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A Duke Energy Company

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April 22, 1998

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Annual Radiological Environmental Operating Report

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Pursuant to Oconee Nuclear Station Technical Specification
6.6.1.5, please find enclosed the Oconee Nuclear Site Annual
Radiological Environmental Operating Report for 1997.

Very truly yours,

W. R. McCollum, Jr.
Site Vice President
Oconee Nuclear Station

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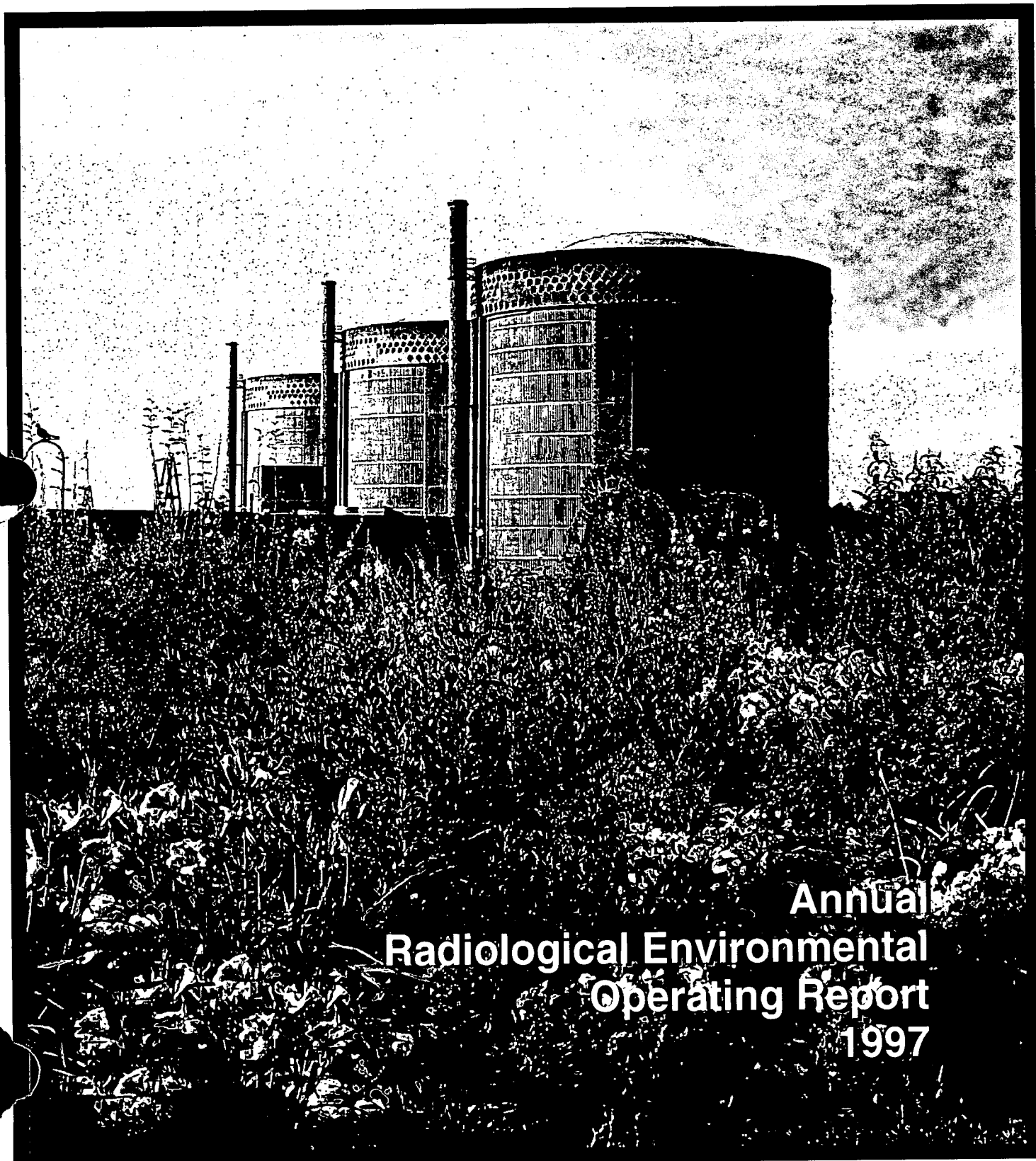
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Oconee Nuclear Station Units 1, 2 and 3



**Annual
Radiological Environmental
Operating Report
1997**



ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT

DUKE POWER COMPANY
OCONEE NUCLEAR STATION
Units 1, 2, and 3

1997

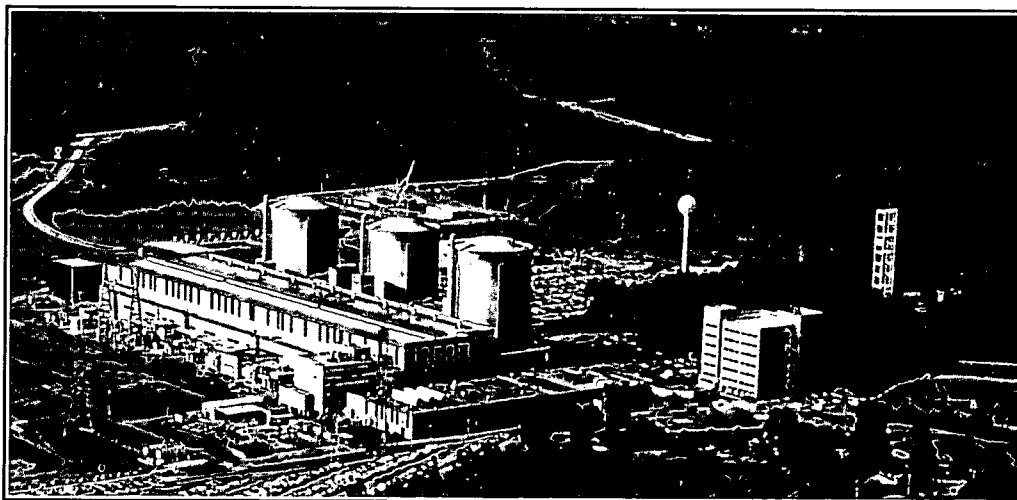


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LIST OF ACRONYMS USED IN THIS TEXT *(in alphabetical order)*

BW	BiWeekly
C	Control
CL	Critical Level
DEHNR	Department of Environmental Health and Natural Resources
DHEC	Department of Health and Environmental Control
EPA	Environmental Protection Agency
LLD	Lower Limit of Detection
M	Monthly
MDA	Minimum Detectable Activity
mrem	millirem
NIST	National Institute of Standards and Technology
NRC	Nuclear Regulatory Commission
ODCM	Offsite Dose Calculation Manual
pCi/kg	picocurie per kilogram
pCi/l	picocurie per liter
pCi/m3	picocurie per cubic meter
Q	Quarterly
REMP	Radiological Environmental Monitoring Program
SA	Semiannually
SLCs	Selected Licensee Commitments
SM	Semimonthly
TECH SPECS	Technical Specifications
TLD	Thermoluminescent Dosimeter
μCi/ml	microcurie per milliliter
UFSAR	Updated Final Safety Analysis Report
W	Weekly

1.0 EXECUTIVE SUMMARY

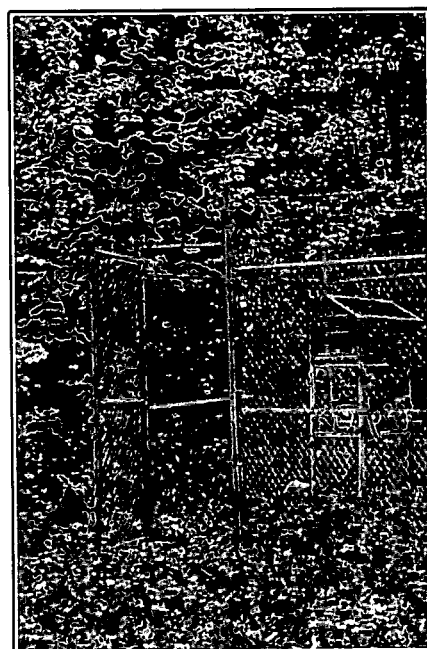
This Annual Radiological Environmental Operating Report describes the Oconee Nuclear Station Radiological Environmental Monitoring Program (REMP), and the program results for the calendar year 1997.

Included are the identification of sampling locations, descriptions of environmental sampling and analysis procedures, comparisons of present environmental radioactivity levels and pre-operational environmental data, comparisons of doses calculated from environmental measurements and effluent data, analysis of trends in environmental radiological data as potentially affected by station operations, and a summary of environmental radiological sampling results. Quality assurance practices and program changes are also discussed. All samples were successfully collected without any deviation from sampling procedures.

Sampling activities were conducted as prescribed by Selected Licensee Commitments (SLC's). Required analyses were performed and detection capabilities were met for all samples as required by SLC's. Supplemental analyses were performed for some media for additional information. One-thousand fifty samples were analyzed comprising 1518 test results in order to compile data for the 1997 report. Based on the annual land use census, the current number of sampling sites for Oconee Nuclear Station is sufficient.

Concentrations observed in the environment in 1997 for station related radionuclides were within the ranges of concentrations observed in the past. Inspection of data showed that radioactivity concentrations in surface water, drinking water, shoreline sediment, and fish are higher than the activities reported for samples collected prior to the operation of the station. Measured concentrations were not higher than expected, and all positively identified measurements were within limits as specified in SLC's.

Additionally, environmental radiological monitoring data is consistent with effluents introduced into the environment by plant operations. The total body dose estimated to the maximum exposed member of the public as calculated by environmental sampling data, excluding TLD results, was $1.4\text{E-}01$ mrem for 1997. It is therefore concluded that station operations has had no significant radiological impact on the health and safety of the public or the environment.



Air Sampling at
Oconee Nuclear Station

2.0 INTRODUCTION

2.1 SITE DESCRIPTION AND SAMPLE LOCATIONS

Oconee Nuclear Station (ONS) is located in Oconee County, South Carolina, approximately 8 miles northeast of Seneca, South Carolina, on the shore of Lake Keowee. This lake was formed by damming the Keowee and Little Rivers in that location. Immediately to the south is the U.S. Government Hartwell Project. The Keowee Hydroelectric Plant near the station joins Lake Keowee and the upper reaches of Lake Hartwell. To the north, the Jocassee Hydroelectric Plant joins Lake Jocassee and Lake Keowee. Jocassee is a pumped storage plant.

ONS consists of three pressurized water reactors. Each unit has an output of 866 megawatts net. Unit 1 began commercial operation 7/15/73. Unit 2 began commercial operation 9/09/74, and Unit 3 on 12/16/74.

Site specific locations for the Radiological Environmental Monitoring Program are defined in the Duke Power Company Offsite Dose Calculation Manual (ODCM). Figures 2.1-1 and 2.1-2 are maps depicting the Thermoluminescent Dosimeter (TLD) monitoring locations and the sampling locations. The samples obtained from the locations include Airborne Radioiodine and Particulates, Drinking Water, Surface Water, Milk, Broadleaf Vegetation, Shoreline Sediment and Fish. Table 2.1-A lists the specific samples required for each location. Table 2.1-B lists the locations of all the TLDs.

2.2 SCOPE AND REQUIREMENTS OF THE REMP

An environmental monitoring program has been in effect at Oconee Nuclear Station since 1969, four years prior to operation of Unit 1 in 1973. The preoperational program provides data on the existing environmental radioactivity levels for the site and vicinity which may be used to determine whether increases in environmental levels are attributable to the station. The operational program provides surveillance and backup support of detailed effluent monitoring which is necessary to evaluate the significance, if any, of the contributions to the existing environmental radioactivity levels that result from station operation.

This monitoring program is based on NRC guidance as reflected in the Selected Licensee Commitments Manual, with regard to sample media, sampling locations, sampling frequency, and analytical sensitivity requirements. Indicator and control locations were established for comparison purposes to distinguish radioactivity of station origin from natural or other "man-made" environmental radioactivity. The environmental monitoring program also verifies projected and anticipated radionuclide concentrations in the environment and related exposures from releases of radionuclides from Oconee Nuclear Station. This program satisfies the requirements of Section IV.B.2 of Appendix I to 10CFR50 and provides surveillance of all appropriate critical exposure pathways to man and protects vital interests of the company,

public, and state and federal agencies concerned with the environment. Reporting levels for radioactivity found in environmental samples are listed in Table 2.2-A. Table 2.2-B lists the REMP analysis and frequency schedule.

The Annual Land Use Census, required by Selected Licensee Commitments, is performed to ensure that changes in the use of areas at or beyond the site boundary are identified and that modifications to the Radiological Environmental Monitoring Program are made if required by changes in land use. This census satisfies the requirements of Section IV.B.3 of Appendix I to 10CFR50. Results are shown in Table 3.9.

Participation in an interlaboratory comparison program as required by Selected Licensee Commitments provides for independent checks on the precision and accuracy of measurements of radioactive material in REMP sample matrices. Such checks are performed as part of the quality assurance program for environmental monitoring in order to demonstrate that the results are valid for the purposes of Section IV.B.2 of Appendix I to 10CFR50. A summary of the results obtained as part of this comparison program are in Section 5 of this annual report.

2.3 STATISTICAL AND CALCULATIONAL METHODOLOGY

2.3.1 ESTIMATION OF THE MEAN VALUE

There was one (1) basic statistical calculation performed on the raw data resulting from the environmental sample analysis program. The calculation involved the determination of the mean value for the indicator and the control samples for each sample medium. The mean is a widely used statistic. This value was used in the reduction of the data generated by the sampling and analysis of the various media in the Radiological Environmental Monitoring Program. The following equation was used to estimate the mean (reference 6.8):

$$\bar{x} = \frac{\sum_{i=1}^N x_i}{N}$$

Where:

\bar{x} = estimate of the mean,

i = individual sample,

N = total number of samples with a net activity (or concentration)

x_i = net activity (or concentration) for sample i .

NOTE: "Net activity (or concentration)" is the activity (or concentration) determined to be present in the sample. No "Minimum Detectable Activity", "Lower Limit of

Detection", "Less Than Level", or negative activities or concentrations are included in the calculation of the mean.

2.3.2 LOWER LEVEL OF DETECTION, MINIMUM DETECTABLE ACTIVITY, AND CRITICAL LEVEL

The Lower Level of Detection (LLD), Minimum Detectable Activity (MDA), and Critical Level (CL) are used throughout the Environmental Monitoring Program.

LLD - The LLD, as defined in the Selected Licensee Commitments Manual is the smallest concentration of radioactive material in a sample that will yield a net count, above the system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal. The LLD is an *a priori* lower limit of detection. The actual LLD is dependent upon the standard deviation of the background counting rate, the counting efficiency, the sample size (mass or volume), the radiochemical yield, and the radioactive decay of the sample between sample collection and counting. The "required" LLD's for each sample medium and selected radionuclides are given in the Selected Licensee Commitments and are listed in Table 2.2-C.

MDA - The MDA may be thought of as an "actual" LLD for a particular sample measurement remembering that the MDA is calculated using a sample background instead of a system background.

CL - The CL is defined as the net count rate which must be exceeded before a sample is considered to contain any measurable activity above the background.

2.3.3 TREND IDENTIFICATION

One of the purposes of an environmental monitoring program is to determine if there is a buildup of radionuclides in the environment due to the operation of the nuclear station. Visual inspection of tabular or graphical presentations of data (including preoperational) is used to determine if a trend exists. A decrease in a particular radionuclide's concentration in an environmental medium does not indicate that reactor operations are removing radioactivity from the environment but that reactor operations are not adding that radionuclide to the environment in quantities exceeding the preoperational level and that the normal removal processes (radioactive decay, deposition, resuspension, etc.) are influencing the concentration.

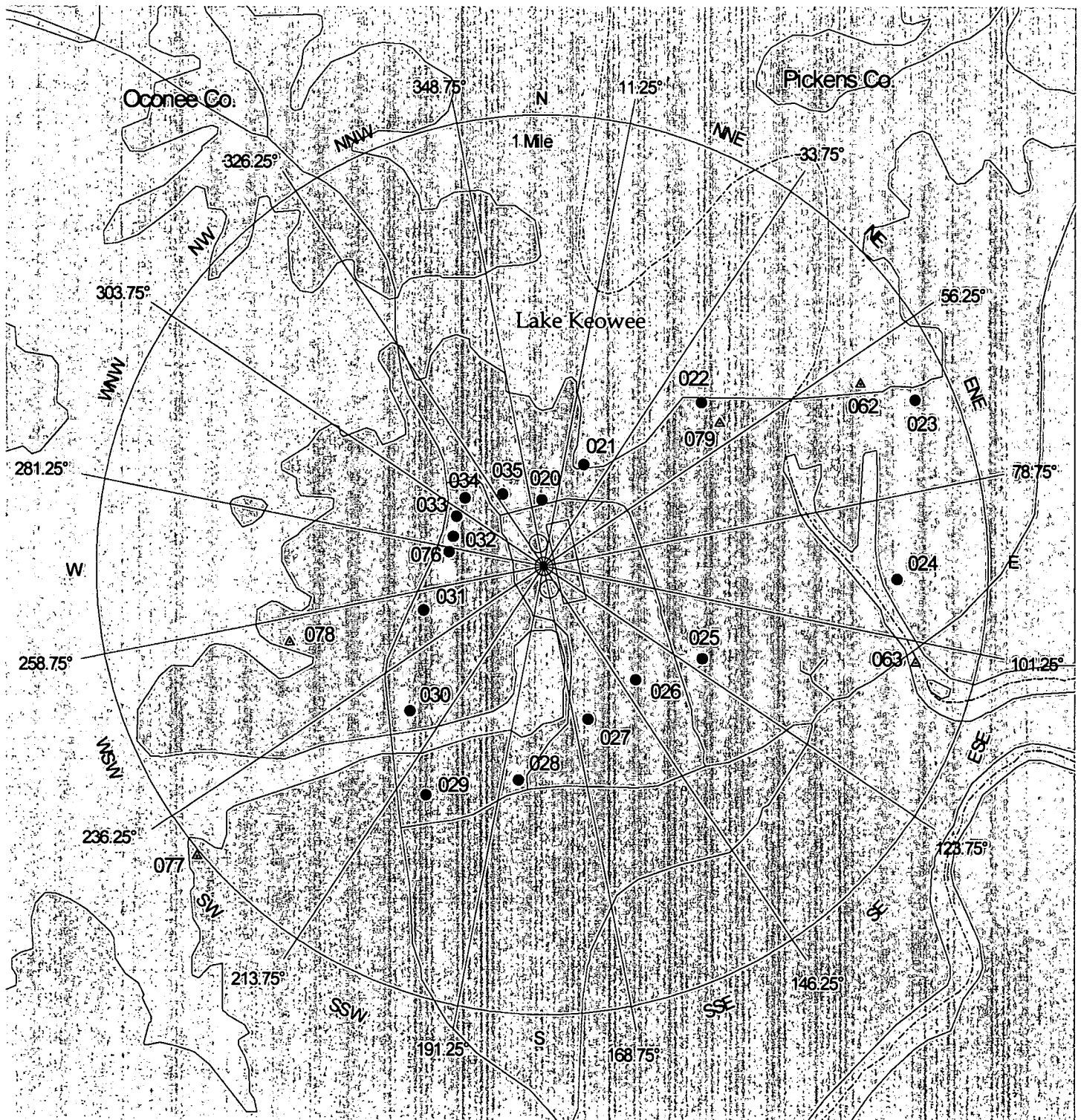
Substantial increases or decreases in the amount of a particular radionuclide's release from the nuclear plant will greatly affect the resulting environmental levels; therefore, a knowledge of the release of a radionuclide from the nuclear plant is necessary to completely interpret the trends, or lack of trends, determined from the environmental data. Some factors that may affect environmental levels of radionuclides include prevailing weather conditions (periods of drought, solar cycles or heavier than normal

precipitation), construction in or around either the nuclear plant or the sampling location, and addition or deletion of other sources of radioactive materials (such as the Chernobyl accident). Some of these factors may be obvious while others are sometimes unknown. Therefore, how trends are identified will include some judgment by plant personnel.

