

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

Title: **BRIEFING ON SYSTEM RELIABILITY STUDIES -
PUBLIC MEETING**

Location: **Rockville, Maryland**

Date: **Wednesday, February 7, 1996**

Pages: **1 - 61**

SECRETARIAT RECORD COPY

ANN RILEY & ASSOCIATES, LTD.

1250 I St., N.W., Suite 300
Washington, D.C. 20005
(202) 842-0034

DISCLAIMER

This is an unofficial transcript of a meeting of the United States Nuclear Regulatory Commission held on February 7, 1996 in the Commission's office at One White Flint North, Rockville, Maryland. The meeting was open to public attendance and observation. This transcript has not been reviewed, corrected or edited, and it may contain inaccuracies.

The transcript is intended solely for general informational purposes. As provided by 10 CFR 9.103, it is not part of the formal or informal record of decision of the matters discussed. Expressions of opinion in this transcript do not necessarily reflect final determination or beliefs. No pleading or other paper may be filed with the Commission in any proceeding as the result of, or addressed to, any statement or argument contained herein, except as the Commission may authorize.

1 UNITED STATES OF AMERICA
2 NUCLEAR REGULATORY COMMISSION

3 *****

4 BRIEFING ON SYSTEM RELIABILITY STUDIES

5 *****

6 PUBLIC MEETING

7 *****

8
9 U.S. Nuclear Regulatory Commission
10 One White Flint North
11 Rockville, Maryland
12 Wednesday, February 7, 1996
13

14 The Commission met, pursuant to notice, at 10:00
15 a.m., the Honorable Shirley A. Jackson, chairman, presiding.
16

17
18 COMMISSIONERS PRESENT:

19 SHIRLEY A. JACKSON, CHAIRMAN

20 KENNETH C. ROGERS, COMMISSIONER
21
22
23
24
25

ANN RILEY & ASSOCIATES, LTD.
Court Reporters
1250 I Street, N.W., Suite 300
Washington, D.C. 20005
(202) 842-0034

1 STAFF AND PRESENTERS SEATED AT THE COMMISSION TABLE:

2

3

JOHN C. HOYLE, SECRETARY

4

MARTIN MALSCY, DEPUTY GENERAL COUNSEL

5

JAMES TAYLOR, EDO

6

EDWARD JORDAN, Director, AEOD

7

ASHOK THADANI, NRR

8

PATRICK BARANOWSKY, Chief, Reliability and Risk

9

Assessment Branch, AEOD

10

STEVEN MAYS, Section Chief, Risk Assessment

11

Branch, AEOD

12

13

14

15

16

17

18

19

20

21

22

23

24

25

ANN RILEY & ASSOCIATES, LTD.
Court Reporters
1250 I Street, N.W., Suite 300
Washington, D.C. 20005
(202) 842-0034

P R O C E E D I N G S

[10:00 a.m.]

CHAIRMAN JACKSON: Good morning, ladies and gentlemen. The purpose of today's meeting is for the staff to brief the Commission on the use of operational data to estimate the unavailability of risk significant systems in U.S. power plants. Even though the overall trends have levelled off in recent years, the performance of the nuclear industry has shown a decline in the occurrence of operational events. As the number of events decrease, we must look for alternative performance measures to monitor and track plant performance.

In addition there is a need for predictive risk informed measures to the extent possible. As an alternative approach the staff is pursuing system reliability and unavailability studies for selected risk significant systems. As part of today's briefing the staff will describe some of its reliability studies, as well as any generic implications or conclusions derived from these studies.

I am also interested in knowing what progress has been made to add or to replace some of the conventional performance indicators, with risk informed indicators. As you know I have been a proponent of risk informed performance based regulation, which allows the NRC to focus,

1 as well as licensees, on the most safety significant aspects
2 of reactor operations. I recently, in fact, gave two PRA
3 seminars, one at MIT and the other at the University of
4 Maryland, where I actually used results from the staff's
5 HPCI study to illustrate some the basic PRA concepts. So
6 you better watch me. So I may have a few questions for you
7 on these concepts.

8 I understand to copies of the presentation slides
9 are available at the entrance to the room.

10 Commissioner Rogers, do you have anything?

11 COMMISSIONER ROGERS: Not at this time.

12 CHAIRMAN JACKSON: If not, Mr. Taylor, you and
13 your team may proceed.

14 MR. TAYLOR: Good morning.

15 The purpose of this briefing is to convey the
16 staff's results of its reliability analysis of two safety
17 related systems at U.S. plants. The staff will explain how
18 these data fit into the overall probable risk assessment
19 program plan and specific AEOD activities in support of that
20 plan.

21 With me at the table are Ashok Thadani of NRR, Ed
22 Jordan, Par Baranowsky and Steve Mays of AEOD. Ed Jordan
23 will continue.

24 MR. JORDAN: Good morning.

25 CHAIRMAN JACKSON: Good morning.

1 MR. JORDAN: The briefings that we have given over
2 the past year connect together and I would like to try to
3 make that connection for you. The April 26th briefing on
4 the development of the proposed rule for reliability data is
5 the source of credible performance data that we expect to
6 use for this type of work and that rulemaking has gone
7 through the review process and is about ready for
8 publication.

9 We briefed the Commission in August of 1995 on a
10 transition of the existing set of checked performance
11 indicators to make them more risk based and to reduce the
12 costs of compiling the data. My staff has provided me with
13 a program plan on that ongoing development. At the time we
14 briefed the Commission we were proposing a discontinuity or
15 considering a discontinuity in performance indicators while
16 we developed the risk based. We are no longer doing that
17 as we described and we have an orderly transition, but our
18 goal clearly is to shift towards the risk based.

19 In November we explained the role of the revolving
20 accident sequence precursor program as an element of the
21 transition and as a part of describing individual and
22 industry safety performance in defensible risk terms.

23 Today we are here to provide a reliability and
24 availability results for emergency diesel generators and BWR
25 high pressure injection systems. Using available data

1 sources and state-of-the-art methods. This is the same
2 methodology we plan to use once we have the routine
3 collection of plant specific data. Although today we are
4 data limited, there are already some useful insights from
5 this analysis. The final slide will briefly describe our
6 program plan for this activity through 1997 and beyond.

7 Mr. Steve Mays will conduct the briefing.

8 MR. BARANOWSKY: Although I will give an
9 introductory discussion.

10 MR. JORDAN: I am sorry.

11 MR. BARANOWSKY: It is really an overview to put
12 the system reliability studies into context with our whole
13 program to use reliability and risk analysis to look at
14 operating experience.

15 [Slide.]

16 MR. BARANOWSKY: We are using, of course, insights
17 from PRA to identify what we should look at and then we are
18 using PRA methods and extensions of PRA methods to perform
19 the analysis. Now, we know that operating experience alone
20 can't give us the full risk picture, but certainly using
21 risk and reliability techniques to analyze operating
22 experience is, I think, a good way to focus your analysis
23 and evaluation of operational data. So that is where we are
24 coming from, at least philosophically.

25 [Slide.]

1 MR. BARANOWSKY: The overall program's objectives
2 are to use the operating experience to assess and trend risk
3 indicators, to compare results of this analysis with IPEs
4 and PRAs, to identify the technical insights that come out
5 of those analysis, especially those that are important with
6 regard to equipment reliability and to provide the insights
7 to both industry and to NRC programs.

8 As sort of an additional added extra that comes
9 along with this we will be providing scrutable data sources
10 and failure rate estimates that can be used by the NRC for
11 reliability and risk applications.

12 CHAIRMAN JACKSON: You did say scrutable?

13 MR. BARANOWSKY: Scrutable, as opposed to
14 inscrutable. I think that is an important part of our
15 analysis, is to make it all tractable in some way.

16 Let me move to view graph three, please?

17 [Slide.]

18 MR. BARANOWSKY: As I had mentioned we did talk to
19 you in August and we provided a picture of our thinking on
20 how we were looking at industry wide risk and decomposing it
21 into its elements, both plant specific and also in terms of
22 the elements that go into plant specific risk. This chart
23 is a condensed summary of what we told you at that time and
24 it shows a train of thought that goes and connects the
25 system reliability studies that we are going to talk about

1 today to core damage frequency plant risk and ultimately
2 some indication of industry risk.

3 The elements of this diagram, then, are the
4 building blocks for what we think would be the risk based
5 indicators that we would develop in the future.

6 Let me move to view graph four?

7 [Slide.]

8 MR. BARANOWSKY: This view graph shows the full
9 set of activities that we either have planned or ongoing
10 right now that are relevant to our perceptions of how we
11 should go about analyzing the operational data. We talked
12 to you, I think it was in November, about the accident
13 sequence precursor program, so I won't mention too much
14 about that now, except to say that both plant specific and
15 industry wide risk perspectives can be derived from that
16 program.

17 We are also in the process of compiling a
18 reference document for initiating events, which I am not
19 going to talk about today, but that is an important part, of
20 course, of the whole accident sequence progression.

21 The system reliability studies that we are going
22 to talk about today, were also discussed with the ACRS in
23 November and I think they were received, the presentation
24 was received, favorably by the ACRS.

25 If I could go to number five, please?

1 [Slide.]

2 MR. BARANOWSKY: Recently, we distributed a common
3 cause failure database and some generic analysis that we had
4 been performing for review, trial use and comment. We sent
5 this to the staff and we also sent it to INPO. The database
6 contains a limited amount of proprietary information from
7 the Nuclear Plant Reliability Data System, and we have asked
8 INPO if we could distribute this information to NPRDS users
9 that already have access to proprietary data, because I
10 think it is the most comprehensive set of data on common
11 cause failure that is available now in the world.

