



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 28, 2015

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear Connecticut, Inc.
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION, UNIT NO. 3 - ISSUANCE OF AMENDMENT
REGARDING TECHNICAL SPECIFICATION CHANGES TO AUXILIARY
FEEDWATER SYSTEM (TAC NO. MF4692)

Dear Mr. Heacock:

The Commission has issued the enclosed Amendment No. 263 to Renewed Facility Operating License No. NPF-49 for the Millstone Power Station, Unit No. 3 (MPS3). This amendment is in response to your application dated August 19, 2014, as supplemented on January 26, 2015.

The amendment revises MPS3 Technical Specification (TS) 3/4.7.1.2, "Auxiliary Feedwater System," Surveillance Requirement 4.7.1.2.1.b. by replacing the surveillance frequency and acceptance criteria for the Auxiliary Feedwater (AFW) pumps with a reference to the Inservice Testing Program (TS 4.0.5) for the specific pump testing acceptance criteria and the surveillance frequency, which is consistent with the other pump and valve surveillance requirements in the TS. The amendment also adds information on suitable plant conditions for performance of the steam turbine driven AFW pump surveillance.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "R. Guzman", is written over a horizontal line.

Richard V. Guzman, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures:

1. Amendment No. 263 to NPF-49
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-423

MILLSTONE POWER STATION, UNIT NO. 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 263
Renewed License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated August 19, 2014, as supplemented on January 26, 2015, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, revised through Amendment No. 263 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated into the license. DNC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael I. Dudek, Acting Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License
and Technical Specifications

Date of Issuance: July 28, 2015

ATTACHMENT TO LICENSE AMENDMENT NO. 263

RENEWED FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

4

Insert

4

Replace the following page of the Appendix A Technical Specifications, with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

3/4 7-5

Insert

3/4 7-5

(2) Technical Specifications

The Technical Specifications contained in Appendix A, revised through Amendment No. 263 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated into the license. DNC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- (3) DNC shall not take any action that would cause Dominion Resources, Inc. (DRI) or its parent companies to void, cancel, or diminish DNC's Commitment to have sufficient funds available to fund an extended plant shutdown as represented in the application for approval of the transfer of the licenses for MPS Unit No. 3.
- (4) Immediately after the transfer of interests in MPS Unit No. 3 to DNC, the amount in the decommissioning trust fund for MPS Unit No. 3 must, with respect to the interest in MPS Unit No. 3, that DNC would then hold, be at a level no less than the formula amount under 10 CFR 50.75.
- (5) The decommissioning trust agreement for MPS Unit No. 3 at the time the transfer of the unit to DNC is effected and thereafter is subject to the following:
- (a) The decommissioning trust agreement must be in a form acceptable to the NRC.
 - (b) With respect to the decommissioning trust fund, investments in the securities or other obligations of Dominion Resources, Inc. or its affiliates or subsidiaries, successors, or assigns are prohibited. Except for investments tied to market indexes or other non-nuclear-sector mutual funds, investments in any entity owning one or more nuclear power plants are prohibited.
 - (c) The decommissioning trust agreement for MPS Unit No. 3 must provide that no disbursements or payments from the trust, other than for ordinary administrative expenses, shall be made by the trustee until the trustee has first given the Director of the Office of Nuclear Reactor Regulation 30 days prior written notice of payment. The decommissioning trust agreement shall further contain a provision that no disbursements or payments from the trust shall be made if the trustee receives prior written notice of objection from the NRC.
 - (d) The decommissioning trust agreement must provide that the agreement can not be amended in any material respect without 30 days prior written notification to the Director of the Office of Nuclear Reactor Regulation.

PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

ACTION: (Continued)

| Inoperable Equipment | Required ACTION |
|--|---|
| e. Three auxiliary feedwater pumps in MODE 1, 2, or 3. | <p>e.</p> <p>----- NOTE -----</p> <p>LCO 3.0.3 and all other LCO required ACTIONS requiring MODE changes are suspended until one AFW pump is restored to OPERABLE status.</p> <p>-----</p> <p>Immediately initiate ACTION to restore one auxiliary feedwater pump to OPERABLE status.</p> |

SURVEILLANCE REQUIREMENTS

4.7.1.2.1 Each auxiliary feedwater pump shall be demonstrated OPERABLE:

- a. At the frequency specified in the Surveillance Frequency Control Program by:

----- NOTE -----

Auxiliary feedwater pumps may be considered OPERABLE during alignment and operation for steam generator level control, if they are capable of being manually realigned to the auxiliary feedwater mode of operation.