Oconee Nuclear Station

Figure 2.1-1

Sampling Locations Map (Site Boundary)



- TLD Locations
- ▲ All Other Locations



Oconee Nuclear Station

Figure 2.1-2

Sampling Locations Map (Ten Mile Radius)

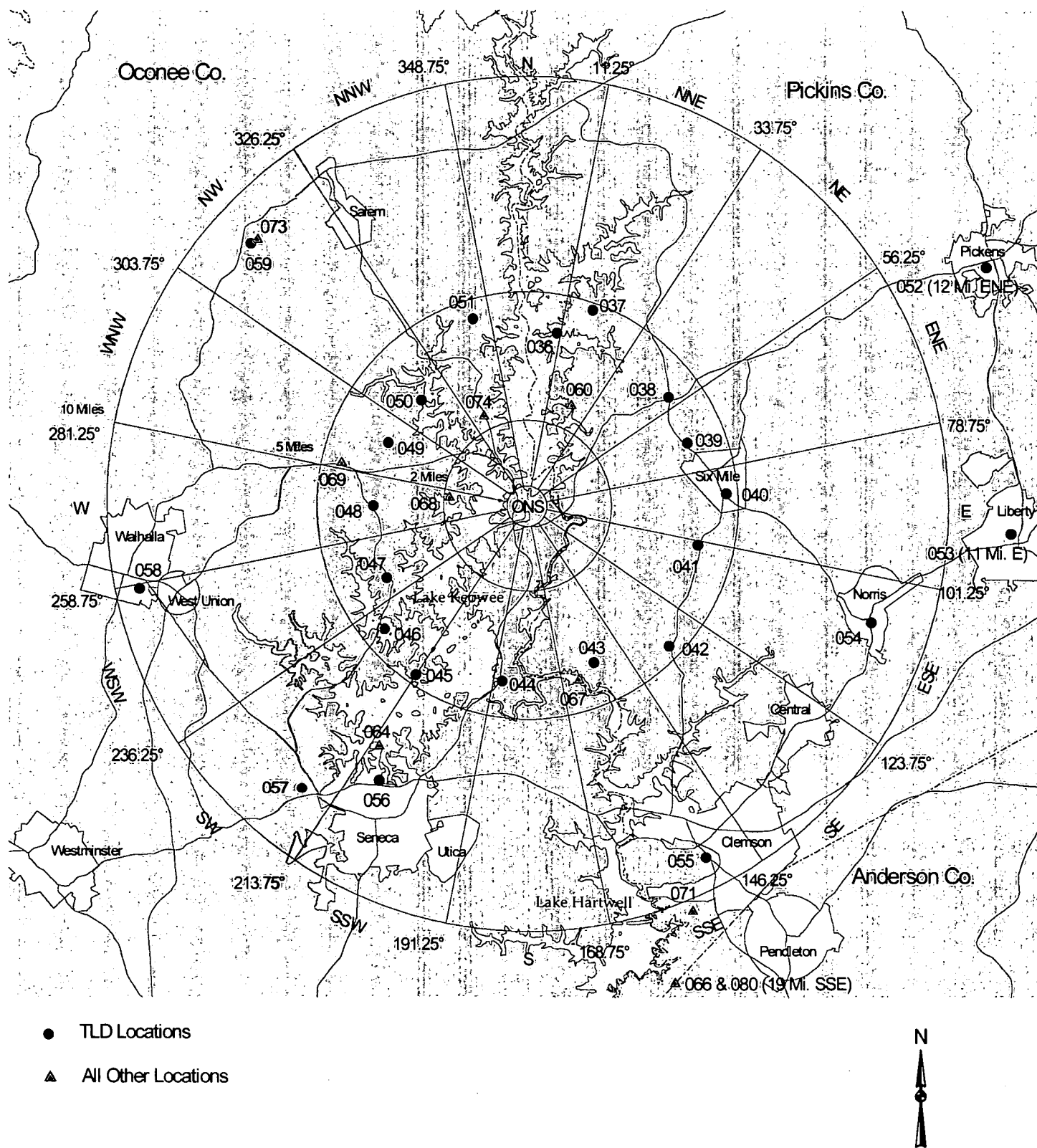


TABLE 2.1-A

OCONEE RADIOLOGICAL MONITORING PROGRAM SAMPLING LOCATIONS

Table 2.1-A Codes			
W	Weekly	SM	Semimonthly
BW	BiWeekly	Q	Quarterly
M	Monthly	SA	Semiannually
C	Control		

Site #	Location Description	Air Rad. & Particulate	Surface Water	Drinking Water	Shoreline Sediment	Fish	Milk	Broadleaf Vegetation
060	Greenville Water Intake Road (2.6 mi NNE)*	W		M		SA		M
062 C	Lake Keowee Hydro Intake (0.8 mi ENE)		M					
063	Lake Hartwell Hwy 183 Bridge (0.8 mi ESE) [000.7]		M		SA	SA		
064 C	Seneca (6.7 mi SSW) [004.1]			M				
066	Anderson (19.0 mi SSE) [012]			M				
067	Lawrence Ramsey Bridge Hwy 27 (4.2 mi SSE) [005.2]				SA	SA		
068 C	High Falls County Park (2.0 mi W)				SA			
069	Orr Dairy (4.5 mi WNW) [002.1]						SM	
071	Clemson Dairy (10.3 mi SSE) [006.3]						SM	
073 C	Tamassee DAR School (9.2 mi NW)	W						M
074	Keowee Key Resort (2.3 mi NNW)	W						
077	Skimmer Wall (1.0 mi SW)	W						M
078	Recreation Site (0.6 mi WSW)	W						
079	Keowee Dam (0.5 mi NE)	W						M
080C	Martin Dairy (19.0 mi SSE)						SM	

* Control for Fish Only

[] Location Numbers prior to 1984

TABLE 2.1-B

**OCONEE RADIOLOGICAL MONITORING PROGRAM
SAMPLING LOCATIONS**

(TLD SITES)

Site #	Location Description		Site #	Location Description	
020	SITE BOUNDARY	0.1 miles N	040	MICROWAVE TOWER, SIX MILE	4.5 miles E
021	SITE BOUNDARY	0.3 miles NNE	041	JCT HWY 101 & 133	4.0 miles ESE
022	SITE BOUNDARY	0.5 miles NE	042	LAWRENCE CHAPEL CHURCH, HWY 133	5.0 miles SE
023	SITE BOUNDARY	0.9 miles ENE	043	HWY 291 AT ISSAQUEENA PARK ENTRANCE	4.0 miles SSE
024	SITE BOUNDARY	0.8 miles E	044	HWY 130 AT LITTLE RIVER DAM	4.0 miles S
025	SITE BOUNDARY	0.4 miles ESE	045	TERMINUS OF HWY 588 AT CROOKED CREEK	5.0 miles SSW
026	SITE BOUNDARY	0.3 miles SE	046	HWY 188 AT CROOKED CREEK BRIDGE	4.5 miles SW
027	SITE BOUNDARY	0.4 miles SSE	047	NEW HOPE CHURCH, HWY 188	4.0 miles WSW
028	SITE BOUNDARY	0.5 miles S	048	JCT HWY 175 & 188	4.0 miles W
029	SITE BOUNDARY	0.6 miles SSW	049	JCT HWY 201 & 92	4.0 miles WNW
030	SITE BOUNDARY	0.4 miles SW	050	STAMP CREEK LANDING - END OF HWY 92	4.0 miles NW
031	SITE BOUNDARY	0.3 miles WSW	051	HWY 128, 1 MILE N OF HWY 130	4.5 miles NNW
076	SITE BOUNDARY	0.2 miles W	052	DPC BRANCH OFFICE SITE - PICKENS	12.0 miles ENE
032	SITE BOUNDARY	0.2 miles WNW	053	DPC BRANCH OFFICE SITE - LIBERTY	11.0 miles E
033	SITE BOUNDARY	0.2 miles WNW	054	POST OFFICE - HWY 93 NORRIS	9.5 miles ESE
034	SITE BOUNDARY	0.2 miles NW	055	CLEMSON METEOROLOGY PLOT	9.5 miles SSE
035	SITE BOUNDARY	0.2 miles NNW	056	WATER TOWER - SENECA	8.4 miles SSW
036	MILE CREEK LANDING	4.0 miles N	057	OCONEE MEMORIAL HOSPITAL	9.0 miles SW
037	KEOWEE CHURCH, HWY 327	4.5 miles NNE	058 C	BRANCH RD SUBSTATION WALHALLA, CONTROL	9.4 miles WSW
038	DURHAM CONVENIENCE MART, JCT HWY 183 & 133	4.0 miles NE	059	TAMASSEE DAR SCHOOL	9.2 miles NW
039	HWY 133, 1 MILE EAST OF JCT HWY 183 & 133	4.0 miles ENE			

TABLE 2.2-A

**REPORTING LEVELS FOR RADIOACTIVITY
CONCENTRATIONS IN ENVIRONMENTAL SAMPLES**

Analysis	Water (pCi/liter)	Air Particulates or Gases (pCi/m ³)	Fish (pCi/kg-wet)	Milk (pCi/liter)	Broadleaf Vegetation (pCi/kg-wet)
H-3	20,000 ^(a)				
Mn-54	1,000		30,000		
Fe-59	400		10,000		
Co-58	1,000		30,000		
Co-60	300		10,000		
Zn-65	300		20,000		
Zr-Nb-95	400				
I-131	2 ^(b)	1		3	100
Cs-134	30	10	1,000	60	1,000
Cs-137	50	20	2,000	70	2,000
Ba-La-140	200			300	

(a) For drinking water samples. This is 40CFR Part 141 value.

(b) If low-level I-131 analyses are performed.

TABLE 2.2-B

REMP ANALYSIS FREQUENCY

Sample Medium	Analysis Schedule	Gamma Isotopic	Tritium	Low Level I-131	Gross Beta	TLD
Air Radioiodine and Particulates	Weekly	X				
Direct Radiation	Quarterly					X
Surface Water	Monthly	X				
	Quarterly Composite		X			
Drinking Water	Monthly	X		X	X	
	Quarterly Composite		X			
Shoreline Sediment	Semiannually	X				
Milk	Semimonthly	X		X		
Fish	Semiannually	X				
Broadleaf Vegetation	Monthly	X				

TABLE 2.2-C

MAXIMUM VALUES FOR THE LOWER LIMITS OF DETECTION

Analysis	Water (pCi/liter)	Air Particulates or Gases (pCi/m ³)	Fish (pCi/kg-wet)	Milk (pCi/liter)	Broadleaf Vegetation (pCi/kg-wet)	Sediment (pCi/kg-dry)
Gross Beta	4					
H-3	2000					
Mn-54	15		130			
Fe-59	30		260			
Co-58, 60	15		130			
Zn-65	30		260			
Zr-95	30					
Nb-95	15					
I-131	15 ^(a)	0.07		1	60	
Cs-134	15	0.05	130	15	60	150
Cs-137	18	0.06	150	18	80	180
Ba-La-140	15			15		

(a) LLD for low-level I-131 analyses is 1 pCi/liter

3.0 INTERPRETATION OF RESULTS

Review of 1997 REMP analysis results was performed to identify changes in environmental levels as a result of station operations. The review is summarized in this section. Data from 1997 was compared to preoperational and historical data. Sample data for some media is not directly comparable to preoperational and earlier operational sample results because of either significant changes in the analysis methods or changes in the reporting of the results.

Evaluation for significant trends was performed for the radionuclides that have required LLDs listed in Selected Licensee Commitment 16.11-6. These radionuclides are collectively referred to as "Selected Licensee Commitments radionuclides" and include H-3, Mn-54, Fe-59, Co-58, Co-60, Zn-65, Zr-95, Nb-95, I-131, Cs-134, Cs-137, Ba-140, and La-140. Drinking water gross beta results are routinely trended. Trending of air particulate gross beta results was initiated in 1996 when the analysis was resumed. Trending is also performed for other radionuclides that are detected and could have been the result of station effluents. Only Selected Licensee Commitment radionuclides were detected in 1997.

Trending was performed by comparing annual mean concentrations of any effluent related detected radionuclide to historical results. Factors evaluated include the frequency of detection and the concentration in terms of the percent of the radionuclide's NRC reporting level (Table 2.2-A). All maximum percent of reporting level values were well below the 100% action level.

The highest value reached during 1997 was 55%, for surface water tritium collected during the fourth quarter at the discharge Location 063.

Changes in sample location, analytical technique, and presentation of results must be considered when reviewing for trends. Calculation of the annual mean concentrations has been performed differently over the history of the REMP. During 1979-1986, all net results (sample minus background), positive and negative, were included in the calculation of the mean. Only positive net activity results were used to calculate the mean for the other years. A change in gamma spectroscopy analysis systems in 1987 ended a period when many measurements yielded detectable low-level activity for both indicator and control location samples. It is thought that the method the previous system used to estimate net activity may have been vulnerable to false-positive results.

Data presented in Sections 3.1 - 3.8 support the conclusion that there were no significant increases in radionuclides in the environment around ONS due to station operations in 1997. Similarly, there was no significant increase in ambient background radiation levels in the surrounding areas.

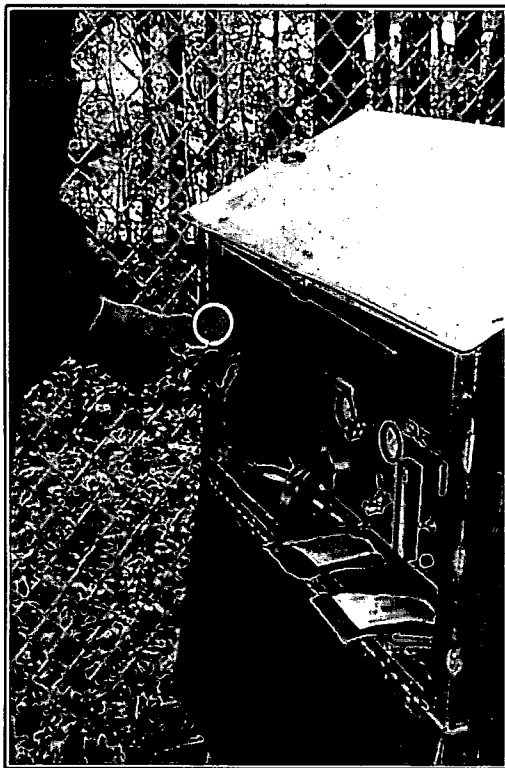
3.1 AIRBORNE RADIOIODINE AND PARTICULATES

Gamma spectroscopy was performed on 312 fiber filters and charcoal cartridges collected during 1997. Five indicator locations and one control location were sampled.

There was no detectable I-131 in air samples in 1997. Table 3.1-A gives the highest indicator location annual mean and control location annual mean for I-131 since the preoperational period. The table shows similar concentrations for both the indicator and control locations and the activities decreasing from early in the operational history of the plant. No I-131 has been detected since 1994.

Cs-137 was not detected in any air radioiodine samples in 1997. In 1996, Cs-137 was detected in one indicator location cartridge sample at a concentration that was 0.08% of the reporting level. However, no Cs-137 was found on the corresponding particulate filter. Also in 1995, Cs-137 was detected in two indicator and two control location cartridges with activities ranging 0.01% to 0.05% of the reporting level. An investigation performed in 1990 lead to the conclusion that Cs-137 activity detected only on the cartridges was not attributed to station effluents but is an active constituent of the charcoal (reference 6.5).

There were no detectable gamma emitting radionuclides detected in air particulate samples in 1997. No gamma emitting particulates have been detected in indicator location samples since the change in gamma spectroscopy analysis systems in 1987.



Oconee Air Monitoring Station

Beta analysis of particulate filters was initiated in March of 1996. Selected Licensee Commitments currently requires only gamma spectroscopy analysis of the particulate filters. Gross beta analysis was performed on particulate filters during the preoperational and early operational history of the plant but has not been required since 1984. Figure 3.1 shows the gross beta results for the indicator location with the highest annual mean and the control location samples. Both the indicator and control location results are similar in concentration and are near the lower range of preoperational gross beta results.

K-40 and Be-7 are the naturally occurring radionuclides that were observed in air samples.