12 In August we talked about performance indicators
13 and moving towards risk and reliability data. We still have
14 that activity going on in both the classical or traditional
15 sense that they are currently produced in and part of this
16 work today is an indication of where we are heading in the
17 future.

18 Lastly, as I said, what goes along with all this
19 work is, you have to have data systems to support the
20 analysis. We did talk about the sequence coding and surge
21 system and NPRDS earlier, and also, as you are aware from
22 recent activity, we have been promulgating the reliability
23 data rule which would supplement our data in a way that
24 would allow us to perform more complete analysis of the
25 types of systems that we are talking about today.

1 CHAIRMAN JACKSON: Before you go on let me ask you
2 a quick question under the common cause failures. Where do
3 things stand or how are they progressing with respect to the
4 international exchange effort on common cause?

5 MR. BARANOWSKY: We have a couple of meetings that
6 we have attended with some European countries and we are now
7 trying to agree on a format for exchanging data. One of the
8 things I want to make sure of is that it is a two way street
9 so that we get something too. It has been a very good
10 cooperative activity, because of the importance of common
11 cause failure for highly redundant systems like three and
12 four trains, the Europeans are quite interested in this
13 since they have many multi train systems. I am hoping that
14 this summer we can have an agreement on how to exchange that
15 data and what protocols would be involved.

16 CHAIRMAN JACKSON: That would be very good. I
17 noted that you said that we had a comprehensive data base,
18 but the flow the other way is useful.

19 MR. BARANOWSKY: Yes.

20 MR. THADANI: If I may add to that, this is
21 clearly one of the limitations in the technology today and
22 for these selected components this is an important piece of
23 work. We are also looking at that to see, in a number of
24 day-to-day decisions that we make, how well we have captured
25 this common cause failure data and information. So that is

1 one piece. The second piece is, as you noted, the PRA
2 implementation plan and one element in that plan is the
3 framework and part of the framework is to make sure we have
4 methods data. So this information will be focused in that
5 activity as well.

6 [Slide.]

7 MR. BARANOWSKY: So that is the overview of our
8 operational data activities that are risk and reliability
9 based.

10 Now, I would like to move into the topic for
11 today, which primarily has to do with the system reliability
12 studies that we have been working on. The intent of that
13 work is to evaluate the reliability and provide engineering
14 insights for risk important systems, based on operating
15 experience. We have a set of objectives which includes
16 trying to use actual demands or demands that are as close as
17 possible to actual demands, along with the associated
18 failures or unavailabilities associated with those demands
19 to estimate reliability.

20 One reason for doing that is so that we cannot
21 worry so much about problems in using piece part data in
22 building up a model, especially when some of the
23 interactions that might occur between equipment might not
24 necessarily be easy to detect from some of the piece part
25 data until one has a full integrated type of demand. In

1 some cases demands that are placed on equipment are not all
2 equivalent in terms of the severity of the conditions that
3 the equipment sees. So from the data that was available to
4 us through LER we want to focus on actual demands.

5 Other elements of our objectives include, we
6 wanted to look at trends and the uncertainty in trends so
7 that we wouldn't be fooled by data fluctuations. Then, of
8 course, compare our findings with the kinds of results that
9 both PRAs and IDEs have indicated, look for any plant
10 specific differences and see what the engineering insights
11 that fall out of this might be.

12 Currently we are evaluating the reliability of
13 systems at power and if we are to look at shutdown
14 reliability and availability, that would have to come, I
15 think, in the future when we have a better picture of the
16 risk significant elements that are associated with that
17 condition.

18 If I could move to view graph seven.

19 [Slide.]

20 MR. BARANOWSKY: We selected a number of systems
21 for this program based on our observations in PRA,
22 considering both their risk reduction and, to some extent,
23 their risk achievement potential. So the list I show on
24 this view graph indicates that for boiling water reactors we
25 have a number of the high pressure coolant injection and

1 cooling systems that we are looking at. For PWRs we are
2 looking at auxiliary feed water and high pressure safety
3 injection. Then there are some systems, like low pressure
4 injection, reactor trip systems and emergency diesel
5 generators, that we are looking at for all plants.

6 Once a base line study is completed, then future
7 updates can be prepared periodically and with a lot less
8 effort once we have established the approach and how to sort
9 the data.

10 So, with that introduction having been completed,
11 I will turn the hard work over to Steve Mays, our Section
12 Chief for the Reliability and Risk Analysis Section, who
13 will discuss the methods and some of our results.

14 MR. MAYS: Thank you.

15 It is a pleasure to be back again. As it has been
16 commented on before we started today, there is a large
17 package of slides here. I think the number of slides are
18 inverse proportional to salary.

19 CHAIRMAN JACKSON: Yours or ours?

20 MR. MAYS: Mine.

21 [Laughter.]

22 MR. MAYS: So I will try to get through these as
23 quickly as I can, but I want to make sure we have the
24 opportunity to discuss any issues that come out.

25 [Slide.]

1 MR. MAYS: The first slide we are going to talk
2 about here is the methodology overview and what I am trying
3 to show you here is, we have got a well thought out process.
4 It can be applied to look into the reliability of any
5 system. This just isn't, how many things do we have, can we
6 put together a report type of process. So we have looked at
7 this stuff to come up with standard methods to use things.
8 We are doing very detailed evaluation of the events and we
9 are using a risk perspective when we are doing that and that
10 will become clear as we talk about some more things later
11 on.

12 So we have looked at this stuff to come up with
13 standard methods to use things. We are doing very detailed
14 evaluation of the events and we are using a risk perspective
15 when we are doing that and that will become clear as we talk
16 about some more things later on. We are having a rigorous
17 mathematical treatment of the data and we have detailed
18 analysis reviews, including peer review both as part of our
19 contractor reports that are being done as well as internal
20 to the agency and also sending information to the industry
21 as part of this process.

22 CHAIRMAN JACKSON: When you say independent peer
23 review, is that through the usual publication route or do
24 you actually have --

25 MR. MAYS: We have a panel of resource experts

1 that we have contracted outside of the normal work on this
2 to come in and review the work and give us insights and
3 comments and we have used that as part of our process as
4 well as meeting with other people in the agency. We sending
5 our reports, as we get them, out for comment to industry
6 groups as well.

7 CHAIRMAN JACKSON: Have you had any interactions
8 with the ACRS?

9 MR. MAYES: We have made the presentation last year
10 at the ACRS and gave them basically the HPCI presentation
11 which you are going to see in this. That was fairly well
12 received.

13 Going on to the next slide.

14 [Slide.]

15 MR. MAYES: On more detail about the methodology
16 approach, you will see this pattern develop in the slides
17 that follow as well as in the reports that we provide. We
18 start with defining what our system boundary is, or defining
19 we are going to get data and what data we are going to
20 consider. Then we get the information from licensee reports
21 and characterize it.

22 There are three sub-bullets under the
23 characterization heading. We found that data tends to be
24 reported as either inoperabilities and then there are among
25 those inoperabilities where you actually lose the safety

1 function. Then there is another subset of that which is
2 safety functions are lost, where we can corresponding what
3 counts excesses and demands. I will talk about that a
4 little bit more on the next line.

5 We also then determined the failure probabilities
6 and we do this using Bayesian techniques and we wanted to
7 characterize the uncertainties, not just come up with
8 maximum likelihood estimators. We combine that information
9 in simplified fault trees to produce our results. Then we
10 do our trending and our comparison analysis after that. You
11 will see examples of all of these as we go through the
12 slides.

13 [Slide.]

14 MR. MAYS: This is a slide that I want to make
15 some emphasis on about the data set relationships. That is
16 slide ten.

17 If you look at this thin diagram, which we kind of
18 refer to it as the reliability egg when we talk about it,
19 there are three areas in which failure in information comes
20 in.

21 The first one is, cases where it is declared to be
22 inoperable in an LER and not all inoperabilities are equal.
23 Sometimes you will get inoperability because somebody hasn't
24 done their surveillance testing in the required interval.
25 That doesn't mean the system couldn't have worked. It means

1 we had a technical inoperability. So we have to go through
2 and screen the inoperabilities to get down to the cases in
3 that Area B there, where the safety function of the system
4 is actually lost, it is not going to work because it is just
5 broken.

6 This is an important point also. When you are
7 doing reliability analysis, it is a function of both the
8 failures and the successes. So having failure information
9 is a necessary contributor, but it is not a sufficient
10 contributor in order to do reliability. So we have to go
11 down to a subset of that data. That data is failures where
12 the function is lost, but where we can also have a
13 corresponding demand count, so that we can count the
14 successes as well as the failures. That is to have us have
15 an unbiased sample of these data so that we can produce
16 reliability analysis that make sense.

17 Of course as you get to smaller and smaller data
18 you have sparser data and that causes you to have a greater
19 uncertainty balance associated with your estimates.

20 [Slide.]

21 MR. MAYS: Moving down to the next thing, I want
22 to talk a little bit about techniques that are employed
23 throughout these analysis. We are using Bayesian and
24 updating techniques to estimate our parameters for
25 reliability and the associated uncertainty. There are three

1 basic methods of using Bayes techniques that we use and I
2 will discuss each of them a little bit for people.