Verifying each auxiliary feedwater manual, power operated, and automatic valve in each water flow path and in each required steam supply flow path to the steam turbine driven auxiliary feedwater pump, that is not locked, sealed, or otherwise secured in position, is in the correct position.

- NOTE -----
- b. Not required to be performed for the steam turbine driven auxiliary feedwater pump until 24 hours after reaching 800 psig in the steam generators.
-

Verify the developed head of each auxiliary feedwater pump at the flow test point is greater than or equal to the required developed head when tested pursuant to Specification 4.0.5. The provisions of Specification 4.0.4 are not applicable to the steam turbine driven auxiliary feedwater pump for entry into MODE 3.



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 263

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-49

DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-423

MILLSTONE POWER STATION, UNIT NO. 3

1.0 INTRODUCTION

By letter dated August 19, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14237A099), as supplemented by letter dated January 26, 2015 (ADAMS Accession No. ML15033A381), Dominion Nuclear Connecticut, Inc. (DNC, the licensee), submitted a request for changes to the Millstone Power Station, Unit No. 3 (MPS3), Technical Specifications (TSs). The amendment revises MPS3 TS 3/4.7.1.2, "Auxiliary Feedwater System," Surveillance Requirement (SR) 4.7.1.2.1.b. by replacing the surveillance frequency and acceptance criteria for the Auxiliary Feedwater (AFW) pumps with a reference to the Inservice Testing (IST) Program (TS 4.0.5) for the specific pump testing acceptance criteria and the surveillance frequency, which is consistent with the other pump and valve surveillance requirements in the TS. The amendment also adds information on suitable plant conditions for performance of the steam turbine driven AFW pump surveillance.

The supplemental letter dated January 26, 2015, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* (FR) on May 26, 2015 (80 FR 30099).

2.0 REGULATORY EVALUATION

The regulatory requirements and guidance which the NRC staff considered in assessing the proposed TS change are as follows:

Section 182a of the Atomic Energy Act (the Act) requires applicants for nuclear power plant operating licenses to include TSs as part of the application. The TSs ensure the operational capability of structures, systems, and components that are required to protect the health and safety of the public. The Commission's regulatory requirements related to the content of the TSs are contained in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36.

In 10 CFR 50.36(a)(1), it states:

A summary statement of the bases or reasons for such specifications ... shall also be included in the application, but shall not become part of the technical specifications.

In 10 CFR 50.36(c), it requires that the TSs include items in the following specific categories: (1) safety limits, limiting safety systems settings, and limiting control settings (50.36(c)(1)); (2) Limiting conditions for operation (50.36(c)(2)); (3) SRs (50.36(c)(3)); (4) design features (50.36(c)(4)); and (5) administrative controls (50.36(c)(5)).

In determining the acceptability of revising MPS3 TSs, the NRC staff used the accumulation of generically approved guidance in NUREG-1431, Revision 4, "Standard Technical Specifications [STS], Westinghouse Plants," dated April 2012. Licensees may revise the TS to adopt current improved STS format and content provided that plant-specific review supports a finding of continued adequate safety because: (1) the change is editorial, administrative or provides clarification (i.e., no requirements are materially altered), (2) the change is more restrictive than the licensee's current requirement, or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework, and additional specialized guidance, are discussed in Section 3.0 in the context of specific proposed changes.

3.0 TECHNICAL EVALUATION

3.1 Proposed Changes

The licensee proposed removing acceptance criteria for the AFW pumps from SR 4.7.1.2.b as well as adding a note that would not require the SR to be performed for the steam driven AFW pump until 24 hours after pressure in the steam generators reaches 800 psig [pounds per square inch gauge].

The current SR format and text are as follows:

At least once per 92 days on a STAGGERED TEST BASIS, tested pursuant to Specification 4.0.5, by:

- 1) Verifying that on recirculation flow each motor-driven pump develops a total head of greater than or equal to 3385 feet;
- 2) Verifying that on recirculation flow the steam turbine-driven pump develops a total head of greater than or equal to 3780 feet when the secondary steam supply pressure is greater than 800 psig. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.

The licensee proposed the following format and text for the SR:

----- NOTE -----

Not required to be performed for the steam turbine driven AFW pump until 24 hours after reaching 800 psig in the steam generators.

Verify the developed head of each AFW pump at the flow test point is greater than or equal to the required developed head when tested pursuant to Specification 4.0.5. The provisions of Specification 4.0.4 are not applicable to the steam turbine driven AFW pump for entry into MODE 3.