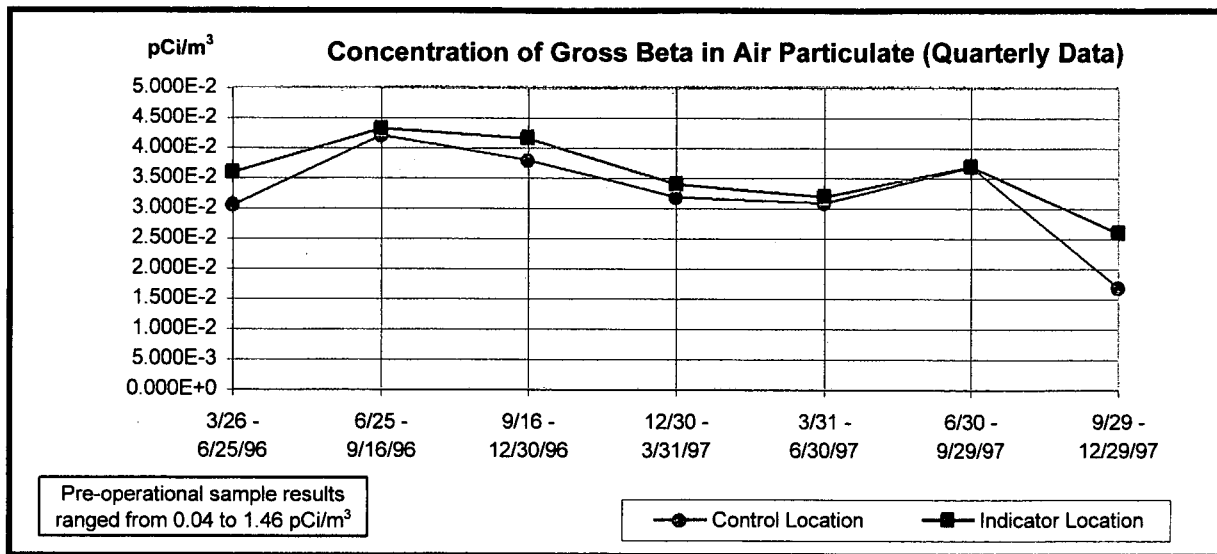
Table 3.1-A Mean Concentration of Air Radioiodine (I-131)

Year	Indicator Location (pCi/m ³)	Control Location (pCi/m ³)
Preoperational 1969-1972	0.00E0	0.00E0
Feb. 1973 - June 1973	0.00E0	0.00E0
July 1973 - Dec. 1973	0.00E0	0.00E0
Jan. 1974 - June 1974	0.00E0	0.00E0
July 1974 - Dec. 1974	2.60E-2	8.00E-3
Jan. 1975 - June 1975	8.65E-2	3.12E-2
July 1975 - Dec. 1975	1.13E-2	9.52E-3
1976	2.76E-2	2.18E-2
1977	3.60E-2	3.60E-2
1978	2.19E-1	1.15E-1
1979	7.54E-3	4.75E-4
1980	3.07E-3	9.67E-4
1981	6.31E-3	5.39E-4
1982	2.87E-3	8.10E-4
1983	1.48E-3	3.05E-4
1984	8.11E-4	-2.30E-5
1985	7.71E-4	4.54E-4
1986	5.02E-3	7.86E-3
1987	4.29E-3	5.19E-3
1988	0.00E0	0.00E0
1989	4.99E-4	0.00E0
1990	0.00E0	0.00E0
1991	0.00E0	0.00E0
1992	0.00E0	0.00E0
1993	0.00E0	0.00E0
1994	1.03E-2	0.00E0
1995	0.00E0	0.00E0
1996	0.00E0	0.00E0
1997	0.00E0	0.00E0

0.00E0 = no detectable measurements

1979 - 1986 mean based on all net activity results

Figure 3.1



There is no reporting level for gross beta in air particulate

Table 3.1-B Mean Concentration of Gross Beta in Air Particulate

Monitoring Period	Indicator Location (pCi/m ³)	Control Location (pCi/m ³)
3/26 - 6/25/96	3.28E-2	3.05E-2
6/25 - 9/16/96	4.15E-2	4.22E-2
9/16 - 12/30/96	4.16E-2	3.80E-2
Average (1996)	3.86E-2	3.69E-2
12/30 - 3/31/97	3.22E-2	3.18E-2
3/31 - 6/30/97	3.19E-2	3.14E-2
6/30 - 9/29/97	3.59E-2	3.72E-2
9/29 - 12/29/97	1.48E-2	1.65E-2
Average (1997)	2.87E-2	2.92E-2

3.2 DRINKING WATER

Gross beta analysis and gamma spectroscopy were performed on 39 monthly drinking water samples. These samples were composited to form 15 quarterly period samples for Tritium analysis. Two indicator locations and a control location were sampled; however, only one of the indicator locations is downstream of the effluent release point.

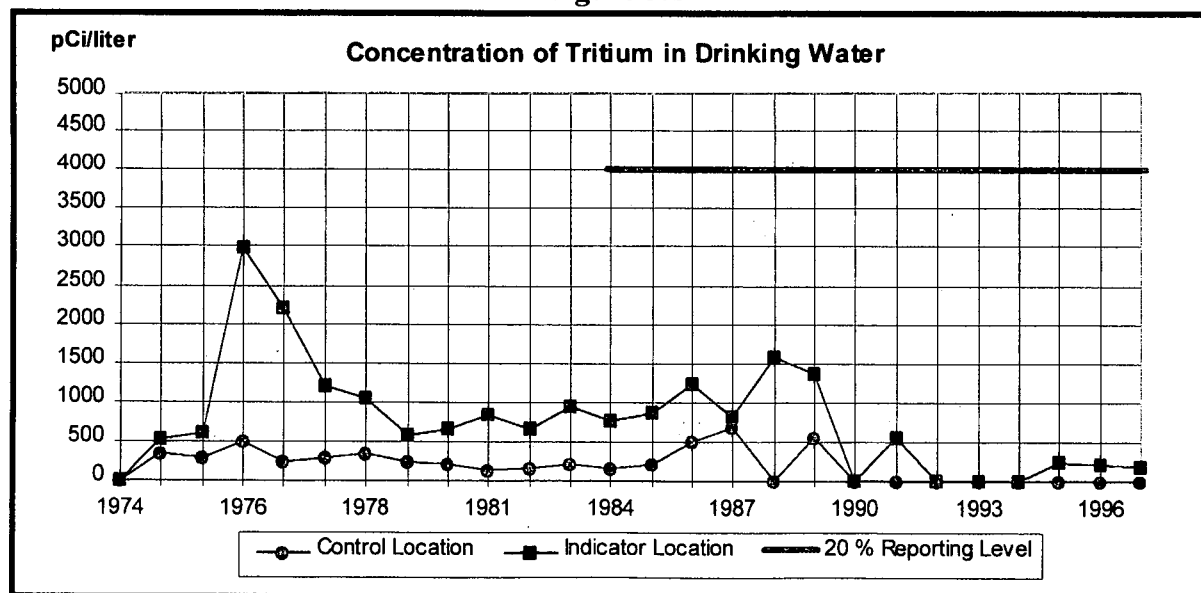
Table 3.2 lists the highest indicator location annual mean and control location annual mean for gross beta results since the preoperational period. The indicator location had an average concentration of 2.52 pCi/liter in 1997, and the control location had a similar concentration of 2.23 pCi/liter. The 1996 indicator mean was 2.07 pCi/liter. The table shows that 1997 gross beta levels in drinking water are slightly lower than preoperational concentrations.

Low levels of Tritium were detected in three of the five downstream indicator location samples in 1997. The mean concentration was 194 pCi/liter, which is 1.0% of the reporting level. No Tritium was detected in drinking water samples from other locations. The downstream indicator location had a mean concentration of 214 pCi/liter in 1996. Table 3.2 and Figure 3.2 show the highest indicator and control location annual means for Tritium since analysis was initiated early in the operational period. Tritium concentrations have decreased at both the indicator and control locations. The closure of the Clemson water plant in 1989 is one reason for the decrease shown in the table and graph. The Clemson site was typically the high mean location when the plant was in operation. However, Tritium concentrations at the current downstream indicator location, Anderson water plant, have also decreased.

There were no gamma emitting radionuclides identified in drinking water samples in 1997. Gamma spectroscopy analysis has not detected any activity in the water supplies since 1988.

K-40 is the naturally occurring radionuclide that was observed in drinking water samples.

Figure 3.2



Current reporting levels implemented 1984

Table 3.2 Mean Concentrations of Radionuclides in Drinking Water

Year	Gross Beta (pCi/l)		Tritium (pCi/l)	
	Indicator Location	Control Location	Indicator Location	Control Location
Preoperational ending Jan. 1971	3.03	5.90	Analysis not required	
Preoperational ending Jan. 1973	3.58	4.94	Analysis not required	
Feb. 1973 - June 1973	Qualitative results reported		Analysis not required	
June 1973 - Dec. 1973	7.15	21.78	Analysis not required	
Jan. 1974 - June 1974	3.13	6.98	Analysis not required	
July 1974 - Dec. 1974	2.24	2.02	525	330
Jan. 1975 - June 1975	1.98	1.59	600	300
July 1975 - Dec. 1975	2.01	1.22	2990	505
1976	2.38	2.00	2196	224
1977	2.70	2.30	1200	290
1978	2.56	2.17	1050	333
1979	1.83	1.36	576	235
1980	1.86	1.63	660	200
1981	1.98	1.88	830	127
1982	2.04	1.45	643	153
1983	1.85	1.54	937	220
1984	1.87	1.08	765	145
1985	2.14	1.16	856	210
1986	1.91	1.04	1240	503
1987	2.00	1.20	815	680
1988	2.00	1.40	1570	0.00
1989	2.30	1.80	1350	559
1990	3.00	2.70	0.00	0.00
1991	1.80	1.40	558	0.00
1992	3.20	1.60	0.00	0.00
1993	2.10	1.90	0.00	0.00
1994	1.90	2.10	0.00	0.00
1995	5.10	2.90	248	0.00
1996	2.07	1.77	214	0.00
1997	2.52	2.23	194	0.00

0.00 = no detectable measurements

1989 - Clemson water plant closes; nearest downstream plant is Anderson.

1979 - 1986 mean based on all net activity results

3.3 SURFACE WATER

Gamma spectroscopy was performed on 26 monthly surface water samples. These samples were composited to form 10 quarterly samples for Tritium analysis. One indicator and one control location were sampled. The indicator location is near the liquid effluent release point.

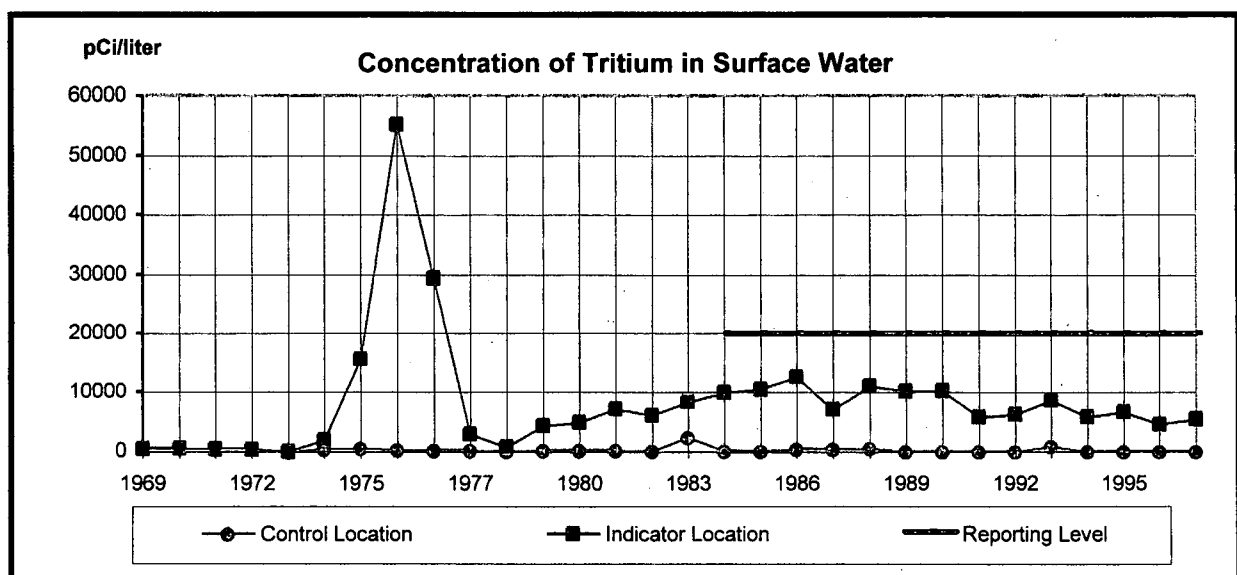
Only Tritium was detected in the samples. All five indicator location samples contained Tritium. The 1997 average concentration was 5497 pCi/liter. This is 27% of the reporting level used for Tritium in surface water. The individual samples ranged from 9.7% to 55% of the reporting level. The 1996 mean concentration was 4535 pCi/liter. Tritium was not detected in any control surface water samples.

Figure 3.3 shows the indicator and control annual means for Tritium since the preoperational period. Table 3.3 lists the indicator annual means. Tritium concentrations exceed preoperational levels but are not increasing.

Gamma spectroscopy analysis has not detected any activity in surface water samples since 1992. Table 3.3 summarizes the indicator annual means of radionuclides detected since the change in the gamma spectroscopy analysis system in 1987. Visual inspection of the tabular data covering the early operational period through 1997 did not reveal any increasing trends.

K-40 is the naturally occurring radionuclide observed in surface water samples in 1997.

Figure 3.3



Current reporting levels implemented 1984

Table 3.3 Mean Concentrations of Radionuclides in Surface Water

Year	Co-58 (pCi/l)	Co-60 (pCi/l)	Nb-95 (pCi/l)	Cs-137 (pCi/l)	H-3 pCi/l)
Preoperational 1969	Qualitative results reported				4.86E2
Preoperational 1970	“				5.94E2
Preoperational 1971	“				4.01E2
Preoperational 1972	“				3.62E2
1973	“				0.00E0
1974	0.00E0	1.32E1	0.00E0	1.60E1	1.99E3
Jan. 1975 - June 1975	0.00E0	0.00E0	0.00E0	0.00E0	1.56E4
July 1975 - Dec. 1975	0.00E0	1.34E1	0.00E0	0.00E0	5.52E4
1976	1.08E2	3.30E1	0.00E0	3.50E1	2.95E4
1977	2.60E1	1.80E1	0.00E0	3.10E1	2.90E3
1978	2.96E2	0.00E0	0.00E0	2.22E1	8.00E2
1979	1.33E0	2.60E0	1.78E0	2.82E0	4.37E3
1980	1.56E0	2.30E0	1.22E0	5.40E0	4.93E3
1981	1.10E0	6.10E-1	1.70E0	3.90E0	7.21E3
1982	6.14E-1	1.99E0	2.29E0	4.85E0	6.13E3
1983	6.99E-1	3.02E0	3.91E-1	6.83E-1	8.40E3
1984	9.40E-1	6.30E-1	7.90E-1	4.83E-1	9.90E3
1985	2.15E-1	6.27E-1	4.95E-1	9.90E-1	1.05E4
1986	3.28E0	1.23E0	1.14E0	3.07E-1	1.26E4
1987	5.10E1	3.40E0	4.00E0	0.00E0	7.08E3
1988	6.20E0	5.00E0	2.50E0	3.50E0	1.10E4
1989	5.30E0	3.00E0	0.00E0	3.40E0	1.02E4
1990	1.70E0	1.60E0	0.00E0	0.00E0	1.03E4
1991	5.40E0	0.00E0	0.00E0	0.00E0	5.76E3
1992	2.50E0	0.00E0	0.00E0	0.00E0	6.22E3
1993	0.00E0	0.00E0	0.00E0	0.00E0	8.62E3
1994	0.00E0	0.00E0	0.00E0	0.00E0	5.75E3
1995	0.00E0	0.00E0	0.00E0	0.00E0	6.65E3
1996	0.00E0	0.00E0	0.00E0	0.00E0	4.54E3
1997	0.00E0	0.00E0	0.00E0	0.00E0	5.50E3

0.00E0 = no detectable measurements

1979-1986 mean based on all net activity results

3.4 MILK

Gamma spectroscopy and low level iodine analysis was performed on 78 milk samples collected in 1997. Two indicator and one control location were sampled.



Milk Sampling

There were no gamma emitting radionuclides identified in indicator or control location samples in 1997. In 1996 Cs-137 was detected in one of the control location samples at a concentration of 4.1 pCi/liter (6% of the reporting level). Cs-137 in milk is not unusual. It is a constituent of nuclear weapons test fallout and has been observed in samples from indicator and control locations in previous years.

Table 3.4 lists the highest indicator location annual mean and control location annual mean for Cs-137 since the preoperational period. The table shows similar concentrations for both indicator and control locations. Cs-137 is the only radionuclide, other than naturally occurring K-40, reported in milk samples since 1988.

Table 3.4 Mean Concentration of Radionuclides in Milk

Year	Cs-137 Indicator (pCi/l)	Cs-137 Control (pCi/l)
Preoperational	1.57E1	1.46E1
Feb. 1973 - June 1973	Qualitative results reported	Qualitative results reported
July 1973 - Dec. 1973	5.80E0	"
Jan. 1974 - June 1974	5.30E0	0.00E0
July 1974 - Dec. 1974	1.11E1	0.00E0
Jan. 1975 - June 1975	1.51E1	9.45E0
July 1975 - Dec. 1975	0.00E0	0.00E0
1976	1.80E1	7.47E0
1977	0.00E0	0.00E0
1978	1.33E1	1.33E1
1979	7.25E0	2.52E0
1980	3.58E0	2.63E0
1981	5.52E0	5.51E0
1982	2.71E0	3.25E0
1983	5.04E0	-4.27E-1
1984	2.30E0	2.58E0
1985	2.38E0	1.31E0
1986	2.92E0	2.97E0
1987	4.90E0	4.90E0
1988	3.90E0	3.20E0
1989	4.70E0	2.90E0
1990	6.40E0	0.00E0
1991	5.00E0	0.00E0
1992	6.60E0	0.00E0
1993	0.00E0	0.00E0
1994	0.00E0	1.80E0
1995	2.30E0	2.00E0
1996	0.00E0	4.10E0
1997	0.00E0	0.00E0

0.00E0 = no detectable measurements

1979 - 1986 mean based on all net activity results

3.5 BROADLEAF VEGETATION

Gamma spectroscopy was performed on 52 broadleaf vegetation samples during 1997. Three indicator locations and one control were sampled. Cs-137 was reported in indicator and control location samples. No other effluent related radionuclide was identified.

Cs-137 was detected more often and in higher concentrations for control Location 073 samples. The majority of control location samples contained Cs-137 with an annual mean of 135 pCi/kg (6.8% of reporting level). The highest concentration identified was 307 pCi/kg. Three of the thirty-nine indicator location samples contained Cs-137. The highest concentration was 47.3 pCi/kg (2.37% of the reporting level). These results are similar to those reported in 1996.

Cs-137 is the only radionuclide, other than naturally occurring, reported in indicator location vegetation samples since the change in gamma spectroscopy analysis systems in 1987.

It is not unusual for Cs-137 to be present in vegetation. It is a constituent of nuclear weapons test fallout and has been observed in samples from indicator and control locations in previous years. Table 3.5 lists the highest indicator location annual mean and control location annual mean for Cs-137 since early in the station's operational history. Visual inspection of the tabular data did not reveal any increasing trends. There is no indication that the Cs-137 is due to ONS operations based on the low concentration observed and the absence of other radionuclides.

K-40 and Be-7 are the naturally occurring radionuclides that were observed in broadleaf vegetation samples.

