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SUBJECT: Forwards Request for Relief 88-04 from requirements of Section XI of ASME boiler & pressure vessel code.

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March 16, 1988

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station, Unit 2
Docket No. 50-270
Second Ten Year Interval
Request for Relief No. 88-04

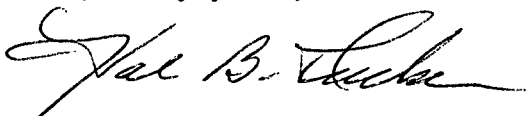
Gentlemen:

Pursuant to 10CFR 50, §50.55a, please find attached request for relief number 88-04 from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (with Addenda through Winter 1980). This request is being submitted due to the availability of access for inspection of the pressure retaining weld in Reactor Coolant Pump 2A2 during the second inspection period of the second ten year interval.

It is requested that this request for relief be reviewed and approved by NRC prior to Unit 2 Cycle 10 startup currently scheduled for April 10, 1988.

This request is considered to supplement the request made by my letter of September 13, 1984. As such, no additional license fees are required.

Very truly yours,



Hal B. Tucker

PJN/306/jgc

Attachment

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Duke Power Company
Oconee Nuclear Station
Second Ten Year Interval
Request for Relief No. 88-04

I. Component for which Exemption is Requested:

- (a) Name and Number: Unit 2 Reactor Coolant Pump
- (b) Function: Recirculates primary coolant water from the Once Through Steam Generator (OTSG) in its respective loop to the reactor vessel
- (c) ASME Section III Code Class: Equivalent Class 1 per NRC Regulatory Guide 1.26
- (d) Valve Category: N/A

II. Reference Code Requirement that has been determined to be impractical:

ASME Boiler and Pressure Vessel Code Section XI, 1980 Edition (with Addenda through Winter 1980) paragraph IWB-2420(a), which states that the sequence of component examinations established during the first inspection interval shall be repeated during each successive inspection interval to the extent practical. In addition, Table IWB-2500-1 Items B12.10 and B12.20 which require a volumetric examination of the pump casing welds and a Visual, VT-3 examination of the pump casing internal surfaces.

III. Basis for Requesting Relief:

The Pressure retaining weld in Reactor Coolant Pump A2 was inspected in the 3rd inspection period of the 1st Ten Year Interval. Due to maintenance activities in the 2nd inspection period of the 2nd Ten Year Interval on Reactor Coolant Pump A1 the pressure retaining weld of the pump casing is now accessible for inspection. The Reactor Coolant Pumps for Units 2 and 3 are manufactured by Bingham-Williamette Company and designed in such a way that a large portion of the internal pressure boundary is inaccessible for visual inspection, and small areas at the outer edges of the volute are inaccessible for volumetric inspection using radiography. The area on the inside radius of the discharge nozzle is too thick (approximately 14") to inspect with any available technique. Ultrasonic inspection is impractical due to the pump casing composed of cast stainless steel having characteristics of large grain size and high attenuation.

IV. Alternate Examination:

The remaining portion of the casing can be visually and volumetrically inspected and the results of this portion of the inspection should be indicative of what conditions exist in the inaccessible areas.

V. Evaluation of acceptability of proposed alternate testing with respect to the level of quality and safety as well as public health and safety:

The examinations proposed under this request for relief are equivalent to the examinations performed during the first ten year interval. The inspection sequence change will have no significant effect on the assurance of weld integrity for the following two reasons:

1. No flaw indications were found in the pressure retaining weld of reactor coolant pump A2 during the first 10-year interval which would require additional (surveillance) reexaminations.
2. Examination of reactor coolant pump A1 pressure retaining weld during the second 10-year interval will produce a broader knowledge of what conditions exist among the reactor coolant pumps.

As such, the proposed examination provides an equivalent level of quality and safety, and will not endanger the health and safety of the public.

VI. Implementation Schedule:

These inspections will be performed during the current (Unit 2 End of Cycle 9) refueling outage. These inspections will be completed during the month of March 1988.