Docket File

DOCKET NOS. 50-269/270/287 DATE: JUL 1 1977

LICENSEE: DUKE POWER COMPANY

FACILITY: Oconee Nuclear Station, Unit Nos. 1, 2 and 3

SUMMARY OF MEETING HELD ON JUNE 22, 1977, TO DISCUSS THE SEISMIC MONITORING PROGRAM AT JOCASSEE DAM

On June 22, 1977, representatives of Duke Power Company (DPC) met with us to discuss the subject topic.

A list of attendees is attached.

The meeting was requested by DPC to discuss technical and schedular aspects of the seismic monitoring and reporting program outlined in our letter dated May 20, 1977.

We provided clarifying information for the short term report referenced in our letter. We agreed that it would be desirable for this to be a complete report even if it meant the submittal date may be a couple of weeks beyond that stated in our letter.

On the monitoring and reporting program, we agreed to the following:

a. Quarterly reports may be submitted within 45 days at the end of the reporting period instead of one month.

b. Four portable monitoring stations in addition to the permanent station already installed is a suitable alternative to three permanent stations. These are to be installed by July 15, 1977.

/S/

D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Operating Reactors

Enclosure:
List of attendees

cc w/encl: See next page

MEMO 4

OFFICE	DOR:ORB#1					
SURNAME	DNeighbors:lb					
DATE	6/30/77					

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Docket
NRC PDR
LOCAL PDR
ORB#1 Reading
NRR Reading
E. G. Case
V. Stello
K. R. Goller
D. Eisenhut
A. Schwencer
D. Davis
G. Lear
R. Reid
L. Shao
B. Grimes
W. Butler
R. Baer
Project Manager
Attorney, OELD
OI&E (3)
Licensing Assistant
Each NRC participant
T. B. Abernathy
J. R. Buchanan

ATTENDEES AT NRC
MEETING WITH
DUKE POWER COMPANY

JUNE 22, 1977

NRC

D. Neighbors
R. Jackson
J. Kelleher
W. Gammill
D. Simpson
L. Heller

DPC

M. Tuckman
L. C. Dail
R. Bhatnager

JUL 1 1977

OFFICE ➤						
SURNAME ➤						
DATE ➤						

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7/1/77

5. Based on additional information provided by the licensee, complete evaluation of fracture toughness of steam generator and reactor coolant pump support materials for each plant having potential problem(s) as identified under Item 4.

6. If the final evaluation based on the additional information or proposed solution provided by the licensee identifies additional concerns, a specific action plan for the plant(s) in question will be developed.

DSS/MEB and DSS/MTEB should follow the activities of the task organization and determine if action on CP/OL reviews will be necessary.

Substantial interaction with outside organizations (licensees and successful bidder per subparagraph 2. above) is expected.

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During the course of the licensing action for North Anna Power Station Unit Nos. 1 and 2, a number of questions were raised as to the potential for lamellar tearing and low fracture toughness of the steam generator and reactor coolant pump support materials for those units. Two different steel specifications (ASTM A36-70a and ASTM A572-70a) covered most of the material used for these supports. Toughness tests, not originally specified and not in the relevant ASTM specifications, were made on those heats for which excess material was available. The toughness of the A36 steel was found to be adequate, but the toughness of the A572 steel was relatively poor at an operating temperature of 80°F. In the case of North Anna Unit Nos. 1 and 2, the applicant has agreed to raise the temperature of the ASTM A572 beams in the steam generator supports to a minimum temperature of 225°F prior to pressurization to levels above 1000psig. Auxiliary electrical heat will be employed to supplement the heat derived from the reactor coolant loop as necessary to obtain the required operating temperature of the structures.

Accordingly, the following tasks for DOR/EB have been submitted as part of the NRR Category A Task Action Plan on this subject:

1. Send generic letter to all PWR licensees stating our concerns and requesting information on the design, materials, fabrication and inspection of the steam generator and reactor coolant pump supports for each plant.

2. Obtain a consultant to provide expertise in evaluating the potential for lamellar tearing and low fracture toughness of the support materials. It is the intent that the consultant will assist in the review work and perform a literature search for fracture toughness data on the materials in question.

3. Based on information supplied by the licensees, categorize the support design and materials as far as practical and select typical designs for further study.

4. Complete preliminary review of typical designs and inform each applicable PWR licensee of the concerns on their particular support system. Where a problem exists, the licensee should be required to provide additional information to demonstrate that the safety margins for their supports are not lower than anticipated. If this cannot be done, the licensee should propose a solution to the problem. For those plants identified as not having a problem, a Safety Evaluation should be completed.