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SUBJECT: Forwards request for relief from requirements of Section XI of ASME Boiler & Pressure Vessel Code. Request submitted due to availability of access for insp of pressure retaining weld in reactor coolant pump 3A1.

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February 9, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station, Unit 3
Docket No. 50-287
RCP ISI Relief Request

Gentlemen:

Pursuant to 10CFR 50, §50.55a, please find attached a request for relief from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (with Addenda through Winter 1980). This request is being submitted due to the availability of access for inspection of the pressure retaining weld in Reactor Coolant Pump 3A1 during the first inspection period of the second ten year interval.

This request is considered to supplement the request made by my letter of September 13, 1984. As such, no additional license fees are required.

Very truly yours,

Hal B. Tucker

Hal B. Tucker

PJN/119/jgm

Attachment

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Duke Power Company
Oconee Nuclear Station
Unit 3

I. Component for which Exemption is Requested:

(a) Name and Number: Reactor Coolant Pump

(b) Function: Recirculates primary coolant water from the Once Through Steam Generator (OTSG) in its respective loop to the reactor vessel

(c) ASME Section III Code Class: 1

(d) Valve Category: N/A

II. Reference Code Requirement that has been determined to be impractical:

ASME Boiler and Pressure Vessel Code Section XI, 1980 Edition (with Addenda through Winter 1980) paragraph IWB-2420(a), which states that the sequence of component examinations established during the first inspection interval shall be repeated during each successive inspection interval to the extent practical. In addition, Table IWB-2500-1 Items B12.10 and B12.20 which require a volumetric examination of the pump casing welds and a Visual, VT-3 examination of the pump casing internal surfaces.

III. Basis for Requesting Relief:

The Pressure retaining weld in Reactor Coolant Pump A1 was inspected in the 3rd inspection period of the 1st Ten Year Interval. Due to maintenance activities in the 1st inspection period of the 2nd Ten Year Interval on Reactor Coolant Pump B1 the pressure retaining weld of the pump casing is now accessible for inspection. The Reactor Coolant Pumps for Units 2 & 3 are manufactured by Bingham-Williamette Company and designed in such a way that a large portion of the internal pressure boundary is inaccessible for visual inspection, and small areas at the outer edges of the volute are inaccessible for volumetric inspection using radiography. The area on the inside radius of the discharge nozzle is too thick (approximately 14") to inspect with any available technique. Ultrasonic inspection is impractical due to the pump casing composed of cast stainless steel having characteristics of large grain size and high attenuation.

IV. Alternate Examination:

The remaining portion of the casing can be visually and volumetrically inspected and the results of this portion of the inspection should be indicative of what conditions exist in the inaccessible areas.

V. Implementation Schedule:

These inspections will be performed during the current (9th) refueling outage. These inspections will be completed during the month of January, 1987.