Table 3.5 Mean Concentration of Radionuclides in Vegetation

Year	Cs-137 Indicator (pCi/kg)	Cs-137 Control (pCi/kg)
July 1974 - Dec. 1974	1.54E3	0.00E0
Jan. 1975 - June 1975	1.55E3	1.59E3
July 1975 - Dec. 1975	0.00E0	0.00E0
1976	0.00E0	0.00E0
1977	0.00E0	7.90E2
1978	1.19E2	8.19E1
1979	5.04E1	2.96E1
1980	2.80E1	1.55E1
1981	2.99E1	2.60E1
1982	2.42E1	2.62E1
1983	7.44E0	5.35E-1
1984	1.37E1	4.74E2
1985	1.62E1	2.20E2
1986	3.28E1	3.12E2
1987	2.70E1	4.20E1
1988	2.40E1	7.50E1
1989	0.00E0	1.08E2
1990	2.73E2	1.74E2
1991	2.20E1	1.45E2
1992	0.00E0	1.46E2
1993	0.00E0	1.49E2
1994	0.00E0	1.06E2
1995	4.30E1	1.58E2
1996	3.79E1	1.83E2
1997	4.73E1	1.35E2

Only qualitative results reported prior to 1974

Control location changed to 073 in 1984

1979 - 1986 mean based on all net activity results

3.6 FISH

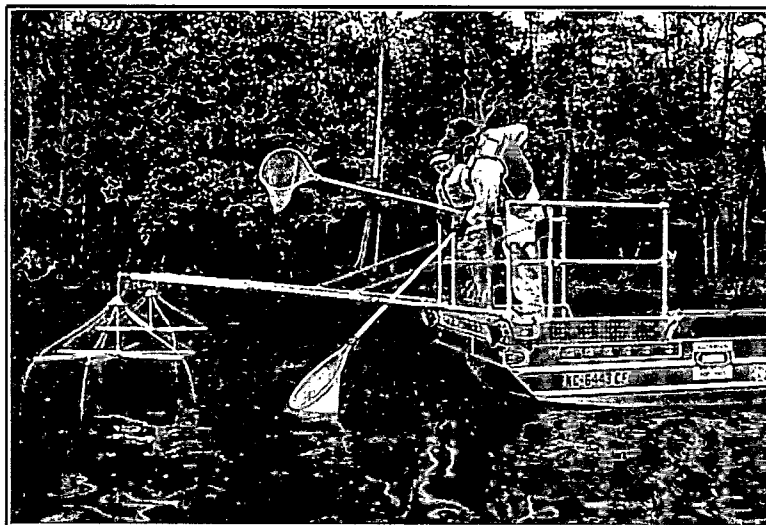
Gamma spectroscopy was performed on 12 fish samples. Two downstream indicator and one control location were sampled. Cs-137 was identified in all the indicator location samples and half of the control location samples. No other effluent related radionuclide was identified. The highest average concentration for Cs-137 was 118 pCi/kg (5.9% of reporting level). The highest individual sample concentration for Cs-137 was 219 pCi/kg (11% of reporting level). The control Cs-137 average concentration was 42.1 pCi/kg. 1996 Cs-137 sample results for all locations were similar. Cs-134 was also detected in 1996 fish samples.

Figures 3.6-1 and 3.6-2 are graphs displaying the annual means for Cs-137 and Cs-134. Both are major contributors to the calculated dose from ingestion of fish. Radioactivity concentrations in downstream fish samples are higher than those reported in preoperational fish samples, however, fluctuations in the graphed results are large and no trends are apparent. Based on these graphs, the levels at the two downstream locations do not appear to be increasing.

One factor affecting the trend analysis is a change in sampling locations. In 1984, a second downstream fish location was added. Location 063 is closer to the liquid effluent discharge point and has been the highest mean indicator since it was added.

K-40 was observed in fish samples in addition to the radionuclides discussed above.

Table 3.6 lists the highest indicator location annual means since the preoperational period for radionuclides detected in 1997. Also included in the table are radionuclides that have been identified in this media since the change in analysis systems in 1987. Comparison of data to previous years does not indicate any increases in concentrations.



Fish Sampling

Figure 3.6-1

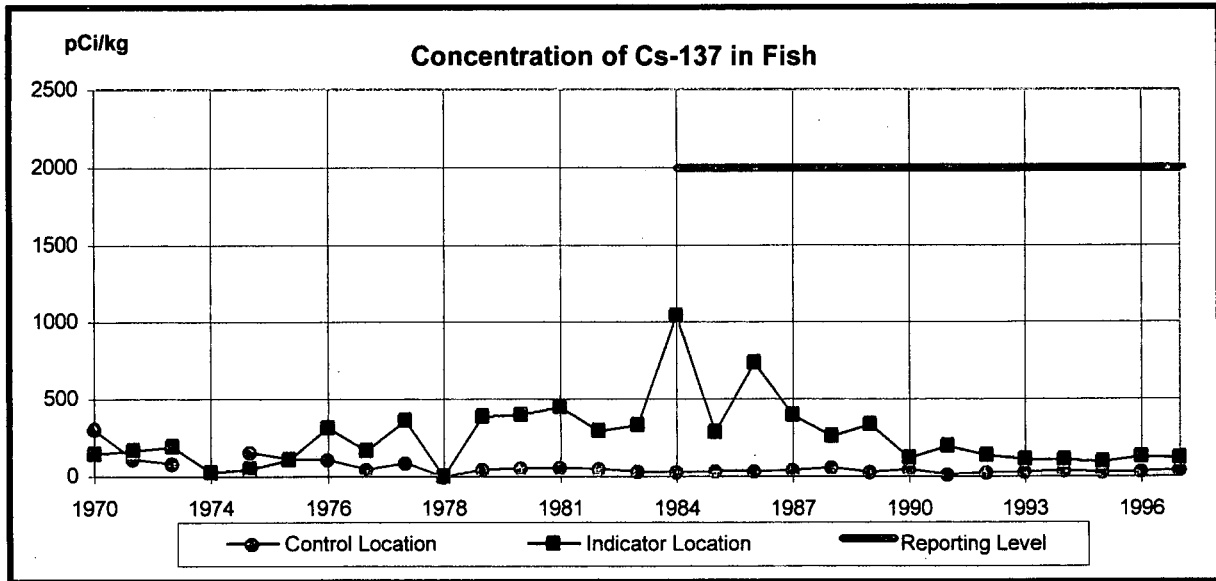
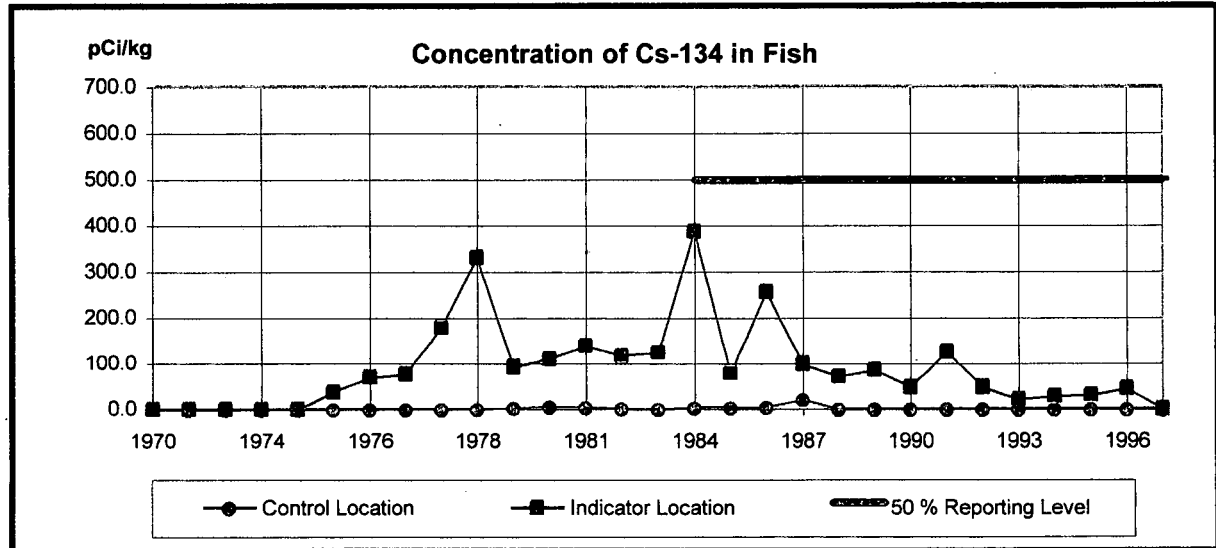


Figure 3.6-2



Current reporting levels implemented 1984

Table 3.6 Mean Concentrations of Radionuclides in Fish

Year	Co-58 (pCi/kg)	Co-60 (pCi/kg)	Cs-134 (pCi/kg)	Cs-137 (pCi/kg)
Preop ending Jan. 1971	0.00E0	0.00E0	0.00E0	1.46E2
Preop ending Jan. 1973	0.00E0	0.00E0	0.00E0	1.66E2
Feb. 1973 - June 1973	Qualitative results reported-no significant measurements above background			
July 1973 - Dec. 1973	0.00E0	0.00E0	0.00E0	1.89E2
Jan. 1974 - June 1974	0.00E0	0.00E0	0.00E0	2.47E1
July 1974 - Dec. 1974	0.00E0	0.00E0	0.00E0	4.85E1
Jan. 1975 - June 1975	0.00E0	0.00E0	3.81E1	1.05E2
July 1975 - Dec. 1975	8.50E1	0.00E0	7.00E1	3.13E2
1976	5.70E1	1.14E2	7.73E1	1.66E2
1977	0.00E0	0.00E0	1.80E2	3.60E2
1978	3.27E2	0.00E0	3.31E2	0.00E0
1979	1.91E0	1.56E1	9.26E1	3.88E2
1980	1.45E1	1.90E1	1.10E2	3.99E2
1981	2.25E1	1.49E1	1.40E2	4.51E2
1982	9.83E-1	8.03E0	1.17E2	2.94E2
1983	3.35E1	4.53E0	1.24E2	3.32E2
1984	1.21E2	6.23E1	3.87E2	1.04E3
1985	1.62E1	1.10E1	7.93E1	2.85E2
1986	9.56E1	2.59E1	2.57E2	7.36E2
1987	1.63E2	6.30E1	9.80E1	3.93E2
1988	9.60E1	0.00E0	7.20E1	2.60E2
1989	4.30E1	1.50E1	8.60E1	3.36E2
1990	1.50E1	0.00E0	4.80E1	1.19E2
1991	4.59E1	0.00E0	1.25E2	1.94E2
1992	6.10E1	0.00E0	4.80E1	1.36E2
1993	0.00E0	0.00E0	2.10E1	1.10E2
1994	0.00E0	0.00E0	2.80E1	1.05E2
1995	0.00E0	0.00E0	3.10E1	9.20E1
1996	0.00E0	0.00E0	4.49E1	1.25E2
1997	0.00E0	0.00E0	0.00E0	1.18E2

0.00E0 = no detectable measurements

1979 - 1986 mean based on all net activity results

3.7 SHORELINE SEDIMENT

Gamma spectroscopy was performed on six sediment samples. Two downstream indicator locations and one control location were sampled.

Cs-137 was identified in both indicator and control location samples. The highest indicator location annual mean was 106 pCi/kg. The 1997 control location mean was 38.3 pCi/kg. No other effluent related radionuclide was identified. 1996 Cs-137 sample results for all locations were similar. Co-58 and Cs-134 were also detected in 1996 shoreline sediment samples. Table 3.7 lists the highest indicator location annual means since shoreline sediment was initiated in 1984. Included in the table are radionuclides that have been identified in this media since the change in analysis systems in 1987.

Visual inspection of the tabular data did not reveal any trends. Figure 3.7-1 is a graph of the Cs-137 annual means. Figure 3.7-2 is a graph of the Co-60 annual means. Both are major contributors to the calculated dose from shoreline sediment. Fluctuations in the graphed results are large and no trends are apparent.

Previous environmental reports (reference 6.5) have addressed the fluctuations in shoreline sediment sample results. Some of these are attributed to differences in the actual point of sampling due to periods of drought. Samples are collected at the edge of the water. Reduced lake levels caused some samples to be taken at points that are normally submerged and where sediment deposition is expected to be greater.

The 1997 doses from shoreline sediments were low and well within all dose limits.

K-40 and Be-7 are the naturally occurring radionuclides observed in shoreline sediment samples.

Figure 3.7-1

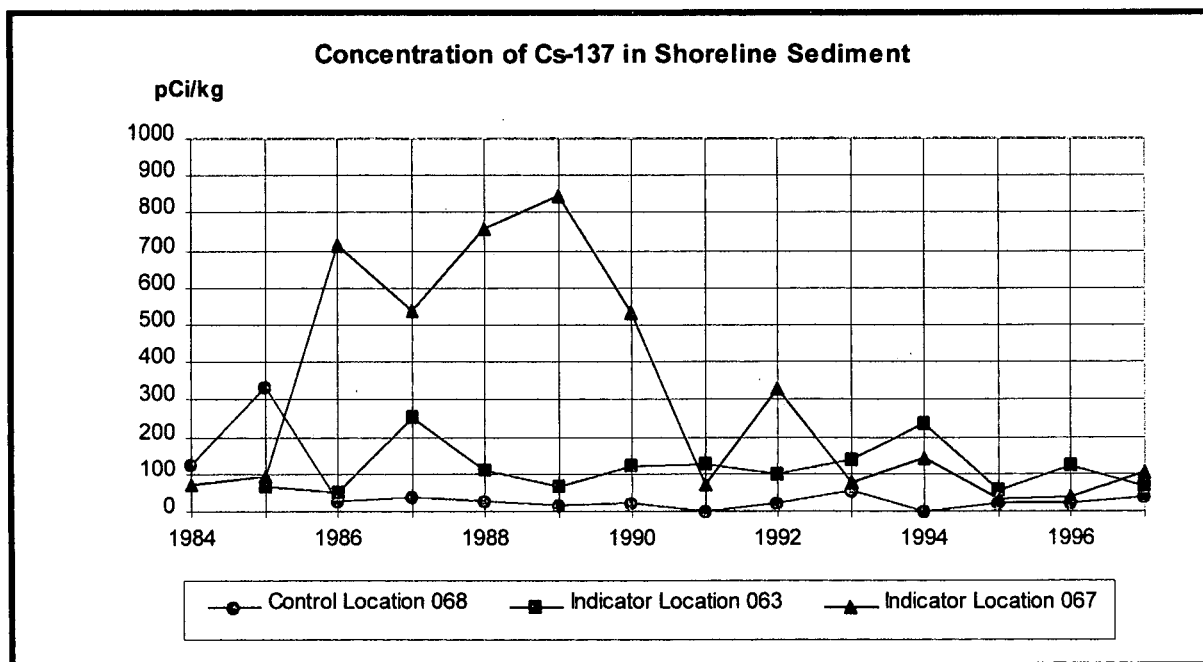
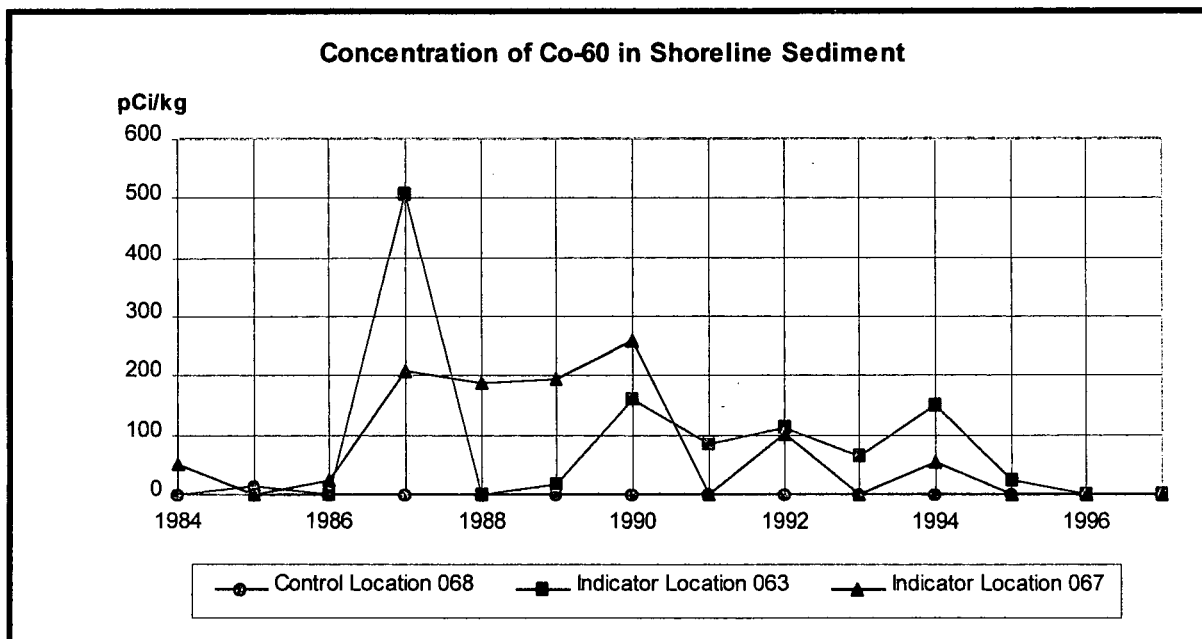


Figure 3.7-2



There are no reporting levels for shoreline sediment

Table 3.7 Mean Concentrations of Radionuclides in Shoreline Sediment (pCi/kg)

Year	Mn-54	Co-58	Co-60	Zn-65	Cs-134	Cs-137	Ag-110m	Sb-125
1984	1.10E1	1.09E1	1.19E1	0.00E0	7.77E1	5.16E1	0.00E0	0.00E0
1985	9.39E0	1.27E0	4.79E0	0.00E0	7.63E1	9.47E1	0.00E0	0.00E0
1986	2.24E1	1.62E1	2.50E1	0.00E0	1.41E2	7.12E2	0.00E0	0.00E0
1987	5.40E1	4.70E2	5.07E2	0.00E0	1.01E2	6.22E2	3.46E2	0.00E0
1988	3.30E1	1.20E2	1.87E2	6.70E1	6.60E1	7.59E2	1.62E2	3.67E2
1989	2.30E1	1.24E2	1.96E2	0.00E0	5.40E1	8.48E2	5.50E1	1.86E2
1990	3.40E1	8.00E1	2.59E2	0.00E0	4.50E1	5.36E2	1.71E2	9.00E1
1991	3.26E1	5.60E1	8.57E1	0.00E0	6.91E1	1.24E2	1.10E2	1.78E2
1992	8.79E1	1.79E2	1.12E2	0.00E0	5.60E1	3.31E2	1.69E2	2.08E2
1993	8.20E1	8.20E1	6.50E1	0.00E0	3.20E1	1.36E2	5.63E1	1.11E2
1994	5.30E1	7.00E1	1.49E2	0.00E0	6.70E1	2.38E2	1.04E2	1.29E2
1995	1.43E2	3.90E1	2.40E1	0.00E0	1.10E1	5.20E1	0.00E0	0.00E0
1996	0.00E0	5.10E1	0.00E0	0.00E0	1.98E1	1.19E2	0.00E0	0.00E0
1997	0.00E0	0.00E0	0.00E0	0.00E0	0.00E0	1.06E2	0.00E0	0.00E0