3 The simple Bayesian method is where we start with
4 a noninformative prior and take that information and use the
5 actual plant failures and success counts to come up with an
6 estimate of the uncertainty. We do that because the
7 noninformative prior allows us to have uncertainty intervals
8 that are predominantly due to the data density of the
9 information network we are gathering, rather than some other
10 artificial constraint.

11 COMMISSIONER ROGERS: Could you just explain that
12 term a little bit? When I read that I couldn't think of
13 anything except somebody who lived in a monastery.

14 MR. MAYS: Actually, Reverend Bayes did live in a
15 monastery. As you know, he was both a theologian and a
16 mathematician in the 1700s. The Bayesian approach to
17 updating is an inference approach that says that you can
18 determine the likelihood of some event based on your prior
19 knowledge as well as your current information that you know.
20 A mathematical technique was developed by Reverend Bayes and
21 published in about 1764, I believe, two years after he died,
22 actually.

23 The noninformative prior means that you start off
24 with a distribution that does not give you any specific
25 information about what the failure probability is, It is

1 equally distributed between zero and one, which means is
2 basically has a mean value of .5. Subsequently, because of
3 the spread and the uncertainty, when you do the update, the
4 resulting posterior distribution has an uncertainty that is
5 primarily a result of the density of the data you have.

6 For example, if you had one failure in ten events
7 you would have a mean estimate of .1 and you would have an
8 uncertainty associated with that. If you had 100 failures
9 in 1,000 estimates you would have the same mean estimate,
10 but because you had a greater data density, the uncertainty
11 around that estimate would be much smaller. So this is the
12 technique for being able to do that properly.

13 CHAIRMAN JACKSON: You know, in doing these
14 calculations there is some, I guess, choice or flexibility
15 in terms of probability distributions. I am familiar with
16 the gamma distributions and the beta distributions. So what
17 you are talking about really has to do with the way these
18 parameters get fixed and that not only tells you what the
19 means are, or tied to the means, but tells you what the
20 confidence in them would be. I guess, I was going to ask
21 this later, once you talked about the HPCI system and the
22 diesel generators, but more HPCI because that is the one
23 that I am more familiar with. That is, how sensitive are
24 the results to distributional assumptions, in terms of
25 either the type of distribution that is chosen or how the

1 parameterization of the given distribution is determined?
2 What I am trying to say is, if one went from using the sort
3 of standard gamma/beta distributions that people use, to
4 Wyble or some other distribution, the question is, how much
5 change?

6 MR. MAYS: What change to the results are going to
7 take place? That is a good question and, quite frankly, we
8 haven't done a sensitivity analysis of that type. We used
9 beta distributions for demand type information and gammas
10 for hourly type distributions. That is a fairly standard
11 process and we didn't go back and look at Wyble or other
12 distributions as a function of that. I would have to get
13 back to you. We haven't gotten to that point.

14 CHAIRMAN JACKSON: No, that was a question. I was
15 just wondering. Maybe if you could when you go through with
16 talking about the HPCI studies, given the distributions you
17 did use, how sensitive are they to how they are
18 parameterized, but I'll just listen.

19 MR. MAYS: We did look at how to look at the
20 uncertainty in a way that would maximize it, for instance.
21 Then we also looked at differences between using simple
22 Bayes and empirical Bayes. So we did a number of
23 calculations and we also asked ourselves does this result
24 pass the sanity test, because you can put in any prior
25 distribution that you want and, therefore, you can make the

1 result come out any way you want. So what you have to come
2 out with is a result that makes some sense, too, and we did
3 some of those kinds of analyses.

4 CHAIRMAN JACKSON: Okay.

5 MR. MAYS: The next technique that we used in the
6 analyses was empirical Bayes approach. Basically what we
7 did here was we started with a population information which
8 would generally come from the simple Bayes analysis of an
9 individual group and then we would discover and look for
10 variations within the group that would indicate that there
11 was some significant plant-to-plant or failure-mode-to-
12 failure-mode or other types of variation within that
13 population. When we found evidence of that information we
14 would go and use an empirical Bayesian update to do the
15 plant specific or the year-to-year variation. So we start
16 with the population information and then we take the plant
17 specific information and do an update of that population
18 information to come up with the plant specific estimates.

19 One of the things that we are trying to do here is
20 do trending information as well. We found that the
21 constrained noninformative prior approach, which is
22 described in the reports, was what we wanted to do there,
23 because if you use either the simple Bayes or the empirical
24 Bayes you can get some misleading information about either
25 the mean estimates or the uncertainties when you do

1 trending. You start with a population in information which
2 can have a fairly tight uncertainty band associated with it,
3 because there is a fairly high density for seven years worth
4 of operating experience. If you then want to go back and
5 look at the variation in the performance on a year-to-year
6 basis, each year has a very limited amount of operating
7 experience compared to the total. So if you used an
8 empirical Bayes approach you would be artificially
9 constrained by that uncertainty from that data density.

10 So what we would do is we would use the mean
11 estimate that came out of the population results and then we
12 would diffuse the prior, so that we would have the maximum
13 uncertainty associated around with that mean, so the
14 subsequent updates would be a function of the density of the
15 year-to-year data rather than a function of the density of
16 the total. So we used that process to do trending analysis.

17 With that brief introduction to Bayesian
18 techniques, incomplete as it is, I would like to go on and
19 talk about the high pressure coolant injection results.

20 [Slide.]

21 MR. MAYS: The first view graph here is just a
22 picture of the system with dotted lines around the areas for
23 which failures were not included in the data analysis and
24 that is a fairly straightforward look at what we did.

25 [Slide.]

1 MR. MAYS: The next slide illustrates the
2 reliability evaluation model we did. There are a couple of
3 points I would like to make about this because they are
4 important to how we do this in this evaluation as well as in
5 the diesel generators. This isn't a standard fault tree.
6 It doesn't list every component in the system. This is a
7 hierarchical model of how the system would operate and we
8 only break it down to the level in which we have data that
9 indicates there is a significant difference in the
10 population of the data and how they perform.

11 We start off with a basic system model, it says.
12 It doesn't start, it doesn't run or it was out of service
13 and wasn't available, that is our basic starting point. In
14 the HPCI case we looked at the data and found we had to
15 break that down further, because we found differences in the
16 population information about injection valve operation as
17 opposed to the rest of the valves and pumps in the system
18 and we found differences between the run information and the
19 start information. So we had to break the model a little
20 further down than the more basic one, but that is the basic
21 concept that we use for doing this.

22 Our concept when we pool information together in
23 the populations requires three things. It has to have the
24 same statistical population when you look for the failure
25 rates. We also have to have similar demand requirements for

1 groups of components. So if something has a tremendously
2 different demand than another one, you wouldn't group them
3 even if they statistically looked like they had the same
4 failure rate.

5 The other one is, we have to see similar failure
6 experience so that the nature of the failures in a
7 population appear to be common to one another. So those are
8 the requirements we have for pooling or breaking apart
9 information from the data.

10 [Slide.]

11 MR. MAYS: The next slide talks about the actual
12 data information that we derived from the failures and
13 success information we were able to find and the associated
14 Bayesian intervals and this is the information that gets fed
15 into the basic events in this reliability model to come up
16 with the overall system unreliability estimates.

17 CHAIRMAN JACKSON: You are using gamma?

18 MR. MAYS: The starting point on these was the
19 simple Bayes with noninformative prior for these estimates.
20 So you can see from this, for example, that we had about 63
21 actual unplanned demands for the HPCI system over the seven
22 year period to actually start and inject into the reactor.
23 During that time we had one maintenance out of service event
24 where it wasn't available to do so.

25 The significant other differences I would like to

1 pull out in this, if you notice, under the failure to start,
2 other than the injection valve, there is a significantly
3 higher number of demands. That is because we were able to
4 determine that the cyclic, once every 18-month, test of the
5 system where you put an actual emergency system on to the
6 system and start it up into full flow through the re-circ
7 line, was very similar to the kinds of things that were
8 happening from an engineering and a statistical standpoint,
9 from the actual unplanned demand. So that gave us more data
10 that was appropriately poolable. So under those conditions
11 we have a higher number of demand and we have failures that
12 are listed for that set and the Bayesian intervals are on
13 the right-hand side.

14 There were a couple of other things we found as
15 well. We found that the failure to start associated with
16 the injection valve represented a different population so we
17 separated that.

18 A couple of other points that are important are,
19 we looked at the fact that the operating experience told us
20 that many of these failures had some recovery probability
21 associated with them and recovery in this case was recovery
22 in a PRA sense. The recovery was fairly quick, it was from
23 the control room, it would be part of a simple process.

24 For example, the recoveries associated with
25 failure to start in this case, or three of the cases, where

1 the turbine control valve was operating erratically so they
2 put it in manual and that solved the problem. That was done
3 from the control room very quickly and it didn't cause the
4 system to trip off. So we also included in the model the
5 information about whether or not the failures were
6 recoverable based on the operating experience and included
7 that as well.

8 We had failure to run information, although most
9 of the runs of the HPCI system in the operating experience
10 were fairly short duration. So we used basically a demand
11 probability type evaluation for failure to run on the HPCI
12 system.

13 The last one at the bottom of the page was
14 something we found from the operating experience and was
15 interesting to us as well. We found that there was a
16 different population and a different experience associated
17 with the injection valve reopening.

18 Some HPCI systems get used in a pressure control
19 mode or an intermittent injection mode, in which it will
20 inject the first time and the injection valve is closed the
21 recirc valve is opened and then will subsequently be
22 reopened later. We found that that had a different
23 reliability associated with that.