3.2 NRC Staff Evaluation

The NRC staff reviewed the justification for the SR changes provided by the licensee. The licensee stated in their application that the proposed change to SR 4.7.1.2.b is consistent with NUREG-1431 format for STS SR 3.7.5.2. The evaluation below will support the staff's conclusion that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

The MPS3 AFW system consists of two motor-driven 50 percent capacity AFW pumps, one turbine-driven 100 percent capacity AFW pump, and the associated piping and valves necessary to connect the demineralized water storage tank to the pump suction, as well as the pump discharges to the feedwater system. The pumps are used as an emergency source of feedwater supply to the steam generators and are required to ensure safe shutdown in the event of loss of power or function as an engineered safeguards system to remove core decay heat. The pumps are on standby service during normal plant operation.

The AFW pumps are subject to periodic testing and acceptance criteria in accordance with the IST Program. Compliance with the IST Program is required by TS Section 4.0.5 and 10 CFR 50.55a. The IST Program includes specific acceptance criteria for pumps that are not subject to arbitrary changes. The licensee stated that the IST acceptance criteria for the AFW pumps (e.g., developed head and pump flowrate) are based on the design basis requirements.

Regulations in SR 4.7.1.2.1.b state that the motor-driven AFW pumps (3FWA*P1A and 3FWA*P1B) are tested to verify that on recirculation flow each pump develops a total head greater than or equal to 3,385 feet, and the turbine-driven AFW pump (3FWA*P2) is tested to verify that on recirculation flow it develops a total head greater than or equal to 3,780 feet. In letter dated January 26, 2015, in response to a request for additional information, the licensee stated that the minimum head of 3,385 feet for the motor-driven AFW pumps equates to a differential pressure of 1467.3 pounds per square inch differential (psid), and the minimum head of 3,780 feet for the turbine-driven AFW pump equates to a differential pressure of 1,638.7 psid. The licensee also stated that for the IST quarterly test on recirculation flow, the differential pressure reference values for the pumps are: 1,489.6 psid for AFW pump 3FWA*P1A, 1,480.5 psid for AFW pump 3FWA*P1B, and 1,736.9 psid for AFW pump 3FWA*P2. These differential pressure values are greater than the differential pressure values associated with the minimum total head values in SR 4.7.1.2.1.b.

The NRC staff concluded that performing the IST for these pumps instead of the surveillance tests is acceptable because the IST differential pressure reference values bound the pump total head requirements in SR 4.7.1.2.1.b.

Along with the proposed TS changes, the licensee submitted TS Bases changes corresponding to the proposed TS changes. The NRC staff noted that TS Bases changes are consistent with the proposed TS changes and provide the purpose for each requirement in the specification consistent with the Commission's Final Policy Statement on TSs Improvements for Nuclear Power Reactors dated July 22, 1993 (58 FR 39132).

The NRC staff determined that the proposed changes to SR 4.7.1.2.b is consistent with NUREG-1431 format for STS SR 3.7.5.2. Based on the requirements of 10 CFR 50.36 and the IST Program, the staff concludes that removing the AFW pump acceptance criteria from SR 4.7.1.2.1.b is acceptable. Based on the need to allow time to develop sufficient steam pressure to perform the AFW pump test, the staff concludes that adding the Note to SR 4.7.1.2.b is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified on June 26, 2015, of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* (FR) on May 26, 2015 (80 FR 30099). The amendment also involves changes to administrative procedures or requirements and makes editorial, corrective or other minor revisions. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations; and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Hamm

Date: July 28, 2015

July 28, 2015

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear Connecticut, Inc.
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION, UNIT NO. 3 - ISSUANCE OF AMENDMENT
TECHNICAL SPECIFICATION CHANGES TO AUXILIARY FEEDWATER
SYSTEM (TAC NO. MF4692)

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Sincerely,

/RA/

Richard V. Guzman, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures:

1. Amendment No. 263 to NPF-49
2. Safety Evaluation

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| DATE | 7/06/2015 | 7/28/2015 | 7/07/2015 | 7/08/2015 |
| OFFICE | NRR/DSS/STSB/BC | OGC | NRR/DORL/LPLI-1/BC(A) | NRR/DORL/LPLI-1/PM |
| NAME | RElliott* | CKanatas | MDudek | RGuzman |
| DATE | 5/29/2015 | 7/21/2015 | 7/22/2015 | 7/28/2015 |

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