

AUG 17 1978

Docket Nos. 50-260/270/287

Duke Power Company
ATTN: Mr. William O. Parker
Vice President-Steam Production
Power Building
422 South Church Street
Charlotte, North Carolina 28242

Gentlemen:

This is in reply to your letter of July 17, 1978, in which you submitted a revised "Oconee Nuclear Station Requalification Program for NRC Licensed Personnel." Specifically, the revised program:

1. clarifies the definition of "Backup Licensee" and
2. relieves such Backup Licensees from mandatory participation in the requalification lectures preceeding the annual examination segments.

Based on our review, we have concluded that the revision does not materially decrease the effectiveness of the program. The revised program continues to meet the requirements of Section 50.54(i-1) of 10 CFR Part 50 and Appendix A of 10 CFR Part 55.

Sincerely,

Paul F. Collins, Chief
Operator Licensing Branch
Division of Project Management

cc: Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Continued on next page.

MS-4
R

R. Reid

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DATE	8/14/78	8/14/78	8/15/78			

August 15, 1978

Dockets Nos.: 50-269

and 50-287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President - Steam
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P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

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Gentlemen:

By letter dated May 16, 1978, Duke Power Company submitted a proposed program for the installation of a limited number of tube sleeves in the Oconee Station once-through steam generator tubes. The purpose of these sleeves is to reduce dynamic stress in the region of previously indicated tube abnormalities.

We have previously reviewed the document submitted with your May 16 letter, entitled "Technical Justification for Installing and Testing Tube Sleeves in Once-Through Steam Generator - October 3, 1977 (Rev. 1 February 17, 1978)" on the Three Mile Island Unit No. 2 docket. The results of our review are provided in the Safety Evaluation Report, Supplement 2, for TMI-2, dated February 1978. Based on the above review we have no objection to your proposed installation and test program. A copy of page 5-4 of the TMI-2 Safety Evaluation Report is enclosed.

Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosure:
Page 5-4 of TMI-2 SER

cc w/enclosure: See next page

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Duke Power Company

cc: Mr. William L. Porter
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Tube sleeving test program

Both Babcock & Wilcox and Combustion Engineering have designed test sleeves for insertion in the upper end of a number of steam generator tubes, with appropriate instrumentation, to assess their effect in stiffening the tubes to improve their capability to resist flow-induced vibration, the most likely cause of tube damage found in some other Babcock & Wilcox steam generators.

Safety evaluations have been submitted by the applicant for both designs, including descriptions of the sleeves, locations in the steam generator, method of installation, instrumentation, bench test results, stress analyses, and an evaluation of the effects of the test equipment on the safety of the plant.

Based on a review of the information supplied by the applicant and the similarity of the sleeves to installations in some Combustion Engineering steam generators, we conclude that neither the structural integrity nor the operational characteristics of safety-related equipment involved will be affected by the test program modifications, nor will the consequences of an accident be increased as a result thereof, and therefore that these modifications are acceptable.

After completion of all the above programs we will determine what modifications, if any, we will require to be made to Three Mile Island Unit 2.

Three Mile Island Unit No. 2

Supplement 2 to SER

February 1978