24 We also looked at the PRAs and found that most of
25 the PRAs model the HPCI system in a single injection mode.

1 So when you go to do comparisons you have to make sure you
2 have the right comparison information in order to do that.

3 [Slide.]

4 MR. MAYS: So the next slide represents the HPCI
5 system unreliability results that we found from this
6 application of the data. What you can see is that the
7 overall unreliability was about .056 and the major
8 contributors to the unreliability based on the operating
9 experience were maintenance events and failure to run events
10 that were not readily recoverable.

11 Again, there is an issue that comes up here,
12 because of the data density we only had one event in which
13 we had a maintenance out of service out of 63 demands. So
14 that is fairly sparse data, but we were capturing the
15 uncertainty associated with that because we were using the
16 appropriate techniques to indicate that that wasn't very
17 dense data and we were combining this stuff with the
18 uncertainty so that we get an appropriate estimate of the
19 unreliability.

20 Subsequent to that we were looking at trend
21 information about the reliability of the HPCI system. This
22 next view graph shows three different trending efforts that
23 we did.

24 [Slide.]

25 MR. MAYS: The one on the top left is to look at

1 the unplanned demand rates that the plants were experiencing
2 during this time period. The top right is the failure
3 rates, which is the number of failures per year without
4 regard to whether there was a demand. The one at the bottom
5 is the unreliability, where we are pairing up and matching
6 failures with demands and successes to calculate
7 unreliability.

8 The trends that you see for both unplanned demand
9 rates and system failure rates decreases were statistically
10 significant trends, indicating that was not just
11 fluctuations in the data. When you get to the system
12 unreliability calculation you find that the reliability
13 trend is fairly flat and there is no statistically
14 significant difference between what is shown on this diagram
15 and the assumption of a completely constant failure rate
16 over the time period.

17 The unplanned demand rate and the failure rate are
18 similar information to what we have in the PI reports. We
19 report safety system failures and safety system actuations.
20 In the performance indicator reports we had seen both of
21 those indicators going down over previous years. So this
22 information tends to support that the HPCI system was
23 behaving in a similar way that the total population was from
24 the PIs, but the unreliability is saying that in spite of
25 the fact that we are having some of these things go down,

1 the reliability hasn't really been changing over this period
2 of time.

3 The next thing we went to do was take the
4 reliability information that we had from the HPCI report and
5 compare it to what kind of information we were seeing in the
6 PRAs and the IPEs. This block, which is fairly busy, but it
7 was the best way to put the information together completely,
8 I guess. We had to go and look back at our data and look at
9 the information that was in the IPEs and come up with a set
10 of comparable models. So we had to do some manipulation.

11 So the first caveat I want to say is that, if some
12 plant comes in and says, from this graph, you said my HPCI
13 system was such-and-such, but my PRA said it was different,
14 my answer is, you are probably right. Because what we did
15 was, we went into their IPEs and PRAs and pulled out the
16 equivalent failure to start, failure to run and maintenance
17 out of service information from their data and put it into
18 the similar model that we had and did the comparison there.
19 So we would be comparing apples to apples rather than apples
20 to oranges. We found that different plants would have
21 different rules for how they would do things, such as
22 consider recovery, how they would model whether something
23 was a failure or not. So we had to go back in and pull
24 information out of the IPE submittals to do this. So I want
25 to make sure you understand this is a synthesis, not a

1 direct pull out of the unreliability from their PRAs.

2 In doing so, you can see at the bottom of the
3 chart there is the industry population value that we
4 calculated and the dotted line up the chart gives you the
5 mean of that distribution that we calculated. The diamonds
6 on the chart and their bars are the empirical Bayesian
7 update of the plant specific information and the asterisks
8 with the bars are the information derived from the IPEs and
9 PRAs.

10 What we found was that there was fairly good
11 agreement on very many of these comparisons, but we did note
12 that there were several where the mean values either didn't
13 have uncertainties associated with them or both that and the
14 mean values were falling outside of the intervals that we
15 were seeing from the operating experience. What that tells
16 us is that there is something different about what was in
17 there and what we have. That doesn't mean that either one
18 of them are necessarily wrong. It means that there is
19 something that needs to be understood.

20 If you are going to be taking actions in the
21 regulatory sense, that is based on something on PRA
22 associated with those plants, you need to know what the
23 basis for the models and the reliability were. That was
24 beyond the scope of what we could do in this analysis, but
25 at least points us in the right direction if there is an

1 issue that comes up with respect to any of these particular
2 plants or systems.

3 CHAIRMAN JACKSON: So what are your plans relative
4 to, that you are just noting that these differences exist?
5 Have you in any cases been able to discern what the source
6 of the deviation's sources are?

7 MR. MAYES: We haven't gone back and expended a lot
8 of energy trying to determine what the differences were in
9 this analysis. We did in our transmittal letter, to both
10 NRR and research, when we transmitted this report for
11 information and use, note that there were the differences
12 and indicate that for the research people who would be doing
13 IPE reports to use that would be interesting information.
14 For the NRR people who would be making a decision about
15 whether somebody could or couldn't do something based on
16 some risk argument associated with these systems, we felt
17 that it was important to let them know that they had some
18 information additional to use other than just whether or not
19 somebody had done a PRA.

20 I think it is an issue that we haven't decided yet
21 what the overall process is going to be, but we have been
22 talking with the NRR and research people as we get these
23 results and transmit them over as to what the implications
24 of this is.

25 CHAIRMAN JACKSON: Dr. Thadani is preempting me,

1 he knew I would ask him the question.

2 MR. THADANI: Let me comment on that. I think
3 there is a lot of very important information on this just
4 one single chart. First, it raises obvious questions about
5 how the plant specific data are used or generic data are
6 used and how these data are actually manipulated in the
7 models, in these IPEs. I think, in my view, on HPCI there
8 are enough differences here, 7 out of 23 plants are clear
9 outliers. I mean, there is a real question about those
10 plants as the minimum as to how they handle the specific
11 scenarios. HPCI system in these plants is a very important
12 system. So we have got to home in on that issue, as one
13 piece.

14 The second piece is, we use this information and
15 we haven't got to HPCI in a big way, but we did do that with
16 the exterior feedwater system study that AEOD had done
17 earlier, to focus our inspection attention for areas where
18 we saw some problems. So we decided to go do some focused
19 inspections for our auxiliary feedwater systems.

20 Another way we used this information is, if there
21 are plants where there are a number of issues that indicate
22 we need to pay more attention to those plants, this element
23 just adds to that need to focus on some of those plants. In
24 fact, we at the recent senior management meeting there were
25 some plants on this list we talked about and we indicated

1 the need. There is an action item that came out of the
2 senior management meeting that NRR needs to follow up on
3 this issue.

4 This is the issue related to not just HPCI, but
5 HPCI, reactor core isolation cooling system, which is the
6 other high pressure single train system that earlier BWRs
7 have and then the emergency diesel generators. You have to
8 look at these in total, because that is your protection from
9 some scenarios.

10 CHAIRMAN JACKSON: Can I sort of try to, for my
11 own purposes, paraphrase what you said? You are saying that
12 particularly in the cases where the data indicates outliers,
13 that gives you an ability to focus a number of activities?

14 MR. THADANI: Exactly.

15 CHAIRMAN JACKSON: It is both inspection efforts,
16 as well as, where there were plants that were under
17 discussion anyway, to kind of do a cross feed?

18 MR. THADANI: Yes, the sensitivity goes up.

19 [Slide.]

20 MR. MAYS: Going on to the next slide, one of the
21 other things that we did in each of these studies was to
22 look at the trends associated with the age of the plant.
23 What we did here was we mapped out the failures per
24 operating year associated with the plant low power date over
25 this period, as well as the unreliability estimate on a

1 plant specific basis. Then we plotted it out against its
2 low power license data and fitted the trends to it.

3 Neither one of these trends is significant. We
4 found that the unreliability was not significantly different
5 for either the older or the new plants and the failure rates
6 were not significantly different among the older and the
7 newer plants, over the seven year period for which this
8 study data was put together.

9 [Slide.]

10 MR. MAYS: The next slide looks at something we
11 did when we started looking at the nature of the failures
12 and the kinds of breakdown that they have. We found that
13 there were differences between the method of discovery of
14 the failures and what proportions they were coming out at.

15 Doing that, I want to make a caveat in here. The
16 unplanned demand information is about, depending on which
17 system you go to, anywhere from 5 to 20 times less
18 opportunities than there are in the surveillance test and
19 other operational occurrences. So you need to be aware that
20 the percentages here are relative to the number of events in
21 each category.

22 What we did find was that among the unplanned
23 demands for the HPCI system to operate and the failures that
24 occurred during those unplanned demands, they were split
25 about 60/40 between turbine and control valve issues and

1 other valve issue and other parts of the piping system.

2 We had no unplanned demand failures in which the
3 instrumentation and control systems contributed.

4 When we looked at the surveillance test and operational
5 occurrences, the relative nature of the failures found by
6 those particular types of discoveries were different from
7 what we were seeing in unplanned demands. What that
8 indicates to us and we indicated when we transmitted the
9 report was that, it seems to me that there is a potential
10 for looking at what the focus of the inspections are and the
11 other operational occurrences are, with respect to the
12 events that are effecting the reliability.

13 We have been talking with the inspection people as
14 we have gone through this. We don't have enough information
15 just from HPCI and diesels to know whether this is s
16 systemic issue or one that only effects a particular system
17 or a group of systems. We have been meeting with them and
18 going over the results of what we have had so we can try to
19 come up with an understanding of what this stuff means and
20 whether or not there is something we need to do with respect
21 to either our inspection programs or the way we go about
22 looking for failures.

