



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 24, 2015

LICENSEE: FirstEnergy Nuclear Operating Company

FACILITY: Davis-Besse Nuclear Power Station, Unit 1

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON MAY 06, 2015, AND MAY 19, 2015, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND FIRSTENERGY NUCLEAR OPERATING COMPANY, TO CLARIFY THE RESPONSES TO REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1, LICENSE RENEWAL APPLICATION (TAC NO. ME4640)

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of FirstEnergy Nuclear Operating Company held telephone conference calls on May 06 and May 19, 2015, to clarify information in the applicant's responses to requests for additional information (RAIs) concerning the safety review of the Davis-Besse Nuclear Power Station, Unit 1, license renewal application. The teleconference was useful in clarifying the information provided in the applicant's recent submittal of the reactor vessel internals inspection plan dated April 21, 2015.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a summary of the items discussed with the applicant regarding the submittal.

The applicant had an opportunity to comment on this summary.

Sincerely,

/RA by Jeff Mitchell for /

Richard Plasse, Project Manager
Projects Branch 1
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosure:
As stated

cc: Listserv

July 24, 2015

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ADAMS Accession No.: ML15196A516

*concurrent via email

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DATE	7/ 21 /15	7/ 23 /15	7/ 23 /15	7/ 24 /15

OFFICIAL RECORD COPY

Memo to FirstEnergy Nuclear Operating Company from R. Plasse dated July 24, 2015

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON MAY 06, 2015, AND MAY 19, 2015, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND FIRSTENERGY NUCLEAR OPERATING COMPANY, TO CLARIFY THE RESPONSES TO REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1, LICENSE RENEWAL APPLICATION (TAC NO. ME4640)

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TELEPHONE CONFERENCE CALL
DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS

PARTICIPANTS

AFFILIATIONS

Richard Plasse	U.S. Nuclear Regulatory Commission (NRC)
James Medoff	NRC
Cliff Custer	FirstEnergy Nuclear Operating Company (FENOC)
Larry Hinkle	FENOC
Stephen Slosnerick	FENOC
Steven Dort	FENOC
Trent Henline	FENOC
Bev Watson	AREVA
Jeff Fleck	AREVA
Brian Haibach	AREVA
Ryan Hosler	AREVA
Hasan Charkas	AREVA
Tim Wiger	AREVA
Don Kim	AREVA

DAVIS-BESSE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION
REACTOR VESSEL INTERNALS INSPECTION PLAN
CLARIFICATIONS

The telephone conference calls held on May 06 and May 19, 2015, were initiated by the Nuclear Regulatory Commission staff. The purpose of the calls was to clarify the information provided in the Davis-Besse Nuclear Power Station Reactor Vessel Internals Inspection Plan submitted by letter L-15-139 (ADAMS Accession No. ML15113B134) dated April 21, 2015, as follows:

- Identification of Specific AREVA fabrication record that confirmed the upper flange weld in the core support structure of the Davis Besse reactor had been stress relieved during initial weld fabrication.
- Identify which reactor vessel internal bolting components were replaced with bolts made from X-750 nickel alloy materials and had been analyzed with a cumulative usage factor (CUF) analysis (i.e., ASME low cycle fatigue analysis). Amend the license renewal application (LRA) (time-limited aging analysis (TLAA) sections and updated safety analysis report (USAR) Supplement sections) for these bolts to change the basis for accepting the CUF analyses under 10 CFR 54.21(c)(1)(iii) from use of the Fatigue Monitoring Program to the use of the Fatigue Monitoring Program and Reactor Vessel Internals Program.
- Amend aging management review item on loss of fracture toughness for reactor internal vent valve assembly top and bottom retaining rings in LRA Table 3.1.2-2 to include a plant-specific note that credits the inspections and surveillance testing requirements in Technical Specification 5.5.4 as additional aging management bases for vent valve assembly components.
- Identify how many physical measurements of the plenum rib pad-to-reactor vessel seating surface height were performed consistent one-time physical measurement criteria in MRP-227-A and clarify how the average measured compared to the 0.004 inch acceptance criteria set in Table 4-1 of MRP-227-A for these types of measurements.
- Update the basis for accepting the reduction of ductility analysis for reactor vessel internals components (i.e., the subject of the TLAA in LRA Section 4.2.7) under 10 CFR 54.21(c)(1)(iii) to reflect the new commitment for this TLAA in Commitment No. 3 of the letter of April 21, 2015, in addition to the use of the Reactor Vessel Internals Program.
- Update Commitment Nos. 1, 2, and 3 in the letter of April 21, 2015, to be for NRC review and approval and renumber the amended commitments and incorporate them into Table A-1 of the USAR Supplement.

Following discussion of the information that the NRC staff requested clarification, as noted above, FENOC committed to provide the details in an updated license renewal amendment.

ENCLOSURE 2