(h) Percentage of limit for total activity released.

DUKE POWER COMPANY
OCONEE NUCLEAR STATION
ONS-S/A-08

TABLE 6.6-1 (continued)
REPORT OF RADIOACTIVE EFFLUENTS

Year _____

II. Airborne Releases

	Units	Jan.	Feb.	Mar.	Apr.	May	June	July	Aug.	Sept.	Oct.	Nov.	Dec.	TOTAL
1. Total noble gases	Curies													
2. Total halogens	Curies													
3. Total particulate gross radioactivity (β,γ)	Curies													
4. Total tritium	Curies													
5. Total particulate gross alpha radioactivity	Curies													
6. Maximum noble gas release rate	μCi/sec													
7. Percent of applicable limit for:														
a. noble gas	%													
b. halogens	%													
c. particulates	%													
8. Isotope released:	Curies													
Particulates														
Cs-137														
Ra-La-140														
Sr-90														
Cs-134														
Sr-89														
Halogens														
I-131														
I-133														
Gases														
Kr-85														
Xe-133														
Kr-88														
Kr-87														
Kr-85m														
Xe-138														
Xe-135m														
Xe-135														
Ar-41														