23 CHAIRMAN JACKSON: That is an interesting set of
24 comments, because the thing that would leap out at me, that
25 this is your business, but you look at a surveillance test

1 and the question is, is the surveillance test making the
2 same kind of demand as what might occur in an actual
3 unplanned demand.

4 MR. MAYS: That's true and there are different
5 natures of tests and demands that you can go through. There
6 are tests that determine maximum capability. There are
7 tests to determine just the fact that it is still on an
8 operable state and isn't totally failed and there are tests
9 associated with reliability and they are not always the same
10 thing. So failures associated with those may be telling you
11 different information and that is what we think we are
12 seeing here. That is why it is important to understand what
13 those things are telling us and we are going to be working
14 with the inspection program branch to go over and look at
15 those things.

16 MR. TAYLOR: We worry about that, because the
17 surveillance test is supposed to provide the type of start.
18 Let's not have preconditioning so to speak. Let's face,
19 will the equipment operate? So it is a very interesting
20 line of thought.

21 MR. THADANI: I think Mr. Taylor is exactly right.
22 For HPCI, for example, I think to illustrate this issue,
23 there are basically three surveillance tests that are done.
24 HPCI is a turbine driven pump, single train, and every three
25 months you are supposed to make sure that the pump can come

1 up to speed and deliver the rate of flow. You can do that
2 in different ways. The injection valve just doesn't open
3 like a real demand. You can crack it open. It is not the
4 real challenge during that quarterly test. The other test
5 you do is at lower pressure, because you want to make sure
6 that you can deliver flow when the pressure goes down. You
7 can only do that when the plant is going down, shutting
8 down.

9 The real test that challenges the system is, quite
10 frankly, done only once during shut down, because that
11 challenges all pieces of the high pressure coolant injection
12 system. What we need to do at this point, so that perhaps
13 everything we are doing in the area of INC is well focused,
14 it seems to be leading to some real good results, but maybe
15 other areas we had better probe a little better than we have
16 done.

17 [Slide.]

18 MR. MAYS: The next chart is just another break
19 out of the HPCI discovery and instead of by pieces of
20 equipment is whether it was fail to start or fail to run
21 issues. Similar conclusions were reached about that
22 information.

23 [Slide.]

24 MR. MAYS: To move now to slide 21, and talk about
25 the overall HPCI insights. We have discussed most of these

1 already, but this is just a list of the significant issues
2 that we discovered. There wasn't a discernable trend in
3 reliability, even though the failure rates and the unplanned
4 demand rates were going down and we didn't find any
5 variation in reliability due to the age of the plants.

6 We had some exceptions from the IPE/PRA
7 comparisons, but there were a lot of them where they were
8 comparable. We found differences, as we just discussed,
9 between the actual demands and surveillance inspections.

10 Another interesting piece was that during the
11 unplanned demands, all failures to start that actually
12 failed during unplanned demands, all of them were recovered.
13 The maintenance and testing out of service was important,
14 but again that was 1 failure out of 63 demands, so there is
15 some uncertainty associated with that. The injection valves
16 and the turbine failure to run were dominant contributors to
17 the unreliability.

18 With that I would like to move on to the emergency
19 diesel generator.

20 CHAIRMAN JACKSON: May I make two comments?

21 MR. MAYS: Yes, ma'am, or three if you would like.

22 CHAIRMAN JACKSON: Thank you. Actually, the
23 insights are also interesting. I mean, as you say, you have
24 sparseness of data relative to the maintenance and testing
25 out of service, but that and the bullet above are

1 interesting, because, as you point out all the failures to
2 start from actual demands were recovered. Many times that
3 requires operator intervention and, of course, that is why
4 people run plants. It does point out the sensitivity of
5 both maintenance and people in all of this.

6 [Slide.]

7 MR. MAYS: The diesel generator information will
8 follow the same format we had for the HPCI information. The
9 first diagram there is just an explanation of where failures
10 would occur that would have been considered in the analysis
11 of events.

12 [Slide.]

13 MR. MAYS: The next slide is the diesel generator
14 evaluation model. The biggest difference between this and
15 the HPCI model is that we basically didn't need breakdown
16 within failure to start and failure to run any particular
17 components that showed a different population experience.
18 There were some other issues that came up in the diesel
19 generator report and evaluation that are important. They
20 have to do with the data reportability.

21 There were some differences in the data between
22 plants and it turns out, thanks to some help we got from NRR
23 in understanding what we were seeing, we were able to
24 determine that it had to do with the reportability
25 requirements associated with diesel generators, which

1 affected not only the reportability but also the nature and
2 kind of tests that were being done. Reg. Guide 1108, which
3 was promulgated in 1997 --

4 CHAIRMAN JACKSON: 1997?

5 MR. MAYS: Excuse me, 1977, I can't even read
6 straight. In 1977, it discussed the testing and
7 reportability of diesel generators. Of course, as it came
8 out in 1977 it didn't apply to all plants, because there
9 were plants that already had licenses. It was subsequently
10 incorporated into Reg. Guide 1.9 in July of 1993. What we
11 found was that there were differences in the population of
12 plants and the nature of the failures associated with
13 testing.

14 So we had to divide the group of plants up into
15 those who were reporting under Reg. Guide 1108 and those
16 that were not. So those who were not reporting under Reg.
17 Guide 1108 had significantly less information for which we
18 could do analysis. So this again, doesn't mean that the
19 data wasn't existing. It just means that if we needed to
20 get it we would have to go out to each individual site and
21 pull out information out of logs. For those of us who have
22 done that in PRAs in the past, that is called data dog. So
23 it is difficult and time consuming work to do at that level.

24 This also points out one of the issues that we
25 raised in the reliability rule. You will notice that the

1 structure we asked for in the reliability rule about the
2 nature of the demands, the failures associated with those
3 demands, there is a consistent theme throughout all of this
4 information. Here is an example about how the reportability
5 requirements can effect the ability to do that, because you
6 may not be able to associate demands and failures in an
7 appropriate way to do unreliability. Subsequently, the
8 results of the numerical analysis I am going to give you
9 after this are all associated with the plants reporting
10 under Reg. Guide 1108, which is about half the plants.

11 [Slide.]

12 MR. MAYS: The next slide gives an indication
13 similar to the table from HPCI about the failures and
14 demands and the associated Bayesian intervals. There are a
15 couple of differences on here that I would like to point out
16 and some similarities. You will notice that failure to
17 start, for example, we were able to use the cyclic tests
18 that are done once every refueling outage to start the
19 diesel, load it and have the sequencer go through all of its
20 steps to verify that the diesel can start under an emergency
21 start condition and load all the appropriate loads.

22 So we have many more demands for failure to start
23 associated with that than we do for actual unplanned
24 demands, of which are about a factor of ten to one
25 difference. We had two actual failures to start during

1 actual unplanned demands, neither one of which was
2 recovered. So that affects our recovery probability number.
3 We did also find three different failure rates associated
4 with the failure to run information that we were able to
5 generate and so the numbers on the right-hand side, the
6 Bayesian intervals, are actually failure rates derived from
7 the information about the failures per demand within each
8 time period.

9 During the failure to run events, we had three
10 events where there was a diesel that was required to be
11 operating and it failed while it was running and in all
12 three of those the diesel was recovered. That led us to the
13 results that we have here on the next slide.

14 [Slide.]

15 MR. MAYS: The population of the plants reporting
16 under Reg. Guide 1108 have an unreliability of about .44.
17 That is a 95.6 population reliability average. There is a
18 couple of important things to look at in here. You will
19 notice that the maintenance out of service in this case is 3
20 percent of the total of the 4.4 percent unreliability in
21 here. So that is a fairly large contributor. If you take
22 that 3 percent out, then the reliability of the diesel
23 generators in this population is around 98.6 percent. That
24 tells us that the machines, when they are available, are
25 operating fairly reliably.

1 There are some station blackout implications,
2 station blackout rule implications that we can talk about
3 either now or when we get to the end of the slide.

4 CHAIRMAN JACKSON: Just won't forget.

5 MR. MAYS: I won't forget. It is on there, I
6 won't forget. So let me press on to some of the other
7 things, because I think that is an interesting discussion
8 that we need to talk about.

9 We did the similar trending information with the
10 diesel generator information as with respect to the
11 unplanned demand rates and failure rates and unreliability.
12 We again found the same thing, unplanned demands and
13 failures were going down in a significant fashion, but the
14 unreliability estimates over the same time period were flat.
15 So this is another indication of how you need to have both
16 failures and successes in order to understand the
17 reliability and risk implications of operation.

18 [Slide.]

19 MR. MAYS: The next slide demonstrates the
20 different failure to run rates that we found by plotting out
21 the failures associated with the time within the runs in
22 which the failures occurred. You will notice within the
23 first half-hour there is a number of failures that indicates
24 a different failure rate than the other three periods. That
25 was the basis of this plot for coming up with three

1 different periods for evaluating the failure to run
2 information.

3 Most PRAs either use a single failure to run
4 number or they will use one for short and a second one for
5 later time periods and there will be an effect of that you
6 will see on the next slide.

7 COMMISSIONER ROGERS: Just before leaving this, I
8 thought this was a very interesting slide, but I certainly
9 didn't understand it. I mean, I can understand the first
10 half-hour situation, but why after a diesel generator has
11 started and loaded -- I assume to run involves loading as
12 well?