0.00E0 = no detectable measurements

1984-1986 mean based on all net activity results

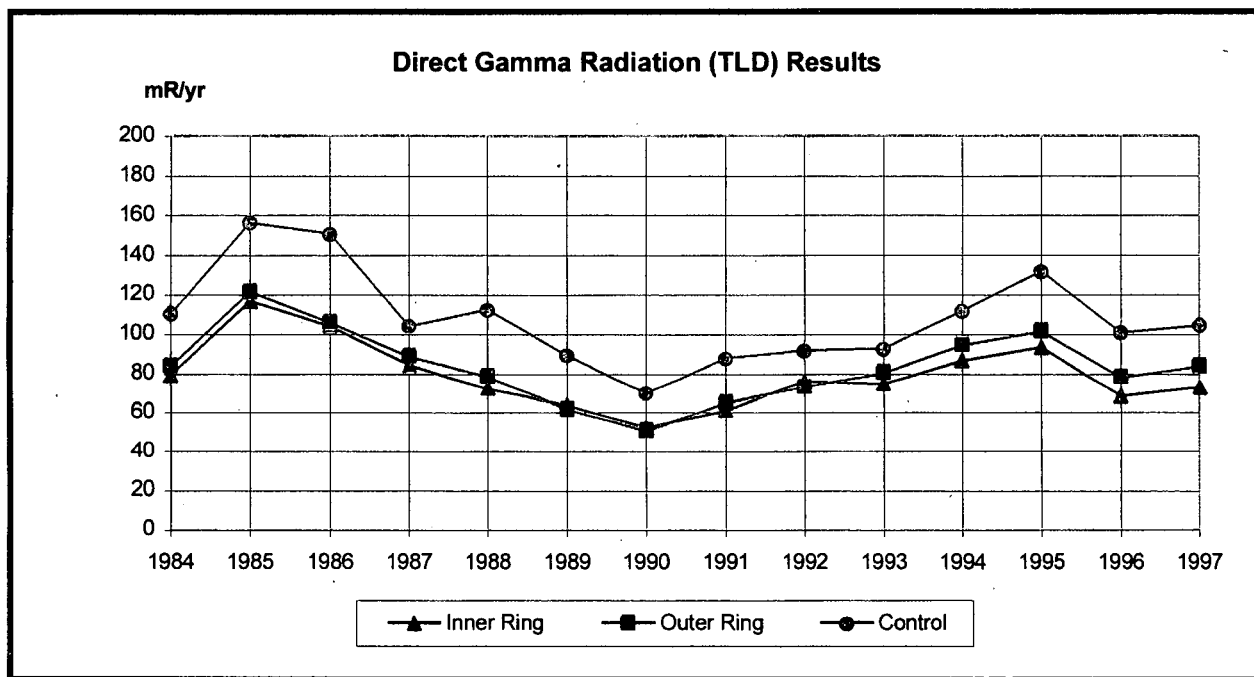
3.8 DIRECT GAMMA RADIATION

Thermoluminescent Dosimeter (TLD) measurements for direct gamma radiation were made each quarter at forty-one locations. TLDs were recovered from all locations for all four quarters in 1997. The highest annual mean exposure for an indicator location was 104.8 milliroentgen. This TLD is located at indicator location 059, 9.2 miles from the station. The annual mean exposure for the control location was 104.5 milliroentgen.

Figure 3.8 and Table 3.8 show TLD inner ring (site boundary), outer ring (4-5 miles), and control location annual averages in milliroentgen per year. Data is provided from 1984 when TLD locations were added and arranged in an inner ring and outer ring configuration. Preoperational data is also provided in the table. As shown in the graph, inner and outer ring averages historically compare closely, with control data somewhat higher. Inner and outer ring averages comprise a number of data points with control averages representing only one location.

In addition, the calculated total body dose (from gaseous effluents) for 1997 was $1.37\text{E-}03$ mrem. This is $<0.002\%$ of the average inner ring TLD values. It can be concluded that discharges from ONS had very little impact upon the measured TLD values.

Figure 3.8



There is no reporting level for Direct Radiation (TLD)

Table 3.8 Direct Gamma Radiation (TLD) Results

Year	Inner Ring Average (mR/yr)	Outer Ring Average (mR/yr)	Control (mR/yr)
Preoperational	113.1	123.9	148.9
1984	79.4	83.8	110.3
1985	116.9	121.5	156.6
1986	104.2	106.0	150.9
1987	84.3	88.8	104.3
1988	72.3	78.6	112.6
1989	63.7	61.7	89.4
1990	52.2	50.7	70.1
1991	61.2	65.0	88.0
1992	76.2	73.2	92.0
1993	74.8	80.6	93.0
1994	86.8	94.7	112.0
1995	93.6	101.7	132.0
1996	68.5	78.3	101.0
Average (1984 - 1996)	79.6	83.4	108.6
1997	72.8	83.8	104.5

3.9 LAND USE CENSUS

The Land Use Census was conducted during the growing season (8/19-8/20/97). The census results are contained in Table 3.9. No program changes were required based on the results of the census.

Table 3.9
Oconee 1997 Land Use Census Results

Sector		Distance (Miles)	Sector		Distance (Miles)
N	Nearest Residence	2.98	S	Nearest Residence	1.85
	Nearest Milk Cow	-		Nearest Milk Cow	-
	Nearest Beef Cow	-		Nearest Beef Cow	-
	Nearest Goat	-		Nearest Goat	-
NNE	Nearest Residence	2.39	SSW	Nearest Residence	1.33
	Nearest Milk Cow	-		Nearest Milk Cow	-
	Nearest Beef Cow	3.00		Nearest Beef Cow	-
	Nearest Goat	-		Nearest Goat	-
NE	Nearest Residence	1.44	SW	Nearest Residence	1.30
	Nearest Milk Cow	-		Nearest Milk Cow	-
	Nearest Beef Cow	3.00		Nearest Beef Cow	1.80
	Nearest Goat	-		Nearest Goat	-
ENE	Nearest Residence	1.25	WSW	Nearest Residence	1.79
	Nearest Milk Cow	-		Nearest Milk Cow	-
	Nearest Beef Cow	2.50		Nearest Beef Cow	-
	Nearest Goat	-		Nearest Goat	-
E	Nearest Residence	1.16	W	Nearest Residence	2.31
	Nearest Milk Cow	-		Nearest Milk Cow	-
	Nearest Beef Cow	2.40		Nearest Beef Cow	4.80
	Nearest Goat	-		Nearest Goat	-
ESE	Nearest Residence	1.50	WNW	Nearest Residence	1.33
	Nearest Milk Cow	-		Nearest Milk Cow	4.50
	Nearest Beef Cow	2.10		Nearest Beef Cow	2.00
	Nearest Goat	-		Nearest Goat	-
SE	Nearest Residence	1.45	NW	Nearest Residence	1.00
	Nearest Milk Cow	-		Nearest Milk Cow	-
	Nearest Beef Cow	4.40		Nearest Beef Cow	-
	Nearest Goat	-		Nearest Goat	-
SSE	Nearest Residence	1.55	NNW	Nearest Residence	1.56
	Nearest Milk Cow	-		Nearest Milk Cow	-
	Nearest Beef Cow	4.60		Nearest Beef Cow	4.50
	Nearest Goat	-		Nearest Goat	-

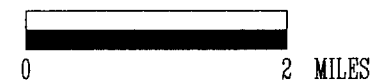
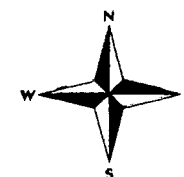
“-“ indicates no occurrences within the 5 mile radius

Oconee Nuclear Station 1997 Land Use Census Map

- Milk cow
- Beef cow
- ★ Residence

All above locations shown
within sectors are approximate.

- Streams and
Shorelines
- Roads
- - - County lines
- + + + Railroads

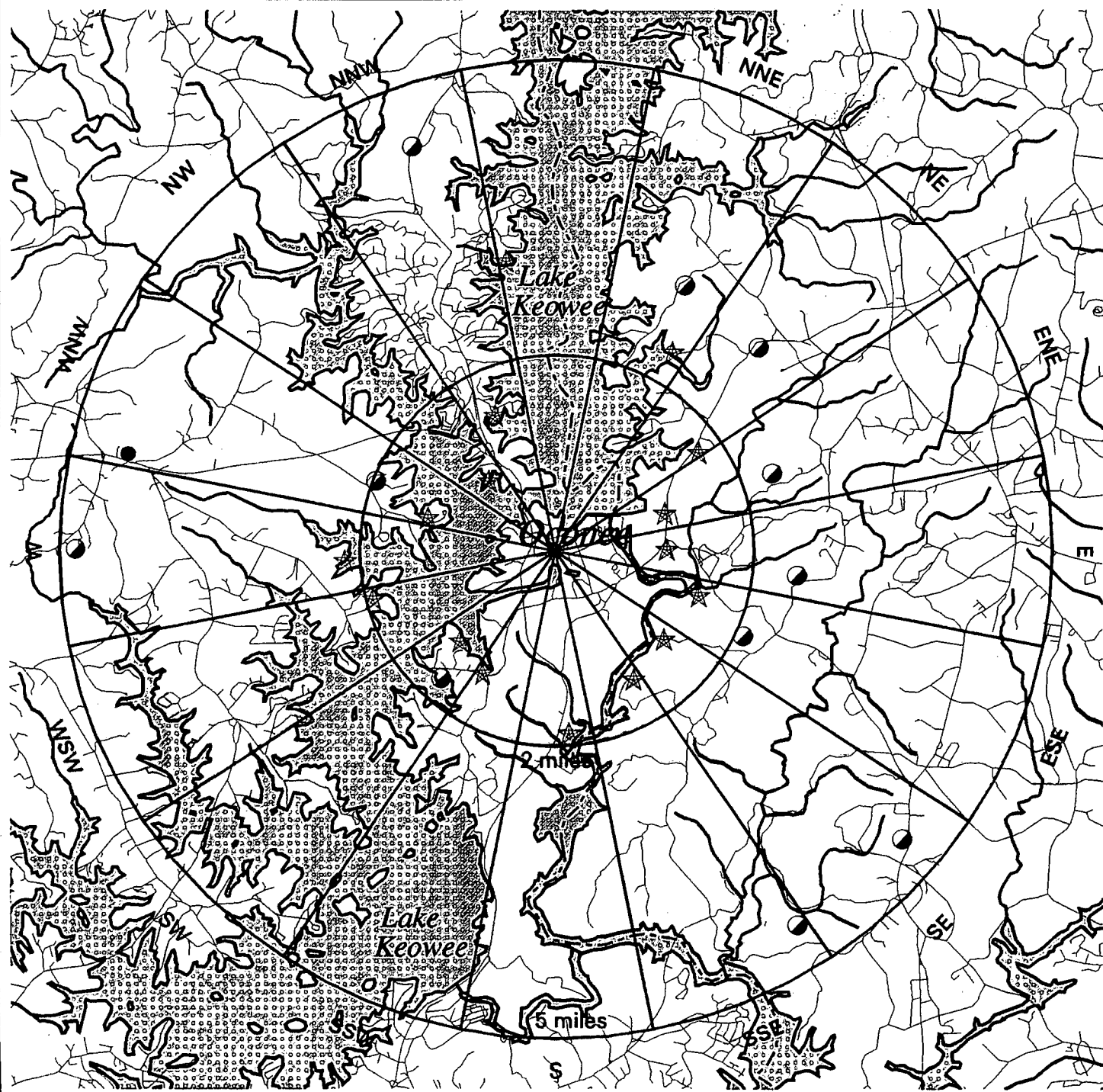


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Projection N.C. State Plane

August 1997

bka 03/17/98



4.0 EVALUATION OF DOSE

4.1 DOSE FROM ENVIRONMENTAL MEASUREMENTS

Annual doses to maximum exposed individuals were estimated based on measured concentrations of radionuclides in 1997 ONS REMP samples. The primary purpose of estimating doses based on sample results was to allow comparison to effluent program dose estimates. Doses based on sample results were conservatively calculated in a manner as equivalent as possible to effluent-based dose estimates.

Doses based on REMP sample results were calculated using the methodology and data presented in NRC Regulatory Guide 1.109. Measured radionuclide concentrations, averaged over the entire year for a specific radionuclide, indicator location, and sample type, were used to calculate REMP-based doses, after subtracting the applicable average background concentration (as measured at the corresponding control location). Regulatory Guide 1.109 consumption rates for the maximum exposed individual were used in the calculations. A dose factor of zero was assumed when the guide listed "NO DATA" as the dose factor for a given radionuclide and organ.

Maximum dose estimates calculated using airborne radioiodine and particulates, drinking water, milk, broadleaf vegetation, fish and shoreline sediment results are reported in Table 4.1-A. The individual critical population and pathway dose calculations are contained in Table 4.1-B.

REMP-based dose estimates of zero were reported for gaseous release pathway sample types. No radionuclides were detected in milk, airborne radioiodine or airborne particulate samples other than naturally-occurring K-40 and Be-7. Similarly, no effluent related radionuclides were detected in broadleaf vegetation samples in excess of that detected in control location samples. Dose estimates were not calculated for surface water samples because surface water is not considered a potable drinking water source. Exposure based on REMP TLD results is discussed in Section 3.8.

Maximum 1997 REMP-based dose estimates for drinking water, fish and shoreline sediment sample results were summed to determine the maximum total dose for the liquid effluent release pathways. The dose contribution from shoreline sediment to each organ, other than the skin, was assumed to equal the total body contribution from shoreline sediment. The maximum total organ dose estimates for the critical age groups have been reported in Table 4.1-A.

4.2 ESTIMATED DOSE FROM RELEASES

Throughout the year, dose estimates were calculated based on actual 1997 liquid and gaseous effluent release data. Effluent-based dose estimates were calculated using the LADTAP and GASPAR computer programs which employ methodology and data presented in NRC Regulatory Guide 1.109. The 1997 ONS Annual Radioactive Effluent Release Report (reference 6.6) included calendar year dose estimates for the maximum exposed individual from liquid and gaseous effluent releases. These reported doses are shown in Table 4.1-A along with the corresponding REMP-based dose estimates.

The effluent-based liquid release doses are summations of the dose contributions from the drinking water, fish and shoreline sediment pathways. The effluent-based gaseous release doses report noble gas exposure separately from iodine, particulate, and tritium exposure. For noble gas exposure there is no critical age group; as the maximum exposed individuals are assumed to receive the same doses, regardless of their age group. For iodine, particulate, and tritium exposure the 1997 ONS Annual Radioactive Effluent Release Report lists the maximum total organ dose for the highest dose location, but only for the maximum organ for the critical age group. The child's liver was the maximum organ in 1997. Effluent-based dose estimates for organs other than the liver are not reported in the iodine, particulate, and tritium exposure summary.

4.3 COMPARISON OF DOSES

The environmental and release data doses given in Table 4.1-A agree reasonably well. The similarity of the doses indicate that the radioactivity levels in the environment do not differ significantly from those expected based on effluent measurements and modeling of the environmental exposure pathways. This indicates that effluent program dose estimates are both valid and reasonably conservative.

In calculations based on liquid release effluent pathways, fish consumption was the predominant dose path based on environmental and effluent samples. The maximum total organ dose based on 1997 environmental sample results was $2.00\text{E-}1$ mrem to the teen's liver. The fish pathway accounted for 94.5% of the total liver dose. Similarly, the maximum total organ dose of $9.61\text{E-}1$ mrem for liquid effluent-based estimates was also to the teen's liver due to fish consumption. The radionuclide contributing to the majority of both estimates was Cs-137.

For all organs but the thyroid, liquid effluent doses were higher than REMP data doses. The effluent thyroid dose was 13.9% of the REMP-based estimate. The major contributors to the effluent thyroid dose were H-3, Co-60, Sb-125, Ag-110m, and Cs-137. Shoreline sediment exposure was the critical pathway. Only Cs-137 was

detected in any actual sediment samples. Drinking water was the critical pathway for the thyroid REMP-based estimate with Tritium as the contributor.

A zero dose value is given for gaseous release pathway doses based on environmental results due to the absence of activity in the pathway samples that could be attributed to station operations. Vegetation ingestion is the critical path for the effluent-based doses, with the child being the critical age. A maximum liver dose of $2.69\text{E-}02$ mrem was calculated. The major contributors to the effluent vegetation doses are Cs-137 (9.76% of the dose) and Tritium (80.99% of the dose). No Cs-137 was detected in broadleaf vegetation samples in excess of control location sample concentrations. Tritium analysis is not performed on gaseous release pathway samples. Noble gas samples are not collected as part of the REMP, preventing an analogous comparison of effluent-based noble gas exposure estimates.

Doses from all sampled paths were summed. The doses calculated do not exceed the 40CFR190 dose commitment limits for members of the public. Doses to members of the public attributable to the operation of ONS are being maintained well within regulatory guidelines.

TABLE 4.1-A

Page 1 of 2

**OCONEE NUCLEAR STATION
1997 ENVIRONMENTAL AND EFFLUENT DOSE COMPARISON****LIQUID RELEASE PATHWAY**

Organ	Environmental or Effluent Data	Critical Age	Critical Pathway	Maximum Dose* (mrem)
Skin	Environmental	Teen	Shoreline Sediment	1.78E-04
Skin	Effluent	Teen	Shoreline Sediment	2.92E-03
Bone	Environmental	Child	Fish	1.71E-01
Bone	Effluent	Child	Fish	8.80E-01
Liver	Environmental	Teen	Fish	2.00E-01
Liver	Effluent	Teen	Fish	9.61E-01
T. Body	Environmental	Adult	Fish	1.40E-01
T. Body	Effluent	Adult	Fish	6.15E-01
Thyroid	Environmental	Child	Drinking Water	2.71E-02
Thyroid	Effluent	Teen	Shoreline Sediment	3.78E-03
Kidney	Environmental	Adult	Fish	8.48E-02
Kidney	Effluent	Teen	Fish	3.28E-01
Lung	Environmental	Child	Fish	4.62E-02
Lung	Effluent	Teen	Fish	1.30E-01
GI-LLI	Environmental	Adult	Drinking Water	2.92E-02
GI-LLI	Effluent	Adult	Fish	4.74E-02

* Maximum dose is a summation of the fish, drinking water and shoreline sediment pathways.

