



JUL 06 2015

LR-N15-0138

10 CFR 50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Salem Nuclear Generating Station Unit 1
Renewed Facility Operating License No. DPR-70
NRC Docket No. 50-272

SUBJECT: Licensee Event Report 272/2015-002-001

REFERENCE: PSEG Letter LR-N15-0056, dated April 10, 2015
Licensee Event Report 272/2015-002-000

The Licensee Event Report (LER), 272/2015-002-001, "Condition Prohibited by Technical Specification for One Channel of Steam Generator Level Indication Inoperable" is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation which was prohibited by the plant's Technical Specifications..." The referenced LER indicated that Salem Nuclear Generating Station would submit a supplement to the LER by July 9, 2015. The results of the causal evaluation are provided in the LER supplement attached to this letter.

Should you have any questions or comments regarding the submittal, please contact David Lafleur of Salem Regulatory Assurance at 856-339-1754.

There are no regulatory commitments contained in this letter.

Sincerely,


John F. Perry
Site Vice President – Salem

Attachments (1)

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cc Mr. D. Dorman, Administrator – Region 1, NRC
 Ms. C. Parker, Licensing Project Manager – Salem, NRC
 Mr. P. Finney, USNRC Senior Resident Inspector, Salem (X24)
 Mr. P. Mulligan, Manager IV, NJBNE
 Mr. R. Braun, President and Chief Nuclear Officer – Nuclear
 Mr. T. Cachaza, Salem Commitment Tracking Coordinator
 Mr. L. Marabella, Corporate Commitment Tracking Coordinator
 Mr. D. Lafleur, Salem Regulatory Assurance



LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of
digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Salem Generating Station – Unit 1	2. DOCKET NUMBER 05000272	3. PAGE 1 OF 3
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4. TITLE Condition Prohibited by Technical Specification for One Channel of Steam Generator Level Indication Inoperable

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	19	2013	2015	- 002	- 001	07	06	2015	FACILITY NAME	DOCKET NUMBER 05000
									FACILITY NAME	DOCKET NUMBER 05000

9. OPERATING MODE	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)			
1 10. POWER LEVEL 100%	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A	

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME	TELEPHONE NUMBER (Include Area Code)
David Lafleur, Senior Regulatory Compliance Engineer, Salem Generating	(856)339-1611

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
B	JE	LT	R369	Y					

14. SUPPLEMENTAL REPORT EXPECTED	15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On August 5, 2014, control room operators identified one steam generator (SG) protection level channel indicator as drifting high and approaching its 3 percent level deviation limit. Subsequent troubleshooting performed on October 10, 2014 identified the level transmitter as inoperable due to exceeding its Technical Specification (TS) calibration acceptance criteria. A past functionality evaluation was completed on February 12, 2015. This evaluation determined that the best estimate for when the transmitter would have exceeded its TS calibration acceptance criteria was August 19, 2013.

The cause of the drifting indication of the steam generator level indicator was due to failure of the 11 SG, channel 2, level transmitter, 1LT519.

This report is made in accordance with 10 CFR 50.73 (a)(2)(i)(B), "Any operation or condition which was prohibited by the plant's Technical Specifications...."

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME	2. DOCKET	6. LERNUMBER			3. PAGE
Salem Generating Station – Unit 1	05000272	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		2015	- 002	- 001	

NARRATIVE**PLANT AND SYSTEM IDENTIFICATION**

Westinghouse - Pressurized Water Reactor {PWR/4}
Steam Generator Level Transmitter {JE/LIT}

Energy Industry Identification System (EIS) codes and component function identifier codes appear as {SS/CCC}.

IDENTIFICATION OF OCCURRENCE

Event Date: August 19, 2013

Discovery Date: February 12, 2015

CONDITIONS PRIOR TO OCCURRENCE

Salem Unit 1 was in operational Mode 1, operating at approximately 100 percent power. No additional structures, systems or components were inoperable at the time of discovery that contributed to this event.

DESCRIPTION OF OCCURRENCE

On April 11, 2013, a new level transmitter was installed for 11 SG, Channel 2, 1LT519 as part of a planned 10 year replacement program.

On August 5, 2014, control room operators identified 1LT519 indication as drifting high and approaching the 3 percent level deviation limit but not exceeding the TS level channel limit.

On October 10, 2014, at 1014, troubleshooting of 1LT519 identified the transmitter as high outside its TS calibration acceptance criteria. The transmitter was calibrated and returned to service.

On February 12, 2015, a past functionality evaluation was completed for 1LT519. The evaluation, based on drift trending, concluded that the best estimate for when 1LT519 would have exceeded its TS calibration acceptance criteria was on August 19, 2013, approximately 130 days after its installation on April 11, 2013.

The subsequent surveillance test performed on February 26, 2015 met its TS acceptance criteria. 1LT519 was subsequently replaced on March 27, 2015.

Salem Unit 1 TS 3.3-1, Action 6 requires that startup or power operation may continue with an inoperable SG level channel provided it is placed in a tripped condition within 6 hours. 1LT519 would have exceeded its TS acceptance criteria on August 19, 2013 and was not placed in a tripped condition within 6 hours as

(01-2014)

LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

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NARRATIVE

required by TS.

This event is reported in accordance with 10 CFR 50.73 (a)(2)(i)(B), "Any operation or condition which was prohibited by the plant's Technical Specifications...."

CAUSE OF EVENT

The cause of the 11 SG, Channel 2 drifting indication and subsequent failure was due to failure of the 1LT519 level transmitter. The completion of the past functionality review was delayed and not completed until February 12, 2015 due to lack of timeliness guidance in the past functionality evaluation process.

SAFETY CONSEQUENCES AND IMPLICATIONS

The safety significance of this event is minimal. This event did not result in any offsite release of radioactivity or increase of offsite dose rates, and there were no personnel injuries or damage to any other safety-related equipment. The remaining two 11 SG level protection channels were functional and their SG LO-LO Level protective trip functions remained capable of generating a reactor trip signal during the time in which 1LT519 was inoperable.

SAFETY SYSTEM FUNCTIONAL FAILURE

A review of this event determined that a Safety System Functional Failure (SSFF) as defined in Nuclear Energy Institute (NEI) 99-02, Regulatory Assessment Performance Indicator Guideline, did not occur. This event did not prevent the ability of a system to fulfill its safety function to either shutdown the reactor, remove residual heat, control the release of radioactive material, or mitigate the consequences of an accident.

PREVIOUS OCCURRENCES

A review of Salem Unit 1 and 2 Licensee Event Reports for the previous three years identified no other similar events.

CORRECTIVE ACTIONS

1. 1LT519 was replaced and tested satisfactorily on March 27, 2015.
2. The removed 1LT519 has been sent to the manufacturer for failure analysis.
3. Guidance will be added to the engineering technical evaluation procedure for past operability and functionality reviews to be completed within 45 days from the discovery date.

COMMITMENTS

This LER contains no regulatory commitments.