13 MR. MAYS: Yes, sir.

14 COMMISSIONER ROGERS: That there is an increase in
15 the failures, I mean, once it is running, these things, the
16 generators, are very reliable once they start running. So
17 the failures are coming from what? Not from the generator
18 itself, presumably, but from some of the other loaded
19 equipment or what? I mean, I just don't understand what is
20 happening here, because once you start a generator and it
21 starts running, five hours is nothing.

22 MR. MAYS: Actually, the failure rate is going
23 down as you go up in time, the rate is going down. We did
24 experience some failures. We found they were broken off in
25 a couple of groups. Some of them had to do with voltage

1 regulator and controls of that nature, that after a certain
2 number of hours would start to degrade. We found some
3 issues associated with cooling water or lubrication oil that
4 would be leaking or perhaps become a problem later on down
5 the line. There were a number of different causes
6 associated with this information. I don't have the
7 specifics of that with me right now, but that information is
8 in the report that we have, to discuss where the fail to
9 runs came from.

10 COMMISSIONER ROGERS: But the diesel generators
11 are more reliable, in terms of running reliability, as time
12 goes on, not less reliable. This is a cumulative plot which
13 could be misleading. The failure rates are dropping off in
14 time.

15 CHAIRMAN JACKSON: Well, the slopes tell you that.

16 COMMISSIONER ROGERS: Right, okay.

17 [Slide.]

18 MR. MAYS: A comparison with the diesel generator
19 reliability information from operating experience and the
20 information we were able to pull out of the IPE/PRAs are
21 shown on the next view graph. There are three distinct
22 groups there. The bottom is those in which the PRAs
23 indicated a six-hour mission time for their diesel
24 generators. The middle one is for those with PRAs that had
25 eight-hour mission times and the ones at the top are for 24-

1 hour mission times.

2 You notice that the information from the PRAs
3 matches up fairly well with the operating experience that we
4 were able to observe. The differences are all primarily in
5 the 24-hour ones in which the PRAs generally had a worse
6 reliability number in their models than what we were seeing
7 from the operating experience. We went back and looked at
8 that and it appears that the failure to run numbers that
9 they were using for 24-hour mission times were more along
10 the lines of what we were seeing in the middle period rather
11 than in the period from 14 to 24 hours. We believe that is
12 primarily the reason why those are lower.

13 Looking at the trends associated with the age of
14 the plants, we found some really interesting information
15 here. What we found was that, when you plot the failures
16 per diesel generator year, you will notice a trend that is
17 going upward as the low power license date gets closer to
18 the present, meaning that we were seeing statistically
19 higher failures per year for the newer plants than we were
20 seeing for the older plants.

21 When we went back and looked at that there is a
22 couple of factors that influenced that. The first one is
23 that this data set that we were using was from '87 to '93
24 and there were a significant number of new plants that come
25 online in about that time period, or were new just before

1 that. Also, when we looked at the plants that had the
2 higher failure rates we discovered that a lot of them had
3 situations where there were designer installation errors
4 that were being detected early on and were subsequently,
5 because it was under 1.108, had to do accelerated testing.
6 So we were finding a lot of failures that were occurring
7 early in the period of analysis and were subsequently, in
8 the later years, not reoccurring.

9 So we would see two or three different failures of
10 a similar nature in the first year or two and then,
11 subsequently, we would see less. That is another reason why
12 some of the plants in the '80 to '90 time period were
13 showing higher failure rates there. When you go back and
14 take the failures and match them up with the successes and
15 demands and plot the unreliabilities and put the trend
16 through there, you don't see any statistical information
17 that indicates that the reliability of the older plants in
18 this population was any different than the reliability of
19 the new plants.

20 Again, it is another case of where you need to
21 look at the whole picture and not just the failures in order
22 to be able to understand what is going on.

23 [Slide.]

24 MR. MAYS: The insights that we drew from the
25 diesel generator report were, again, that there was no

1 discernable trend in the reliability over the period of
2 study, even though failure rates and unplanned demands were
3 both decreasing and we did notice a higher failure rate for
4 the plants that were in the 1980 to 1990 licensing period.

5 We did discover the three distinct failure to run
6 rates and we felt we had pretty good agreement between our
7 operating experience and the information from the IPEs and
8 the PRAs. We did find some similar differences between the
9 actual unplanned demand failures and the routine
10 surveillance inspections as we did with the HPCI system. We
11 did find that the failures to start on actual unplanned
12 demands for the diesel generators were not readily
13 recoverable based on the operating experience as compared to
14 what HPCI was.

15 We did also look, because the diesel generator
16 analysis here is on a train basis, the plants all have one
17 or more of them or two or more of them I should say, and so
18 an important part of our look was to look for common cause
19 failures associated with this data. We found no common
20 cause failures in multiple diesels during actual unplanned
21 demands. Of course, with only 100-and-some-odd unplanned
22 demands we weren't expecting to see any, based on what our
23 current assumptions are about common cause failure
24 probabilities. We did see some common cause failure events
25 in some of the surveillance testing information and that

1 information is incorporated in the analysis and in the
2 results.

3 The demand reliability, failure to start failure
4 to run information is consistent with the station blackout
5 rule assumptions for the plants reporting under Reg. Guide
6 1108. The interesting part of that is, when the Reg. Guide
7 was written and the analysis was done for the station
8 blackout rule, the indications were that unavailability due
9 to maintenance out of service for testing was in the
10 ballpark of 7 times to -3. We are seeing 3 percent, which
11 is about four times higher than that.

12 However, the 95 percent reliability targets and
13 the 97-and-a-half percent reliability targets were based on
14 the assumption that the maintenance out of service was
15 fairly low and we are seeing a higher maintenance out of
16 service, but we are seeing a lower failure to start failure
17 to run contribution now than what was present when the
18 analysis was done to station blackout rule.

19 The net effect is that it is about a wash in terms
20 of individual train reliability, but I think it is important
21 to know, also, that maintenance out of service to the
22 overall mission when you have multiple diesel generators,
23 because you don't have the same common cause contribution
24 associated with maintenance out of service that you would
25 from failure to start or failure to run events. So it is a

1 complicated situation with respect to how much maintenance
2 out of service is allowable or tolerable, because you have
3 to go again and go back and put the whole model together in
4 order to be able to appropriately deal with that issue.

5 COMMISSIONER ROGERS: That seems to say that that
6 is a tough job for a licensee to do.

7 MR. MAYES: Actually, what it seems like they are
8 doing is balancing off maintenance out of service versus
9 reliability, such that you do more maintenance while the
10 diesels and the plant is operating to keep it in a reliable
11 state so that on demand it is fairly reliable. That is
12 somewhat speculative on my part, but it is not surprising to
13 see things like that occur.

14 MR. THADANI: There are two parts. Clearly, one
15 part is the issue of station blackout and what were the
16 assumptions. The end results may turn out to be okay, but
17 nevertheless there are plants, as Steve said, which have
18 established goals of higher reliability, such as 97.5
19 percent. The maintenance rule requires that the licensees
20 balance unavailability and unreliability.

21 That is, if you have a system that who's
22 unreliability is ten to the minus three and you are
23 maintaining it 30 days in a year there is something wrong,
24 because you are very likely going to get in trouble when
25 that system is under maintenance. So there is explicit

1 requirement under part 8.3 of the maintenance rule. Each
2 licensee is to track data basically, information, to see are
3 they seeing trends is unavailable due to maintenance well
4 above what they would want. They need to balance these. I
5 think time will tell how well that is really working. We
6 need a little more experience with that to see how well it
7 really works in practice.

8 MR. MAYS: When we compared the results of the
9 plant specific unreliability estimates excluding the
10 maintenance out of service portion, we compared the failure
11 to start failure to run probabilities against what the
12 plants had committed to, whether the 95 or the 97.5 percent
13 targets, we found that when you don't consider the
14 maintenance out of service aspect all the plants would meet
15 a 95 percent target, based on the mean value of the plant
16 specific distribution from the operating experience and that
17 18 of the 19 plants over this period of time would meet the
18 97.5 percent target. The one plant whose mean value of its
19 distribution was below 97.5 was at 97.1 and the range of the
20 distribution was up to 99 and down to 94. So it is not that
21 terribly significant a difference.

22 We also looked at the maintenance out of service
23 associated with shut down as opposed to operation and we
24 found that there was about a three to four times higher
25 maintenance out of service on demand, on reliability

1 associated with diesel generators and shut down, as opposed
2 to in operation.

3 So having looked at those two system reliability
4 and performance information, we would want to talk a little
5 bit about where we see this information going and we
6 discussed a little bit of it as we went along already.

7 We think that the reliability information from
8 these system studies can be used directly in risk
9 assessments and licensee performance whenever we have this
10 information and want to look at how somebody is performing
11 on any particular issue. As we talked to you back in
12 August, this is the kind of information we had in mind when
13 we talked about going to more risk based performance
14 indicators.

15 The other thing we want to do is, both the agency
16 and the industry often have programs to improve the
17 performance of systems or components or other parts of the
18 plants. This kind of analysis technique is useful for
19 determining whether the risk and reliability really have
20 been effected by what the program is hoping to accomplish.
21 So this is a process by which we can track data and see
22 whether or not we are getting the gains we were hoping to
23 get or whether or not we have a different problem.