GASEOUS RELEASE PATHWAY

NOBLE GAS EXPOSURE

Organ	Environmental or Effluent Data	Critical Age	Critical Pathway	Maximum Dose (mrem)
Skin	Environmental	-	-	Not Sampled
Skin	Effluent	N/A	Noble Gas	3.12E-03
T. Body	Environmental	-	-	Not Sampled
T. Body	Effluent	N/A	Noble Gas	1.37E-03

IODINE, PARTICULATE, and TRITIUM

Organ	Environmental or Effluent Data	Critical Age	Critical Pathway	Maximum Dose* (mrem)
Bone	Environmental	-	-	0.00E+00
Liver	Environmental	-	-	0.00E+00
Liver	Effluent	Child	Vegetation	2.69E-02
T. Body	Environmental	-	-	0.00E+00
Thyroid	Environmental	-	-	0.00E+00
Kidney	Environmental	-	-	0.00E+00
Lung	Environmental	-	-	0.00E+00
GI-LLI	Environmental	-	-	0.00E+00

* Maximum dose is a summation of the inhalation, milk and vegetation pathways.

TABLE 4.1-B

Maximum Individual Dose for 1997 based on Environmental Measurements (mrem) for Oconee Nuclear Station

Age	Sample Medium	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Skin
Infant	Airborne	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Drinking Water	0.00E+00	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02	0.00E+00
	Milk	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	<u>TOTAL</u>	0.00E+00	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02	0.00E+00
Child	Airborne	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Drinking Water	0.00E+00	2.01E-02	2.01E-02	2.01E-02	2.01E-02	2.01E-02	2.01E-02	0.00E+00
	Milk	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Broadleaf Vegetation	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Fish	1.71E-01	1.71E-01	3.11E-02	6.93E-03	6.03E-02	2.61E-02	7.96E-03	0.00E+00
	Shoreline Sediment	3.18E-05	3.18E-05	3.18E-05	3.18E-05	3.18E-05	3.18E-05	3.18E-05	3.72E-05
	<u>TOTAL</u>	1.71E-01	1.91E-01	5.12E-02	2.71E-02	8.04E-02	4.62E-02	2.81E-02	3.72E-05
Teen	Airborne	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Drinking Water	0.00E+00	1.05E-02	1.05E-02	1.05E-02	1.05E-02	1.05E-02	1.05E-02	0.00E+00
	Milk	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Broadleaf Vegetation	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Fish	1.36E-01	1.89E-01	7.14E-02	8.39E-03	7.00E-02	3.23E-02	1.10E-02	0.00E+00
	Shoreline Sediment	1.52E-04	1.52E-04	1.52E-04	1.52E-04	1.52E-04	1.52E-04	1.52E-04	1.78E-04
	<u>TOTAL</u>	1.36E-01	2.00E-01	8.21E-02	1.90E-02	8.07E-02	4.30E-02	2.17E-02	1.78E-04
Adult	Airborne	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Drinking Water	0.00E+00	1.49E-02	1.49E-02	1.49E-02	1.49E-02	1.49E-02	1.49E-02	0.00E+00
	Milk	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Broadleaf Vegetation	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
	Fish	1.27E-01	1.85E-01	1.25E-01	1.09E-02	6.99E-02	3.05E-02	1.43E-02	0.00E+00
	Shoreline Sediment	2.73E-05	2.73E-05	2.73E-05	2.73E-05	2.73E-05	2.73E-05	2.73E-05	3.18E-05
	<u>TOTAL</u>	1.27E-01	2.00E-01	1.40E-01	2.58E-02	8.48E-02	4.54E-02	2.92E-02	3.18E-05

Note: Dose tables are provided for sample media displaying positive nuclide occurrence.

Oconee Nuclear Station
Dose from Drinking Water Pathway for 1997 Data
Maximum Exposed Infant

Infant Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 330 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.99E-05	4.51E-06	NO DATA	4.41E-06	NO DATA	7.31E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	3.60E-06	8.98E-06	NO DATA	NO DATA	NO DATA	8.97E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	3.08E-05	5.38E-05	2.12E-05	NO DATA	NO DATA	1.59E-05	2.57E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	1.08E-05	2.55E-05	NO DATA	NO DATA	NO DATA	2.57E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.84E-05	6.31E-05	2.91E-05	NO DATA	3.06E-05	NO DATA	5.33E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	4.20E-08	1.73E-08	1.00E-08	NO DATA	1.24E-08	NO DATA	1.46E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	2.06E-07	5.02E-08	3.56E-08	NO DATA	5.41E-08	NO DATA	2.50E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	3.59E-05	4.23E-05	1.86E-05	1.39E-02	4.94E-05	NO DATA	1.51E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	3.77E-04	7.03E-04	7.10E-05	NO DATA	1.81E-04	7.42E-05	1.91E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	5.22E-04	6.11E-04	4.33E-05	NO DATA	1.64E-04	6.64E-05	1.91E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	1.71E-04	1.71E-07	8.81E-06	NO DATA	4.06E-08	1.05E-07	4.20E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	066	194.00	0.00E+00	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02
Dose Commitment (mrem) =										0.00E+00	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02	1.97E-02

Oconee Nuclear Station
Dose from Drinking Water Pathway for 1997 Data
Maximum Exposed Child

Child Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 510 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.07E-05	2.85E-06	NO DATA	3.00E-06	NO DATA	8.98E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	1.80E-06	5.51E-06	NO DATA	NO DATA	NO DATA	1.05E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	1.65E-05	2.67E-05	1.33E-05	NO DATA	NO DATA	7.74E-06	2.78E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	5.29E-06	1.56E-05	NO DATA	NO DATA	NO DATA	2.93E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.37E-05	3.65E-05	2.27E-05	NO DATA	2.30E-05	NO DATA	6.41E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	2.25E-08	8.76E-09	6.26E-09	NO DATA	8.23E-09	NO DATA	1.62E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	1.16E-07	2.55E-08	2.27E-08	NO DATA	3.65E-08	NO DATA	2.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	1.72E-05	1.73E-05	9.83E-06	5.72E-03	2.84E-05	NO DATA	1.54E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	2.34E-04	3.84E-04	8.10E-05	NO DATA	1.19E-04	4.27E-05	2.07E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	3.27E-04	3.13E-04	4.62E-05	NO DATA	1.02E-04	3.67E-05	1.96E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	8.31E-05	7.28E-08	4.85E-06	NO DATA	2.37E-08	4.34E-08	4.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	066	194.00	0.00E+00	2.01E-02	2.01E-02	2.01E-02	2.01E-02	2.01E-02	2.01E-02
Dose Commitment (mrem) =										0.00E+00	2.01E-02	2.01E-02	2.01E-02	2.01E-02	2.01E-02	2.01E-02

Oconee Nuclear Station
Dose from Fish Pathway for 1997 Data
Maximum Exposed Child

Child Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg)

H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 5497 pCi/l x 0.9 = 4947 pCi/kg

Usage (intake in one year) = 6.9 kg

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Fish (pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.07E-05	2.85E-06	NO DATA	3.00E-06	NO DATA	8.98E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	1.80E-06	5.51E-06	NO DATA	NO DATA	NO DATA	1.05E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	1.65E-05	2.67E-05	1.33E-05	NO DATA	NO DATA	7.74E-06	2.78E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
C0-60	NO DATA	5.29E-06	1.56E-05	NO DATA	NO DATA	NO DATA	2.93E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.37E-05	3.65E-05	2.27E-05	NO DATA	2.30E-05	NO DATA	6.41E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	2.34E-04	3.84E-04	8.10E-05	NO DATA	1.19E-04	4.27E-05	2.07E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	3.27E-04	3.13E-04	4.62E-05	NO DATA	1.02E-04	3.67E-05	1.96E-06	063	75.90	1.71E-01	1.64E-01	2.42E-02	0.00E+00	5.34E-02	1.92E-02	1.03E-03
H-3	NO DATA	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	063	4947.00	0.00E+00	6.93E-03	6.93E-03	6.93E-03	6.93E-03	6.93E-03	6.93E-03
Dose Commitment (mrem) =										1.71E-01	1.71E-01	3.11E-02	6.93E-03	6.03E-02	2.61E-02	7.96E-03

Oconee Nuclear Station
Dose from Shoreline Sediment Pathway for 1997 Data
Maximum Exposed Child

Shoreline Recreation = 14 hr (in one year)
 Shore Width Factor = 0.2
 Sediment Surface Mass = 40 kg/m²

Child Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External
 Dose Factor (mrem/hr per pCi/m²) x Shore Width Factor x Sediment Surface Mass (kg/m²) x
 Sediment Concentration (pCi/kg)

External Dose Factor Standing on Contaminated Ground			Highest Annual Net Mean Concentration		Dose	
Radionuclide	(mrem/hr per pCi/m ²)		Indicator Location	Sediment (pCi/kg)	(mrem)	
	T. Body	Skin			T. Body	Skin
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00
Cs-137	4.20E-09	4.90E-09	063	67.70	3.18E-05	3.72E-05
Dose Commitment (mrem) =					3.18E-05	3.72E-05

Oconee Nuclear Station
Dose from Drinking Water Pathway for 1997 Data
Maximum Exposed Teen

Teen Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 510 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	5.90E-06	1.17E-06	NO DATA	1.76E-06	NO DATA	1.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	9.72E-07	2.24E-06	NO DATA	NO DATA	NO DATA	1.34E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	5.87E-06	1.37E-05	5.29E-06	NO DATA	NO DATA	4.32E-06	3.24E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.81E-06	6.33E-06	NO DATA	NO DATA	NO DATA	3.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	5.76E-06	2.00E-05	9.33E-06	NO DATA	1.28E-05	NO DATA	8.47E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	8.22E-09	4.56E-09	2.51E-09	NO DATA	4.42E-09	NO DATA	1.95E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	4.12E-08	1.30E-08	8.94E-09	NO DATA	1.91E-08	NO DATA	3.00E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	5.85E-06	8.19E-06	4.40E-06	2.39E-03	1.41E-05	NO DATA	1.62E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	8.37E-05	1.97E-04	9.14E-05	NO DATA	6.26E-05	2.39E-05	2.45E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	1.12E-04	1.49E-04	5.19E-05	NO DATA	5.07E-05	1.97E-05	2.12E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	2.84E-05	3.48E-08	1.83E-06	NO DATA	1.18E-08	2.34E-08	4.38E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	066	194.00	0.00E+00	1.05E-02	1.05E-02	1.05E-02	1.05E-02	1.05E-02	1.05E-02
Dose Commitment (mrem)=										0.00E+00	1.05E-02	1.05E-02	1.05E-02	1.05E-02	1.05E-02	1.05E-02

Oconee Nuclear Station
Dose from Fish Pathway for 1997 Data
Maximum Exposed Teen

Teen Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg)

H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 5497 pCi/l x 0.9 = 4947 pCi/kg

Usage (intake in one year) = 16 kg

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	5.90E-06	1.17E-06	NO DATA	1.76E-06	NO DATA	1.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	9.72E-07	2.24E-06	NO DATA	NO DATA	NO DATA	1.34E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	5.87E-06	1.37E-05	5.29E-06	NO DATA	NO DATA	4.32E-06	3.24E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.81E-06	6.33E-06	NO DATA	NO DATA	NO DATA	3.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	5.76E-06	2.00E-05	9.33E-06	NO DATA	1.28E-05	NO DATA	8.47E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	8.37E-05	1.97E-04	9.14E-05	NO DATA	6.26E-05	2.39E-05	2.45E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	1.12E-04	1.49E-04	5.19E-05	NO DATA	5.07E-05	1.97E-05	2.12E-06	063	75.90	1.36E-01	1.81E-01	6.30E-02	0.00E+00	6.16E-02	2.39E-02	2.57E-03
H-3	NO DATA	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	063	4947.00	0.00E+00	8.39E-03	8.39E-03	8.39E-03	8.39E-03	8.39E-03	8.39E-03
Dose Commitment (mrem) =										1.36E-01	1.89E-01	7.14E-02	8.39E-03	7.00E-02	3.23E-02	1.10E-02

Oconee Nuclear Station
Dose from Shoreline Sediment Pathway for 1997 Data
Maximum Exposed Teen

Shoreline Recreation = 67 hr (in one year)
 Shore Width Factor = 0.2
 Sediment Surface Mass = 40 kg/m²

Teen Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External
 Dose Factor (mrem/hr per pCi/m²) x Shore Width Factor x Sediment Surface Mass (kg/m²) x
 Sediment Concentration (pCi/kg)

External Dose Factor Standing on Contaminated Ground			Highest Annual Net Mean Concentration		Dose	
Radionuclide	(mrem/hr per pCi/m ²)		Indicator Location	Sediment (pCi/kg)	(mrem)	
	T. Body	Skin			T. Body	Skin
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00
Cs-137	4.20E-09	4.90E-09	063	67.70	1.52E-04	1.78E-04
Dose Commitment (mrem) =					1.52E-04	1.78E-04

Oconee Nuclear Station
Dose from Drinking Water Pathway for 1997 Data
Maximum Exposed Adult

Adult Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 730 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator	Water	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
								Location	(pCi/l)							
Mn-54	NO DATA	4.57E-06	8.72E-07	NO DATA	1.36E-06	NO DATA	1.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	7.45E-07	1.67E-06	NO DATA	NO DATA	NO DATA	1.51E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	4.34E-06	1.02E-05	3.91E-06	NO DATA	NO DATA	2.85E-06	3.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.14E-06	4.72E-06	NO DATA	NO DATA	NO DATA	4.02E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	4.84E-06	1.54E-05	6.96E-06	NO DATA	1.03E-05	NO DATA	9.70E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	6.22E-09	3.46E-09	1.86E-09	NO DATA	3.42E-09	NO DATA	2.10E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	3.04E-08	9.75E-09	6.60E-09	NO DATA	1.53E-08	NO DATA	3.09E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	4.16E-06	5.95E-06	3.41E-06	1.95E-03	1.02E-05	NO DATA	1.57E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	6.22E-05	1.48E-04	1.21E-04	NO DATA	4.79E-05	1.59E-05	2.59E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	7.97E-05	1.09E-04	7.14E-05	NO DATA	3.70E-05	1.23E-05	2.11E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	2.03E-05	2.55E-08	1.33E-06	NO DATA	8.67E-09	1.46E-08	4.18E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	066	194.00	0.00E+00	1.49E-02	1.49E-02	1.49E-02	1.49E-02	1.49E-02	1.49E-02
Dose Commitment (mrem) =										0.00E+00	1.49E-02	1.49E-02	1.49E-02	1.49E-02	1.49E-02	1.49E-02

Oconee Nuclear Station
Dose from Fish Pathway for 1997 Data
Maximum Exposed Adult

Adult Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg)

H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 5497 pCi/l x 0.9 = 4947 pCi/kg

Usage (intake in one year) = 21 kg

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	4.57E-06	8.72E-07	NO DATA	1.36E-06	NO DATA	1.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	7.45E-07	1.67E-06	NO DATA	NO DATA	NO DATA	1.51E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	4.34E-06	1.02E-05	3.91E-06	NO DATA	NO DATA	2.85E-06	3.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.14E-06	4.72E-06	NO DATA	NO DATA	NO DATA	4.02E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	4.84E-06	1.54E-05	6.96E-06	NO DATA	1.03E-05	NO DATA	9.70E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	6.22E-05	1.48E-04	1.21E-04	NO DATA	4.79E-05	1.59E-05	2.59E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	7.97E-05	1.09E-04	7.14E-05	NO DATA	3.70E-05	1.23E-05	2.11E-06	063	75.90	1.27E-01	1.74E-01	1.14E-01	0.00E+00	5.90E-02	1.96E-02	3.36E-03
H-3	NO DATA	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	063	4947.00	0.00E+00	1.09E-02	1.09E-02	1.09E-02	1.09E-02	1.09E-02	1.09E-02
Dose Commitment (mrem) =										1.27E-01	1.85E-01	1.25E-01	1.09E-02	6.99E-02	3.05E-02	1.43E-02

Oconee Nuclear Station
Dose from Shoreline Sediment Pathway for 1997 Data
Maximum Exposed Adult

Shoreline Recreation = 12 hr (in one year)
 Shore Width Factor = 0.2
 Sediment Surface Mass = 40 kg/m²

Adult Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External
 Dose Factor (mrem/hr per pCi/m²) x Shore Width Factor x Sediment Surface Mass (kg/m²) x
 Sediment Concentration (pCi/kg)

External Dose Factor Standing on Contaminated Ground			Highest Annual Net Mean Concentration		Dose (mrem)	
Radionuclide	(mrem/hr per pCi/m ²)		Indicator Location	Sediment (pCi/kg)	T. Body	Skin
	T. Body	Skin				
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00
Cs-137	4.20E-09	4.90E-09	063	67.70	2.73E-05	3.18E-05
Dose Commitment (mrem) =					2.73E-05	3.18E-05

5.0 QUALITY ASSURANCE

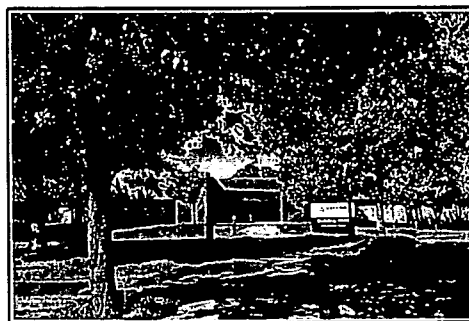
5.1 DUKE POWER COMPANY'S REMP

5.1.1 SAMPLE COLLECTION

The ONS Chemistry Section performed the environmental sample collections as specified by approved sample collection procedures.

5.1.2 SAMPLE ANALYSIS

Radiological and Environmental Services performed the environmental sample analyses as specified by approved analysis procedures. The Radiological and Environmental Services Laboratory is located in Huntersville, North Carolina, at Duke Power Company's Environmental Center.



Duke Power Company's
Environmental Center

5.1.3 DOSIMETRY ANALYSIS

The Radiation Dosimetry and Records group performed environmental dosimetry measurements as specified by approved dosimetry analysis procedures.

5.1.4 INTRALABORATORY QUALITY ASSURANCE

Radiological and Environmental Services has an internal quality assurance program which monitors each type of instrumentation for reliability and accuracy. Daily quality control checks ensure that instruments are in proper working order and these checks are used to monitor instrument performance.

Additionally, National Institute of Standards and Technology (NIST) standards that represent counting geometries are analyzed as unknowns at various frequencies ranging from weekly to annually to verify that efficiency calibrations are valid. The frequency is dependent upon instrument use and performance. Investigations are performed and documented should calibration verification data fall out of limits.

Method spike and blank samples are analyzed with sample analyses that are processed in batches. These include gross beta in drinking water and all tritium analyses.

5.1.5 INTERLABORATORY QUALITY ASSURANCE

5.1.5.1 INTERLABORATORY QUALITY ASSURANCE PROGRAM

The Radiological and Environmental Services Laboratories participated in the Environmental Protection Agency (EPA) cross check program during 1997. Results of these cross-checks are displayed in Table 5.0-A.

