

March 27, 1978

Dockets Nos.: 50-270
and 50-287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President - Steam
Production
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

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Gentlemen:

RE: OCONEE NUCLEAR STATION UNITS NOS. 2 AND 3

By letters dated July 8, September 21, 1977 and March 6, 1978, you submitted a proposed inservice inspection and testing program description and a request for relief from selected ASME Code requirements pursuant to 10 CFR 50.55a(g). Although we have not completed our detailed review of your submittal, our preliminary review makes clear to us that your proposed program to implement those ASME Code requirements that you have found to be practical would increase the scope of inservice inspection and testing for your facility beyond that currently required by your Technical Specifications. We have concluded that this upgrading of your inservice inspection and testing program will further enhance safety.

Based on our preliminary review, we agree with your determination that it is impractical within the limitations of design, geometry and materials of construction of components, for you to meet certain of the specified ASME Code requirements and that imposition of those requirements would result in hardships or unusual difficulties without a compensating increase in the level of quality or safety. Therefore, pursuant to 10 CFR 50.55a(g)(6)(i), we hereby grant relief, on an interim basis, pending completion of our detailed review, from those inservice inspection and testing requirements of the ASME Code that you have requested. Moreover, since the scope of the inservice inspection and testing will be increased by your proposed program, and the granting of this relief is based only on the impracticality of selected ASME Code requirements, we have determined that the relief granted neither increases the probability or consequences of accidents previously considered nor decreases safety margins and that, therefore, it does not involve a significant hazards consideration. Therefore, you are authorized to, and should proceed to, implement your proposed program (except where your current Technical Specifications are more restrictive).

OFFICE ➤						
SURNAME ➤						
DATE ➤						

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Duke Power Company

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During the period between now and the date we complete our detailed review of your submittal, you must comply with both your existing Technical Specifications and your proposed inservice inspection and testing program. In the event conflicting requirements arise for some components, you must comply with the more restrictive requirements (e.g., shorter inspection intervals, increased number of parameters measured). In other words, the granting of this relief from ASME Code requirements should not be interpreted to give you relief from any of the requirements in your existing Technical Specifications.

When our detailed review of your [redacted] submittal is complete we will: (1) issue final approval of your program (which may contain modifications resulting from the staff's review), (2) grant relief from any ASME Code requirements that are determined to be impractical for your facility for the duration of the inspection interval and (3) issue appropriate changes to your Technical Specifications.

A copy of the Federal Register Notice related to this action is enclosed.

Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosure:
Federal Register Notice

cc w/enclosure:
See next page,

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UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKETS NOS. 50-270 AND 50-287DUKE POWER COMPANYNOTICE OF GRANTING OF RELIEF FROM ASME SECTION XI
INSERVICE INSPECTION (TESTING) REQUIREMENTS

The U. S. Nuclear Regulatory Commission (the Commission) has granted relief from certain requirements of the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components" to Duke Power Company. The relief relates to the inservice inspection (testing) program for the Oconee Nuclear Station, Units Nos. 2 and 3, located in Oconee County, South Carolina. The ASME Code requirements are incorporated by reference into the Commission's rules and regulations in 10 CFR Part 50. The relief is effective as of its date of issuance.

The relief is granted, on an interim basis, pending completion of our detailed review, from those inservice inspection and testing requirements of the ASME Code that the licensee has determined to be impractical within the limitations of design, geometry, and materials of construction of components, because compliance would result in hardships and unusual difficulties without a compensating increase in the level of quality or safety.

The request for relief complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's

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rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the letter granting relief. Prior public notice of this action was not required since the granting of this relief from ASME Code requirements does not involve a significant hazards consideration.

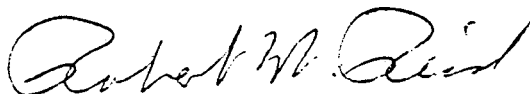
The Commission has determined that the granting of this relief will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

For further details with respect to this action, see (1) the requests for relief dated July 8, September 21, 1977 and March 6, 1978, and (2) the Commission's letter to the licensee dated March 27, 1978.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Oconee County Library, 201 South Spring, Walhalla, South Carolina 29691. A copy of item (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 27th day of March 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Duke Power Company

cc: Mr. William L. Porter
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