24 We have been talking before. We have been meeting
25 with the people from the inspection branch and giving We

1 have been talking before, we have been meeting with the
2 people from the inspection branch and giving input into them
3 as to what we are seeing in these reports so we can have a
4 basis for investigating further into the data to determine
5 how this information might feed into inspection programs and
6 other activities associated with that.

7 Also, as we talked last year in the accident
8 sequence precursor meeting, we are moving towards more plant
9 specific and better, more detailed models in doing our
10 action sequence precursor program and those models are being
11 used in other applications as well and we anticipate plant
12 specific type risk information would be appropriate to put
13 in those kinds of models.

14 CHAIRMAN JACKSON: Before you go on, and you did
15 allude to this in an earlier comment, but I note that with
16 your uses you have talked about the input to the inspection
17 and licensing program of NRR and you alluded to sharing
18 information with research and it strikes me that that is an
19 important use that merits being on your view graph, in the
20 sense that this has the potential for enhancing the quality
21 of PRA methodology and so that kind of a transfer of
22 knowledge, as the research people are the ones reviewing the
23 IPES and PRAs. I think it is very important. So I urge you
24 to post facto add that.

25 MR. JORDAN: It was a mention in words as opposed

1 to --

2 CHAIRMAN JACKSON: I don't know if he said it in
3 words, actually. I am just urging him.

4 MR. JORDAN: Right, well, I want to emphasize that
5 it is one of the elements of the PRA program plan and so we
6 are bringing you one of those pieces.

7 CHAIRMAN JACKSON: Go on.

8 [Slide.]

9 MR. MAYS: The last slide we have here talks about
10 the overall program plan for this information and how it
11 fits with the other activities we are doing that Pat
12 mentioned earlier. In fiscal year '96 we are expecting to
13 complete the diesel generator, aux feed water, high pressure
14 core spray, reactor cool and isolation system and the
15 isolation condenser and initiating event reports
16 information. We intend to plan out and map out what kind of
17 a data base we would put together for receiving the
18 reliability rule information and subsequently distributing
19 that information.

20 As you are aware, we are also moving to an annual
21 report on the performance indicators and we anticipate
22 completing the common cause failure database and reports and
23 to update the accident sequence program and the SCSS
24 database is part of our fiscal year '96 activities.

25 In '97 we intend to update the update the previous

1 reports and also complete the reactor protection system, low
2 pressure systems for BWRs and PWRs, and to start taking this
3 information from this system stuff and being able to take
4 that to research and have them put into one of the
5 simplified plant models. We are also expecting at that
6 point to construct and start implementing the reliability
7 rule database for how we would do that.

8 After that, the process is fairly repetitive, in
9 that we would update the previous analysis to get the
10 information for the reliability data rule and begin putting
11 that into the analysis instead of relying only on the LAR
12 information. That would have the impact of having us being
13 able to further reduce some of our contractor support needed
14 and being able to do more of this work in-house.

15 CHAIRMAN JACKSON: Let me bring you back to that.
16 What kind of balance do we have in terms of capability in-
17 house? You seem very knowledgeable and if you got run over
18 by a truck, what would happen?

19 MR. MAYES: That is why Pat and I never travel in
20 the same vehicle. No, actually, we as part of our program
21 plan looked at both the amount of work we have to do and the
22 contracting work and dollars. We are planning on going from
23 somewhere in the ballpark of \$4 million this year in
24 contract work, to in the year 2000 being about \$2 million in
25 potential contract support with a staff, FTE commitment, of

1 about 10 FTEs. What would happen during that period of time
2 is we are taking the opportunity, both from the project
3 management standpoint and the training standpoint, to bring
4 our staff to the point where they can take over more of this
5 work.

6 That's all I have.

7 MR. JORDAN: I want to make sure that Pat has an
8 opportunity to make any summary statements since I put him
9 around.

10 MR. MAYS: Or corrections.

11 [Laughter.]

12 MR. JORDAN: I have just a couple of comments to
13 make, sort of based on perceptions. I was surprised by the
14 analysis to find that unreliability or reliability hasn't a
15 trend over this period of time. My perception was that
16 there was a trend improvement and it was misled by the
17 frequency of events rather than reliability. So that is
18 sort of defuncts a perception that me and other members of
19 the staff, I think, had. We haven't really changed the
20 reliability of these equipment during this timeframe.

21 To slip further to the idea that the
22 unavailability due to maintenance or testing is an important
23 part and is a very heavy contributor for both diesels and
24 high pressure injection system, we had an event yesterday in
25 which a plant had a loss of off-site power. One of two

1 diesels was in maintenance when the event occurred. So
2 these are real.

3 MR. JORDAN: There are also differences between
4 the nature of failures during actual demands versus those in
5 surveillance.

6 CHAIRMAN JACKSON: That is what jumped out at me
7 on that.

8 MR. JORDAN: We have other work to look at to
9 reexamine surveillance testing as far as its validity and
10 value and are we really testing the right things. I think
11 this indicates that we have to put that risk perspective
12 into that work. Dr. Rossi is involved in managing that in
13 the other branch.

14 Finally, I was also struck by the likelihood of
15 recovering a turbine driven piece of equipment and the
16 diesel generator. The turbine driven pump, which we call a
17 water wheel in some discussions, is a fairly simple device
18 and it is quite recoverable. The diesel, once it fails or
19 it doesn't start, is much more difficult to resume. So, for
20 instance, in accident management, I think, it is interesting
21 to keep those kinds of thoughts in the back of your mind in
22 terms of how likely you are to recover the diesel and how
23 likely you are to restart the HPCI.

24 Maybe Ashok has some comments also?

25 MR. THADANI: No, I think I quite agree with what

1 you said. I just want to make another comment. That is, I
2 think a lot of this information accident sequence precursor
3 activity is absolutely critical. If we are going to learn
4 from experience and will continue to learn from experience,
5 we had better pay attention to what is happening. So these
6 analysis tools, I think, are very important tools. In fact,
7 what we are doing, as you know, we have established senior
8 reactor analyst positions in each of the regions. These
9 people come here for training and we make sure that they are
10 exposed to assessing events using these tools as part of
11 their training program. So that I think the sensitivity and
12 understanding will be greater.

13 MR. TAYLOR: That concludes our brief.

14 CHAIRMAN JACKSON: Thank you.

15 Do you have any other questions, Commissioner
16 Rogers?

17 COMMISSIONER ROGERS: Well, just first, I thought
18 that was a very interesting briefing. I think it is really
19 reveals, I think much more clearly the value of the data
20 that we seek. I think it is beginning to show how important
21 that data really is. I think that was very important.

22 MR. JORDAN: I think it is valuable to the
23 industry to look at this type of analysis, too. I mean, it
24 is their equipment.

25 COMMISSIONER ROGERS: I just had a couple of

1 little questions. One is the SCSS database, where does that
2 stand? At one time I think you were talking about reducing
3 the effort in that direction and starting to relook at that
4 again. Where does that stand now?

5 MR. JORDAN: I'll ask Pat to respond.

6 MR. BARANOWSKY: We are, of course, looking to
7 optimize our resources, but the SCSS is still in existence
8 and what we are doing is we are transferring from the
9 mainframe to a minicomputer or workstation and then trying
10 to make it available it available on the LAN. It is still a
11 significant source of information for us in system
12 reliability studies and getting common cause failure data.

13 CHAIRMAN JACKSON: This transfer will also help in
14 resource?

15 MR. BARANOWSKY: Yes, the other system is quite
16 expensive and I think we are going to have that completed
17 this summer, approximately.

18 COMMISSIONER ROGERS: So that database will be
19 maintained?

20 MR. BARANOWSKY: It will be maintained, yes. We
21 were able to come up with funds to save it.

22 COMMISSIONER ROGERS: Good. The other refers to
23 the simplified plant models. To what extent are you
24 subjecting those to some kind of an outside review process,
25 the models themselves, these simplified models?

1 MR. BARANOWSKY: I don't know if I can answer.

2 Dr. O'Reilly who is more familiar with this can answer that.

3 MR. O'REILLY: I am Pat O'Reilly from AEOD.

4 Commissioner Rogers we are going to have the revision two
5 models put through a systematic quality assurance and
6 checkout program, sponsored by the Office of Nuclear
7 Regulatory Research. That contract has been put in place
8 and should get started fairly soon. We have not had the
9 revision one models subjected to a systematic QA program.
10 What we have been doing is, ad hoc checking out of the model
11 as we review a specific event.

12 COMMISSIONER ROGERS: Has ACRS had an opportunity
13 to comment on those models?

14 MR. O'REILLY: No, they haven't.

15 COMMISSIONER ROGERS: Do you expect to do that?

16 MR. O'REILLY: We will be in conference in
17 research through the technical coordination group to get
18 into that.

19 COMMISSIONER ROGERS: I think they might have some
20 valuable insight there. That is the scenario that they
21 probably have their lot of expertise in.

22 MR. O'REILLY: Yes, it would.

23 COMMISSIONER ROGERS: That's all. Thank you very
24 much. I think it was an excellent briefing.

25 CHAIRMAN JACKSON: I want to thank you, Mr.

1 Taylor, Mr. Jordan and the rest of you for a very
2 informative and excellent presentation this morning. I
3 believe that the ongoing review that you are doing with the
4 operational events database against the results of actual
5 PRAs provides very valuable insights, both into the
6 limitations of PRA methodology, but also the utility of it,
7 particularly in the use of reliability data and assumptions
8 and these kinds of surprises, let us say, we find out in
9 terms of actual reliability. I believe that what you are
10 pursuing in terms of looking at any plant specific
11 deviations and how that can give added focus in inspection,
12 it should be thoroughly understood and reconciled.