5.1.5.2 DUKE POWER'S INTERCOMPARISON PROGRAM

Radiological and Environmental Services participated in the Duke Power Nuclear Generation Department Intercomparison Program during 1997. Interlaboratory cross-check standards, including, marinelli beakers, air filters, air cartridges, gross beta on smears, and tritium in water samples were analyzed at various times of the year by the four counting laboratories in Duke Power Company for this program. A summary of these Intercomparison Reports for 1997 is documented in Table 5.0-B.

5.1.5.3 DUKE POWER'S AUDIT DIVISION

The Oconee Nuclear Station Radiation Protection Section participated in a Quality Assurance audit July 14 through July 24, 1997. This audit was conducted by the Nuclear Assessment and Issues Division, Regulatory Audit Group. One finding pertaining to the Selected Licensee Commitment description of an "NRC-approved cross-check program" was identified. An effective radiological intercomparison program is in place for the Oconee REMP, and the Selected Licensee Commitments have been changed to delete the reference to an NRC approved program. The current program includes an EPA drinking water intercomparison program and the Duke Power intercomparison program. The total EPA intercomparison program is no longer available.

Radiological and Environmental Services participated in a Quality Assurance audit in February, 1997. This audit was conducted by the Nuclear Assessment and Issues Division, Regulatory Audit Group. There were no findings from this audit.

5.1.5.4 U.S. NUCLEAR REGULATORY COMMISSION INSPECTIONS

The Oconee Nuclear Station Radiological Environmental Monitoring Program was audited by the NRC in June 1997. There were no recommendations as a result of this audit and it was noted that an effective radiological environmental monitoring program had been implemented. Radiological and Environmental Services was not audited by the NRC in 1997.

5.1.5.5 NRC/STATE OF S.C. SAMPLING INTERCOMPARISON PROGRAM

Oconee Nuclear Station routinely participates with the Bureau of Radiological Health of the State's Department of Health and Environmental Control (DHEC) in an intercomparison program. Water, milk, vegetation, sediment, and fish samples collected by ONS Chemistry are routinely split with DHEC for intercomparison analysis. DHEC collects air samples near two of the locations sampled for air by ONS. Results of the analyses performed on split and duplicate samples are sent to DHEC for use in their report to the NRC.

5.1.5.6 STATE OF N.C. TLD INTERCOMPARISON PROGRAM

Radiation Dosimetry and Records routinely participates in a TLD intercomparison program. Every six to eight months, the State of North Carolina Radiation Protection Section irradiates environmental dosimeters and sends them to the Radiation Dosimetry and Records group for analysis of the unknown estimated delivered exposure. A summary of the State of North Carolina Environmental Dosimetry Intercomparison Report for 1997 is documented in Table 5.0-C.

5.2 CONTRACTOR LABORATORIES

No contractor laboratories were used during 1997.

TABLE 5.0-A

U.S. ENVIRONMENTAL PROTECTION AGENCY INTERLABORATORY COMPARISON PROGRAM

1997 CROSS-CHECK RESULTS FOR THE RADIOLOGICAL & ENVIRONMENTAL SERVICES LABORATORY

Gamma in Water:

Reference Date	Geometry	Nuclide	Acceptance Range (pCi/liter)	Reference Value (pCi/liter)	Reported Value (pCi/liter)	Cross-Check Status*
2/7/97	3.5 liter Marinelli	I-131	70.4 - 102	86.0	94.2	PASS
4/15/97	3.5 liter Marinelli	Co-60	12.3 - 29.7	21.0	24.6	PASS
		Cs-134	22.3 - 39.7	31.0	31.9	PASS
		Cs-137	13.3 - 30.7	22.0	24.2	PASS
6/6/97	3.5 liter Marinelli	Co-60	9.30 - 26.7	18.0	18.0	PASS
		Zn-65	82.7 - 117	100	100	PASS
		Cs-134	13.3 - 30.7	22.0	17.7	PASS
		Cs-137	40.3 - 57.7	49.0	54.3	PASS
		Ba-133	16.3 - 33.7	25.0	28.3	PASS
10/21/97	3.5 liter Marinelli	Co-60	1.34 - 18.7	10.0	11.0	PASS
		Cs-134	32.3 - 49.7	41.0	39.0	PASS
		Cs-137	25.3 - 42.6	34.0	34.8	PASS
10/31/97	3.5 liter Marinelli	I-131	1.34 - 18.7	10	17.56	PASS
11/7/97	3.5 liter Marinelli	Co-60	18.3 - 35.7	27.0	30.7	PASS
		Zn-65	61.1 - 88.9	75.0	80.0	PASS
		Cs-134	1.30 - 18.7	10.0	11.2	PASS
		Cs-137	65.3 - 82.7	74.0	76.4	PASS
		Ba-133	81.7 - 116	99.0	104	PASS

* Pass = Data within Control Limits, AC = Above Control Limit, BC = Below Control Limit

Beta in Water:

Reference Date	Geometry	Nuclide	Acceptance Range (pCi/liter)	Reference Value (pCi/liter)	Reported Value (pCi/liter)	Cross-Check Status *
1/31/97	2" Planchet	Gross Beta	6.0 - 23.4	14.7	25.0	AC ⁽¹⁾
4/15/97	2" Planchet	Gross Beta	75.6 - 129	102.1	94.9	PASS
7/18/97	2" Planchet	Gross Beta	6.4 - 23.8	15.1	17.0	PASS
10/21/97	2" Planchet	Gross Beta	131 - 156	143.4	148.3	PASS
10/31/97	2" Planchet	Gross Beta	40.2 - 57.6	48.9	60.4	AC ⁽¹⁾

Pass = Data within Control Limits, AC = Above Control Limit, BC = Below Control Limit

(1) Reported value is an average of three analyses, one of which was above the control limit.

Tritium in Water:

Reference Date	Geometry	Nuclide	Acceptance Range (pCi/liter)	Reference Value (pCi/liter)	Reported Value (pCi/liter)	Cross-Check Status *
4/1/97	Plastic 10:10	H-3	6530 - 9271	7900	9835	AC ⁽¹⁾
8/8/97	Plastic 10:10	H-3	9100 - 12920	11010	11047	PASS

Pass = Data within Control Limits, AC = Above Control Limit, BC = Below Control Limit

(1) Reported value is an average of three analyses, one of which was above the control limit

TABLE 5.0-B

DUKE POWER COMPANY INTERLABORATORY COMPARISON PROGRAM

1997 CROSS-CHECK RESULTS FOR THE RADIOLOGICAL & ENVIRONMENTAL SERVICES LABORATORY

Gamma in Charcoal Cartridge:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status*
2/21/97	pCi/total	I-131	1.50E5 - 2.66E5	2.00E5	1.98E5	PASS
6/19/97	pCi/total	I-131	72.8 - 129	97.0	104	PASS
8/22/97	pCi/total	I-131	1.51E5 - 2.67E5	2.01E5	2.04E5	PASS
12/11/97	pCi/total	I-131	23.3 - 41.2	31.0	29.5	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

Gamma in Soil:

Reference Date	Geometry	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status *
2/21/97	0.5 liter Marinelli	pCi/kg	Cr-51	171 - 303	228	273	AC ⁽¹⁾
			Mn-54	41.9 - 74.4	55.9	66.5	PASS
			Co-58	23.3 - 41.2	31.0	33.9	PASS
			Fe-59	49.2 - 87.2	65.6	74.5	PASS
			Co-60	39.8 - 70.6	53.1	52.6	PASS
			Zn-65	40.1 - 71.2	53.5	52.8	PASS
			I-131	37.3 - 66.1	49.7	51.4	PASS
			Cs-134	32.9 - 58.4	43.9	46.4	AC ⁽¹⁾
			Cs-137	25.7 - 45.5	34.2	34.6	PASS
			Ce-141	92.3 - 164	123	135	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

(1) Reported value is an average of three analyses, one of which was above the control limit.

Gamma in Filter:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status *
2/21/97	pCi/total	Cr-51	1.13E5 - 2.01E5	1.51E5	1.55E5	PASS
		Mn-54	2.78E4 - 4.93E4	3.71E4	4.07E4	PASS
		Co-58	1.54E4 - 2.73E4	2.05E4	2.12E4	PASS
		Fe-59	3.26E4 - 5.77E4	4.34E4	5.09E4	PASS
		Co-60	2.65E4 - 4.69E4	3.53E4	3.70E4	PASS
		Zn-65	2.66E4 - 4.71E4	3.54E4	4.28E4	PASS
		Cs-134	2.18E4 - 3.87E4	2.91E4	2.40E4	PASS
		Cs-137	1.7E4 - 3.01E4	2.26E4	2.26E4	PASS
		Ce-141	6.14E4 - 1.08E5	8.18E4	8.72E4	PASS
3/20/97	pCi/total	Cr-51	216 - 383	288	289	PASS
		Mn-54	98.3 - 174	131	146	PASS
		Co-58	44.3 - 78.5	59.0	60.1	PASS
		Fe-59	80.3 - 142	107	129	PASS
		Co-60	98.3 - 174	131	135	PASS
		Zn-65	92.3 - 164	123	135	PASS
		Cs-134	79.5 - 140	106	93.5	PASS
		Cs-137	63.0 - 112	84.0	87.5	PASS
		Ce-141	129 - 229	172	177	PASS
6/19/97	pCi/total	Cr-51	162 - 288	216	228	PASS
		Mn-54	69.0 - 122	92.0	93.9	PASS
		Co-58	63.0 - 112	84.0	87.7	PASS
		Fe-59	62.0 - 109	82.0	87.0	PASS
		Co-60	84.8 - 150	113	114	PASS
		Zn-65	114 - 202	152	179	PASS
		Cs-134	66.8 - 118	89.0	74.5	PASS
		Cs-137	93.8 - 166	125	130	PASS
		Ce-141	108 - 192	144	136	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

Gamma in Filter continued:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status *
9/18/97	pCi/total	Cr-51	183 - 325	244	307	AC ⁽¹⁾
		Mn-54	53.3 - 94.4	71.0	86.2	PASS
		Co-58	36.8 - 65.2	49.0	55.4	PASS
		Fe-59	72.0 - 128	96.0	120	PASS
		Co-60	119 - 211	159	175	PASS
		Zn-65	119 - 210	158	212	AC ⁽¹⁾
		Cs-134	61.5 - 109	82.0	70.4	PASS
		Cs-137	64.5 - 114	86.0	81.9	PASS
		Ce-141	46.5 - 82.5	62.0	70.6	PASS
12/11/97	pCi/total	Cr-51	116 - 205	154	179	PASS
		Mn-54	48.0 - 85.1	64.0	77.2	PASS
		Co-58	30.8 - 54.5	41.0	46.3	PASS
		Fe-59	33.8 - 59.9	45.0	56.9	PASS
		Co-60	56.3 - 99.8	75.0	81.9	PASS
		Zn-65	84.0 - 149	112	131	PASS
		Cs-134	59.3 - 105	79.0	68.3	PASS
		Cs-137	61.5 - 109	82.0	87.1	PASS
		Ce-141	57.8 - 102	77.0	86.9	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

(1) Reported value is an average of three analyses, one of which was above the control limit.

Gamma in Water:

Reference Date	Geometry	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status *
2/21/97	3.5 liter Marinelli	pCi/liter	Cr-51	257 - 455	342	449	AC ⁽²⁾
			Mn-54	62.9 - 112	83.9	99.9	PASS
			Co-58	34.9 - 61.8	46.5	50.2	PASS
			Fe-59	73.8 - 131	98.4	109	PASS
			Co-60	59.8 - 106	79.7	83.7	PASS
			Zn-65	60.2 - 107	80.3	102	PASS
			I-131	55.9 - 99.1	74.5	80.8	PASS
			Cs-134	49.4 - 87.5	65.8	71.5	PASS
			Cs-137	38.5 - 68.2	51.3	54.4	PASS
			Ce-141	139 - 246	185	203	PASS
2/21/97	3.5 liter Marinelli	pCi/liter	Cr-51	5.74E4 - 1.01E5	7.61E4	7.67E4	PASS
			Mn-54	1.38E4 - 2.44E4	1.87E4	1.95E4	PASS
			Co-58	7.80E3 - 1.38E4	1.04E4	1.06E4	PASS
			Fe-59	1.64E4 - 2.91E4	2.19E4	2.28E4	PASS
			Co-60	1.34E4 - 2.36E4	1.78E4	1.81E4	PASS
			Zn-65	1.34E4 - 2.38E4	1.79E4	1.92E4	PASS
			I-131	1.25E4 - 2.21E4	1.66E4	1.68E4	PASS
			Cs-134	1.10E4 - 1.96E4	1.47E4	1.37E4	PASS
			Cs-137	8.55E3 - 1.52E4	1.14E4	1.12E4	PASS
			Ce-141	3.09E4 - 5.48E4	4.12E4	4.18E4	PASS
5/29/97	0.5 liter Marinelli	pCi/liter	Mn-54	518 - 918	690	797	PASS
			Co-58	138 - 243	183	207	PASS
			Fe-59	167 - 297	223	300	PASS
			Co-60	589 - 1044	785	873	PASS
			Zn-65	467 - 827	622	677	PASS
			I-131	523 - 927	697	703	PASS
			Cs-134	461 - 817	614	560	PASS
			Cs-137	390 - 692	520	530	PASS
			Ce-141	179 - 317	238	253	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

(2) Reported value is an average of three analyses, two of which were above the control limit.

Gamma in Water continued:

Reference Date	Geometry	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status *
5/29/97	1.0 liter Marinelli	pCi/liter	Mn-54	518 - 918	690	780	PASS
			Co-58	138 - 243	183	203	PASS
			Fe-59	167 - 297	223	257	PASS
			Co-60	589 - 1044	785	843	PASS
			Zn-65	467 - 827	622	740	PASS
			I-131	523 - 927	697	700	PASS
			Cs-134	461 - 817	614	553	PASS
			Cs-137	390 - 692	520	523	PASS
			Ce-141	179 - 317	238	260	PASS
5/29/97	3.5 liter Marinelli	pCi/liter	Mn-54	518 - 918	690	730	PASS
			Co-58	138 - 243	183	193	PASS
			Fe-59	167 - 297	223	253	PASS
			Co-60	589 - 1044	785	843	PASS
			Zn-65	467 - 827	622	737	PASS
			I-131	523 - 927	697	727	PASS
			Cs-134	461 - 817	614	587	PASS
			Cs-137	390 - 692	520	547	PASS
			Ce-141	179 - 317	238	253	PASS
5/29/97	3.5 liter Marinelli	pCi/liter	Cr-51	5.59E3 - 9.92E3	7.46E3	8.06E3	PASS
			Mn-54	1.25E4 - 2.22E4	1.67E4	1.80E4	PASS
			Co-58	3.33E3 - 5.91E3	4.44E3	4.63E3	PASS
			Fe-59	4.04E3 - 7.17E3	5.39E3	5.93E3	PASS
			Co-60	1.43E4 - 2.53E4	1.90E4	2.03E4	PASS
			Zn-65	1.13E4 - 2.01E4	1.51E4	1.70E4	PASS
			I-131	1.27E4 - 2.25E4	1.69E4	1.80E4	PASS
			Cs-134	1.12E4 - 1.98E4	1.49E4	1.40E4	PASS
			Cs-137	9.45E3 - 1.68E4	1.26E4	1.30E4	PASS
			Ce-141	4.32E3 - 7.67E3	5.77E3	5.97E3	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

Gamma in Water continued:

Reference Date	Geometry	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status *
8/25/97	0.5 liter Marinelli	pCi/liter	Cr-51	3.08E4 - 5.45E4	4.10E4	4.20E4	PASS
			Mn-54	5.17E3 - 9.16E3	6.89E3	7.52E3	PASS
			Co-58	4.23E3 - 7.50E3	5.64E3	6.25E3	PASS
			Fe-59	9.60E3 - 1.70E4	1.28E4	1.42E4	PASS
			Co-60	1.10E4 - 1.96E4	1.47E4	1.59E4	PASS
			Zn-65	1.17E4 - 2.07E4	1.56E4	1.79E4	PASS
			I-131	1.82E4 - 3.22E4	2.42E4	2.29E4	PASS
			Cs-134	5.79E3 - 1.03E4	7.73E3	7.26E3	PASS
			Cs-137	5.95E3 - 1.06E4	7.93E3	7.98E3	PASS
			Ce-141	7.09E3 - 1.26E4	9.45E3	9.51E3	PASS
8/25/97	1.0 liter Marinelli	pCi/liter	Cr-51	3.08E4 - 5.45E4	4.10E4	4.28E4	PASS
			Mn-54	5.17E3 - 9.16E3	6.89E3	7.40E3	PASS
			Co-58	4.23E3 - 7.50E3	5.64E3	6.36E3	PASS
			Fe-59	9.60E3 - 1.70E4	1.28E4	1.41E4	PASS
			Co-60	1.10E4 - 1.96E4	1.47E4	1.57E4	PASS
			Zn-65	1.17E4 - 2.07E4	1.56E4	1.74E4	PASS
			I-131	1.82E4 - 3.22E4	2.42E4	2.38E4	PASS
			Cs-134	5.79E3 - 1.03E4	7.73E3	7.42E3	PASS
			Cs-137	5.95E3 - 1.06E4	7.93E3	8.13E3	PASS
			Ce-141	7.09E3 - 1.26E4	9.45E3	9.87E3	PASS
8/25/97	3.5 liter Marinelli	pCi/liter	Cr-51	3.08E4 - 5.45E4	4.10E4	4.29E4	PASS
			Mn-54	5.17E3 - 9.16E3	6.89E3	7.38E3	PASS
			Co-58	4.23E3 - 7.50E3	5.64E3	6.21E3	PASS
			Fe-59	9.60E3 - 1.70E4	1.28E4	1.40E4	PASS
			Co-60	1.10E4 - 1.96E4	1.47E4	1.57E4	PASS
			Zn-65	1.17E4 - 2.07E4	1.56E4	1.70E4	PASS
			I-131	1.82E4 - 3.22E4	2.42E4	2.43E4	PASS
			Cs-134	5.79E3 - 1.03E4	7.73E3	7.50E3	PASS
			Cs-137	5.95E3 - 1.06E4	7.93E3	7.85E3	PASS
			Ce-141	7.09E3 - 1.26E4	9.45E3	9.78E3	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

Gamma in Water continued:

Reference Date	Geometry	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status *
8/25/97	3.5 liter Marinelli	pCi/liter	Cr-51	359 - 636	478	506	PASS
			Mn-54	60.2 - 107	80.3	88.8	PASS
			Co-58	49.4 - 87.5	65.8	77.2	PASS
			Fe-59	112 - 198	149	166	PASS
			Co-60	129 - 229	172	177	PASS
			Zn-65	136 - 241	181	206	PASS
			I-131	212 - 375	282	212	BC ⁽³⁾
			Cs-134	67.5 - 120	90.0	88.8	PASS
			Cs-137	69.3 - 123	92.4	92.8	PASS
			Ce-141	82.5 - 146	110	119	PASS
12/2/97	3.5 liter Marinelli	pCi/liter	Cr-51	101 - 178	134	144	PASS
			Mn-54	162 - 287	216	236	PASS
			Co-58	62.6 - 111	83.5	94	PASS
			Fe-59	80.3 - 142	107	118	PASS
			Co-60	416 - 737	554	580	PASS
			Zn-65	344 - 609	458	512	PASS
			I-131	251 - 444	334	338	PASS
			Cs-134	206 - 366	275	268	PASS
			Cs-137	230 - 408	307	306	PASS
			Ce-141	33.5 - 59.3	44.6	50.2	PASS

* Pass = Data within Control Limits, AC = Above Control Limit, BC = Below Control Limit

(3) Reported value is an average of three analyses, one of which was below the control limit.

Iodine in Water:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status*
2/24/97	pCi/liter	I-131	8.01 - 14.3	10.8	11.4	PASS
5/29/97	pCi/liter	I-131	65.0 - 115	86.4	113	AC ⁽¹⁾
7/2/97	pCi/liter	I-131	144 - 255	192	215	PASS
7/2/97	pCi/liter	I-131	66 - 117	87.7	76	PASS
8/25/97	pCi/liter	I-131	16 - 28	20.8	24	PASS
8/25/97	pCi/liter	I-131	5.19 - 9.21	6.92	9.05	AC ⁽¹⁾
10/6/97	pCi/liter	I-131	36.0 - 63.8	48.0	45.7	PASS
10/6/97	pCi/liter	I-131	24.1 - 42.8	32.2	31.2	PASS
12/2/97	pCi/liter	I-131	31.1 - 55.2	41.5	42.6	PASS
12/2/97	pCi/liter	I-131	141 - 250	188	212	PASS

* Pass = Data within Control Limits, AC = Above Control Limit, BC = Below Control Limit

(1) Reported value is an average of three analyses, one of which was above the control limit..

Iodine in Milk:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status*
2/19/97	pCi/liter	I-131	72.4 - 128	96.5	104	PASS
5/29/97	pCi/liter	I-131	81.2 - 144	108	130	PASS
7/2/97	pCi/liter	I-131	167 - 297	223	219	PASS
8/25/97	pCi/liter	I-131	9.32 - 16.5	12.4	15.51	PASS
12/2/97	pCi/liter	I-131	96.2 - 171	128	131	PASS

Iodine in Milk continued:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status*
12/2/97	pCi/liter	I-131	31.5 - 55.9	42.0	45.8	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

Tritium in Water:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status*
2/19/97	pCi/liter	H-3	1013 - 1796	1350	1573	PASS
2/19/97	pCi/liter	H-3	19050 - 33782	25400	25113	PASS
5/29/97	pCi/liter	H-3	609 - 1080	813	978	PASS
5/29/97	pCi/liter	H-3	11985 - 21253	15980	15300	PASS
5/29/97	pCi/liter	H-3	1298 - 2302	1731	1918	PASS
5/29/97	pCi/liter	H-3	14768 - 26188	19690	17646	PASS
8/25/97	pCi/liter	H-3	4402 - 7806	5869	5574	PASS
8/25/97	pCi/liter	H-3	268 - 475	357	342	PASS
12/2/97	pCi/liter	H-3	7665 - 13593	10220	11141	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

Beta in Water:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status*
3/20/97	pCi/liter	Gross Beta	108 - 192	144	172	PASS
6/19/97	pCi/liter	Gross Beta	104 - 185	139	131	PASS
9/18/97	pCi/liter	Gross Beta	218 - 387	291	287	PASS
12/11/97	pCi/liter	Gross Beta	173 - 306	230	225	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

Beta in Air Particulate:

Reference Date	Units of Activity	Nuclide	Acceptance Range	Reference Value	Reported Value	Cross-Check Status*
2/21/97	dpm/total	Gross Beta	4928 - 8739	6571	6729	PASS
2/21/97	pCi/total	Gross Beta	2235 - 3963	2980	2828	PASS
8/22/97	pCi/total	Gross Beta	3570 - 6331	4760	4830	PASS

* Pass = Data within Control Limits , AC = Above Control Limit, BC = Below Control Limit

TABLE 5.0-C

STATE OF NORTH CAROLINA DEPARTMENT OF ENVIRONMENTAL HEALTH AND NATURAL RESOURCES

1997 ENVIRONMENTAL DOSIMETER CROSS-CHECK RESULTS

Cross-Check Date	State of N.C. Delivered Value (mR)	Radiation Dosimetry & Records Reported Value (mR)	Acceptance Criteria +/- 10 %
Jun-97	80.0	78.0	Pass

6.0 REFERENCES

- 6.1 Oconee Selected License Commitments
- 6.2 Oconee Technical Specifications
- 6.3 Oconee Final Safety Analysis Review
- 6.4 Oconee Offsite Dose Calculation Manual
- 6.5 Oconee Annual Environmental Operating Report 1969-1996
- 6.6 Oconee Annual Effluent Report 1997
- 6.7 Probability and Statistics in Engineering and Management Science, Hines and Montgomery, 1969, pages 287-293.
- 6.8 Practical Statistics for the Physical Sciences, Havilcek and Crain, 1988, pages 83-93.
- 6.9 Nuclear Regulatory Commission Regulatory Guide 1.109, Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purposes of Evaluating Compliance with 10CFR50, Appendix I.
- 6.10 Radiological and Environmental Services Operating Procedures
- 6.11 NUREG/CR-1276, Users Manual for LADTAP II - A Computer Program for Calculating Radiation Exposure to Man from Routine Release of Nuclear Reactor Liquid Effluents.
- 6.12 Oconee Environmental Chemistry Operating Procedures
- 6.13 NUREG - 0597, Users Guide to GASPAR Code

APPENDIX A

**ENVIRONMENTAL SAMPLING
&
ANALYSIS PROCEDURES**

APPENDIX A

ENVIRONMENTAL SAMPLING AND ANALYSIS PROCEDURES

Adherence to established procedures for sampling and analysis of all environmental media at Oconee Nuclear Station is required to ensure compliance with Station Selected Licensee Commitments. Analytical procedures were employed to ensure that Selected Licensee Commitments detection capabilities were achieved.

Environmental sampling and analyses were performed by ONS Environmental Chemistry, Radiological and Environmental Services, Dosimetry and Records, and Fisheries and Aquatic Ecology.

Section IV of this appendix describes the environmental sampling frequencies and analysis procedures by media type.

I. CHANGE OF SAMPLING PROCEDURES

Broadleaf vegetation location 028 was removed from the sampling program effective 1/01/97. Two other site boundary vegetation locations (077 and 079) with higher deposition factors continue to be sampled.

II. DESCRIPTION OF ANALYSIS PROCEDURES

Gamma spectroscopy analyses are performed using high purity germanium gamma detectors and Canberra analytical software. Designated sample volumes are transferred to appropriate counting geometries and analyzed by gamma spectroscopy. Perishable samples such as fish and broadleaf vegetation are ground to achieve a homogeneous mixture. Soils and sediments are dried, sifted to remove foreign objects (rocks, clams, glass, etc.) then transferred to appropriate counting geometry. Ten percent of samples receiving gamma analysis are analyzed as duplicate analyses.

Low-level iodine analyses are performed by passing a designated sample aliquot through an ion exchange resin to remove and concentrate any iodine in the aqueous sample (milk or water). The resin is then dried and transferred to appropriate counting geometry and analyzed by gamma spectroscopy.

Tritium analyses are performed quarterly by using low-level environmental liquid scintillation analysis technique on a Packard 2550 liquid scintillation system. Tritium samples are batch processed with a tritium spike to verify instrument performance and sample preparation technique are acceptable.

Gross beta analysis is performed by concentrating a designated aliquot of sample precipitate and analyzing by gas-flow proportional counters. Samples are batch processed with a spike sample to verify instrument performance and a blank to ensure sample contamination has not occurred.

III. CHANGE OF ANALYSIS PROCEDURES

No analysis procedures were changed during 1997.

IV. SAMPLING AND ANALYSIS PROCEDURES

A.1 AIRBORNE PARTICULATE AND RADIOIODINE

Airborne particulate and radioiodine samples at each of six locations were composited continuously by means of continuous air samplers. Air particulates were collected on a particulate filter and radioiodines were collected in a charcoal cartridge positioned behind the filter in the sampler. The samplers are designed to operate at a constant flow rate (in order to compensate for any filter loading) and are set to sample approximately 2 cubic feet per minute. Filters and cartridges were collected weekly. A weekly gamma analysis was performed on each filter and a weekly gamma analysis was performed on each charcoal cartridge. The filter and charcoal cartridge were analyzed independently. The continuous composite samples were collected from the locations listed below.

Location 060	=	New Greenville Water Intake Rd. (2.6 mi. NNE)
Location 073	=	Tamassee Dar School (9.2 mi. NW)
Location 074	=	Keowee Key Resort (2.3 mi. NNW)
Location 077	=	Skimmer Wall (1.0 mi. SW)
Location 078	=	Recreation Site (0.6 mi. WSW)
Location 079	=	Keowee Dam (0.5 mi. NE)

A.2 DRINKING WATER

Monthly composite samplers were operated to collect an aliquot at least every two hours. Low-level Iodine-131, gross beta, and gamma analysis was performed on the monthly composites. Tritium analysis was performed on the quarterly composites. The composites were collected monthly from the locations listed below.

Location 060	=	New Greenville Water Intake Rd. (2.6 mi. NNE)
Location 064	=	Seneca (6.7 mi. SSW)
Location 066	=	Anderson (19.0 mi SSE)

A.3 SURFACE WATER

Monthly composite samplers were operated to collect an aliquot at least every two hours. Gamma analysis was performed on the monthly composites. Tritium analysis was performed on the quarterly composites sample. The composites were collected monthly from the locations listed below.

Location 062	=	Lake Keowee/Hydro Intake (0.8 mi. ENE)
Location 063	=	Lake Hartwell - Hwy 183 Bridge (0.8 mi. ESE)

A.4 MILK

Semimonthly grab samples were collected at each dairy. A gamma and low-level Iodine-131 analysis was performed on each sample. The semimonthly grab samples were collected from the locations listed below.

Location 069	=	Orr's Dairy - (4.5 mi. WNW)
Location 071	=	Clemson Dairy - (10.3 mi. SSE)
Location 080	=	Martin's Dairy - (19.0 mi. SSE)

A.5 BROADLEAF VEGETATION

Monthly samples were collected and a gamma analysis was performed on each sample. The samples were collected from the locations listed below.

Location 060	=	New Greenville Water Intake Rd. (2.6 mi. NNE)
Location 073	=	Tamassee Dar School (9.2 mi. NW)
Location 077	=	Skimmer Wall (1.0 mi. SW)
Location 079	=	Keowee Dam (0.5 mi. NE)

A.6 FISH

Semiannual samples were collected and a gamma analysis was performed on the edible portions of each sample. The samples were collected from the locations listed below.

Location 060	=	New Greenville Water Intake Rd. (2.6 mi. NNE)
Location 063	=	Lake Hartwell - Hwy 183 Bridge (0.8 mi. ESE)
Location 067	=	Lawrence Ramsey Bridge, Hwy 27 (4.2 mi. SSE)

A.7 SHORELINE SEDIMENT

Semiannual samples were collected and a gamma analysis was performed on each sample following the drying and removal of rocks and clams. The samples were collected from the locations listed below.

Location 063	=	Lake Hartwell - Hwy 183 Bridge (0.8 mi. ESE)
Location 067	=	Lawrence Ramsey Bridge, Hwy 27 (4.2 mi. SSE)
Location 068	=	High Falls County Park (2.0 mi. W)

A.8 DIRECT GAMMA RADIATION (TLD)

Thermoluminescent dosimeters (TLD) were collected quarterly at forty-one locations. A gamma exposure rate was determined for each TLD. The TLDs were placed as indicated below.

- * An inner ring of 17 TLDs, one in each meteorological sector in the general area of the site boundary.
- * An outer ring of 16 TLDs, one in each meteorological sector in the 6 to 8 kilometer range.
- * The remaining TLDs were placed in special interest areas such as population centers, residential areas, schools, and control locations.

TLD Locations are listed in Table 2.1-B.

A.9 ANNUAL LAND USE CENSUS

An annual Land Use Census was conducted to identify within a distance of 8 kilometers (5.0 miles) from the station, the following locations in each of the sixteen meteorological sectors:

- * The Nearest Residence
- * The Nearest Meat Animal
- * The Nearest Milk-giving Animal (cow, goat, etc.) where milk is used for human consumption

The census was conducted during the growing season from 8/19 to 8/20/97. Results are shown in Table 3.9.

APPENDIX B

**RADIOLOGICAL
ENVIRONMENTAL MONITORING
PROGRAM**

SUMMARY OF RESULTS

1997

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.	
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range		
Air Particulate (pCi/m3)						073 (9.2 mi NW)		
	BETA	312	1.00E-02	2.42E-2 (260/260)	079	2.87E-2 (52/52)	2.92E-2 (52/52)	0
				3.70E-3 - 6.41E-2	(0.5 mi NE)	6.19E-3 - 6.41E-2	4.97E-3 - 6.22E-2	
	CS-134	312	5.00E-02	0.00 (0/260)		0.00 (0/52)	0.00 (0/52)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137	312	6.00E-02	0.00 (0/260)		0.00 (0/52)	0.00 (0/52)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	I-131	312	7.00E-02	0.00 (0/260)		0.00 (0/52)	0.00 (0/52)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00		

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Air Radioiodine (pCi/m3)				073 (9.2 mi NW)			
CS-134	312	5.00E-02	0.00 (0/260)		0.00 (0/52)	0.00 (0/52)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
CS-137	312	6.00E-02	0.00 (0/260)		0.00 (0/52)	0.00 (0/52)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
I-131	312	7.00E-02	0.00 (0/260)		0.00 (0/52)	0.00 (0/52)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Drinking Water (pCi/liter)	064 (6.7 mi SSW)						
	BALA-140	39	15		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	BETA	39	4	066	2.52 (22/26)	2.23 (11/13)	0
				(19.0 mi SSE)	0.72 - 4.52	1.31 - 3.54	
	CO-58	39	15		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CO-60	39	15		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CS-134	39	15		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CS-137	39	18		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	FE-59	39	30		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	H-3	15	2000	066	194 (3/10)	0.00 (0/5)	0
				(19.0 mi SSE)	187 - 204	0.00 - 0.00	
	I-131	39	15		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	LLI-131	39	1		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	MN-54	39	15		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	NB-95	39	15		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	ZN-65	39	30		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	ZR-95	39	30		0.00 (0/26)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Surface Water (pCi/liter)						062 (0.8 mi ENE)	
	BALA-140	26	15		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CO-58	26	15		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CO-60	26	15		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CS-134	26	15		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CS-137	26	18		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	FE-59	26	30		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	H-3	10	2000	063 (0.8 mi ESE)	5497 (5/5)	0.00 (0/5)	0
					1940 - 11050	0.00 - 0.00	
	I-131	26	15		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	MN-54	26	15		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	NB-95	26	15		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	ZN-65	26	30		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	ZR-95	26	30		0.00 (0/13)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269, 270, 287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Milk (pCi/liter)						080 (19.0 mi SSE)	
	BALA-140	78	15	0.00 (0/52)	0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	CS-134	78	15	0.00 (0/52)	0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	CS-137	78	18	0.00 (0/52)	0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	I-131	78	15	0.00 (0/52)	0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	LLI-131	78	1	0.00 (0/52)	0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Broadleaf Vegetation (pCi/kg-wet)						073 (9.2 mi NW)	
	CS-134	52	60		0.00 (0/39)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	
	CS-137	52	80		31.5 (3/39)	47.3 (1/13)	0
					21.3 - 47.3	36.8 - 307	
				077 (1.0 mi SW)	47.3 - 47.3		
	I-131	52	60		0.00 (0/39)	0.00 (0/13)	0
					0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Fish (pCi/kg-wet)						060 (2.6 mi NNE)	
	CO-58	12	130		0.00 (0/8)	0.00 (0/4)	0
					0.00 - 0.00	0.00 - 0.00	
	CO-60	12	130		0.00 (0/8)	0.00 (0/4)	0
					0.00 - 0.00	0.00 - 0.00	
	CS-134	12	130		0.00 (0/8)	0.00 (0/4)	0
					0.00 - 0.00	0.00 - 0.00	
	CS-137	12	150	063	96.2 (8/8)	118 (4/4)	0
				(0.8 mi ESE)	44.1 - 219	72.8 - 219	
	FE-59	12	260		0.00 (0/8)	0.00 (0/4)	0
					0.00 - 0.00	0.00 - 0.00	
	MN-54	12	130		0.00 (0/8)	0.00 (0/4)	0
					0.00 - 0.00	0.00 - 0.00	
	ZN-65	12	260		0.00 (0/8)	0.00 (0/4)	0
					0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non-Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Shoreline Sediment (pCi/kg-dry)						068 (2.0 mi W)	
	CO-58	6	0	0.00 (0/6)	0.00 (0/2)	0.00 (0/2)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	CS-134	6	150	0.00 (0/6)	0.00 (0/2)	0.00 (0/2)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	CS-137	6	180	93.1 (3/4)	067 106 (2/2)	38.3 (2/2)	0
				62.3 - 150 (4.2 mi SSE)	62.3 - 150	23.6 - 53.0	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: Oconee Nuclear Station

Docket No. 50-269,270,287

Location: Oconee County, South Carolina

Report Period: 01-JAN-1997 to 31-DEC-1997

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non- Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean Range	Mean Range	
Direct Radiation TLD (mR/standard quarter)						058 (9.4 mi WSW)	
	164	0.00E+00	19.7 (160/160)	059	26.2 (4/4)	26.1 (4/4)	0
			11.1 - 29.0	(9.2 mi NW)	25.0 - 27.0	25.8 - 26.8	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

APPENDIX C

**SAMPLING DEVIATIONS
&
UNAVAILABLE ANALYSES**

No deviations or unavailable analyses were incurred for the
1997 Radiological Environmental Monitoring Program

APPENDIX D

ANALYTICAL DEVIATIONS

No analytical deviations were incurred for the
1997 Radiological Environmental Monitoring Program

APPENDIX E

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM RESULTS

This appendix includes all of the sample analysis reports generated from each sample medium for 1997. Appendix E is located separately from this report and is permanently archived at Duke Power Company's Environmental Center radiological environmental master file, located at the McGuire Nuclear Station Site in Huntersville, North Carolina.