13 So I encourage your efforts in that area. I also
14 encourage you along the lines that you have already started
15 in terms of these crossfeeds more broadly from AEOD to NRR
16 and research and encourage you along the lines of this
17 increased focus that you are making the use of in NRR. I
18 look forward to hearing more as you work through your
19 program plan and to have periodic updates and look forward
20 to hearing you report as you are developing the more risk
21 significant performance indicators, as you develop that.

22 Thank you very much.

23 We stand adjourned.

24 [Whereupon, at 11:24 a.m., the briefing was
25 concluded.]

CERTIFICATE

This is to certify that the attached description of a meeting of the U.S. Nuclear Regulatory Commission entitled:

TITLE OF MEETING: BRIEFING ON SYSTEM RELIABILITY STUDIES
- PUBLIC MEETING

PLACE OF MEETING: Rockville, Maryland

DATE OF MEETING: Wednesday, February 7, 1996

was held as herein appears, is a true and accurate record of the meeting, and that this is the original transcript thereof taken stenographically by me, thereafter reduced to typewriting by me or under the direction of the court reporting company

Transcriber: Jessal Munson

Reporter: Tessa Minson



OPERATIONAL RELIABILITY OF RISK-SIGNIFICANT SYSTEMS IN U.S. NUCLEAR POWER PLANTS

Edward L. Jordan
Patrick W. Baranowsky
Steven E. Mays

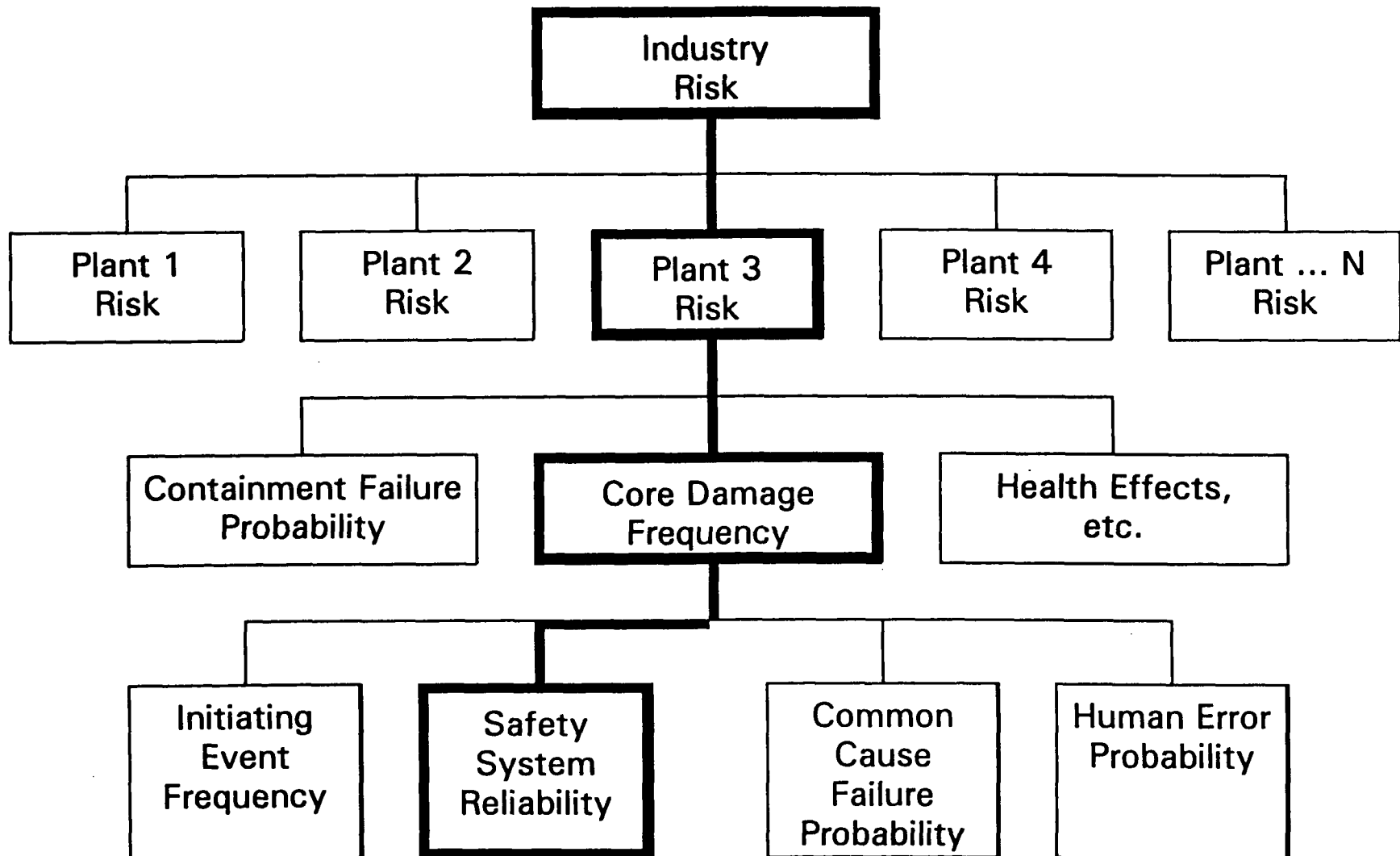
Office for Analysis and Evaluation
of Operational Data

RISK-BASED ANALYSIS OF REACTOR OPERATING EXPERIENCE

Use Reactor Operating Experience to:

- Assess and trend risk indicators
- Compare with probabilistic risk assessments (PRAs) and individual plant examinations (IPEs)
- Identify technical insights relating to risk contributors
- Provide insights to industry and regulatory activities related to risk

DECOMPOSITION OF RISK



PROGRAM ELEMENTS

- Accident Sequence Precursor (ASP) Annual Report
 - ASP methods (Office of Nuclear Regulatory Research)
 - ASP database
- Initiating events periodic report
 - Loss of offsite power (LOOP) database
 - Special initiators (e.g., power-operated relief valves, steam generator tube ruptures, human errors)
- System reliability studies
 - Reliability indicators
 - Component analyses

PROGRAM ELEMENTS (continued)

- **Common cause failures (CCFs)**
 - Database and analysis software
 - Periodic analysis
 - International common cause data exchange effort
- **Performance Indicators (risk/reliability)**
- **Data Systems**
 - Sequence Coding and Search System (SCSS) and Nuclear Plant Reliability Data System (NPRDS)
 - Reliability data

SYSTEM RELIABILITY STUDIES

Purpose:

To evaluate reliability and provide engineering insights of risk-important systems based on operating experience.

Objectives:

- Use actual demands, failures and unavailabilities to estimate reliability
- Analyze trends in reliability
- Quantify uncertainties
- Compare findings with published PRA/IPE values
- Identify plant-specific differences
- Provide engineering insights

STUDIES BEING CONDUCTED

- **Boiling Water Reactor (BWR) Systems**
 - High Pressure Coolant Injection (HPCI)
 - Reactor Core Isolation Cooling (RCIC)
 - High Pressure Core Spray (HPCS)
 - Isolation Condenser (IC)
- **Pressurized Water Reactor (PWR) Systems**
 - Auxiliary Feedwater (AFW)
 - High Pressure Safety Injection (HPSI)
- **BWR and PWR Low Pressure Injection Systems**
- **BWR and PWR Reactor Trip Systems (RPS)**
- **Emergency Diesel Generators (EDGs)**

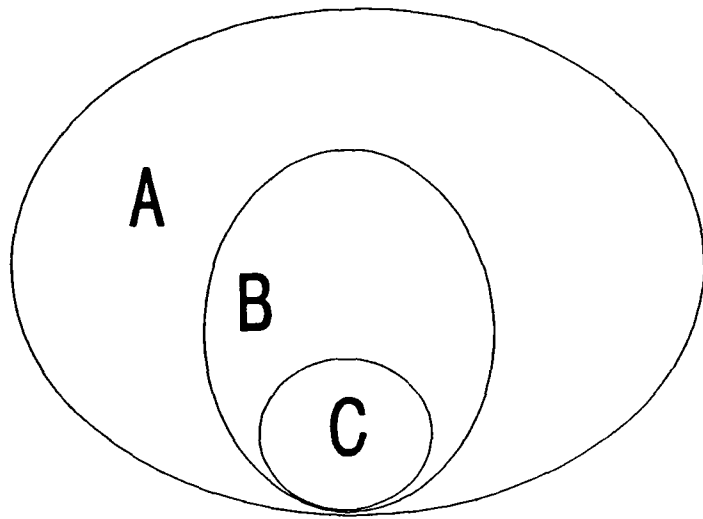
METHODOLOGY OVERVIEW

- **Standardized and systematic study procedure**
- **Detailed evaluation of events using risk analysis methods and models**
- **Rigorous mathematical treatment of reliability and availability data, including uncertainties**
- **Detailed analysis of results, including independent peer review**

METHODOLOGY APPROACH

- Develop system boundary limits
- Gather operational occurrence data from licensee reports
- Characterize occurrence data
 - Reported inoperabilities
 - Inoperabilities where safety function actually lost
 - Safety function losses where actual demands could be counted
- Determine failure probabilities and uncertainty intervals using Bayesian techniques
- Use simple fault tree model to determine unreliabilities
- Compare results with applicable PRAs and IPEs
- Compare results with applicable regulatory activities
- Analyze data trends

DATA SET RELATIONSHIPS

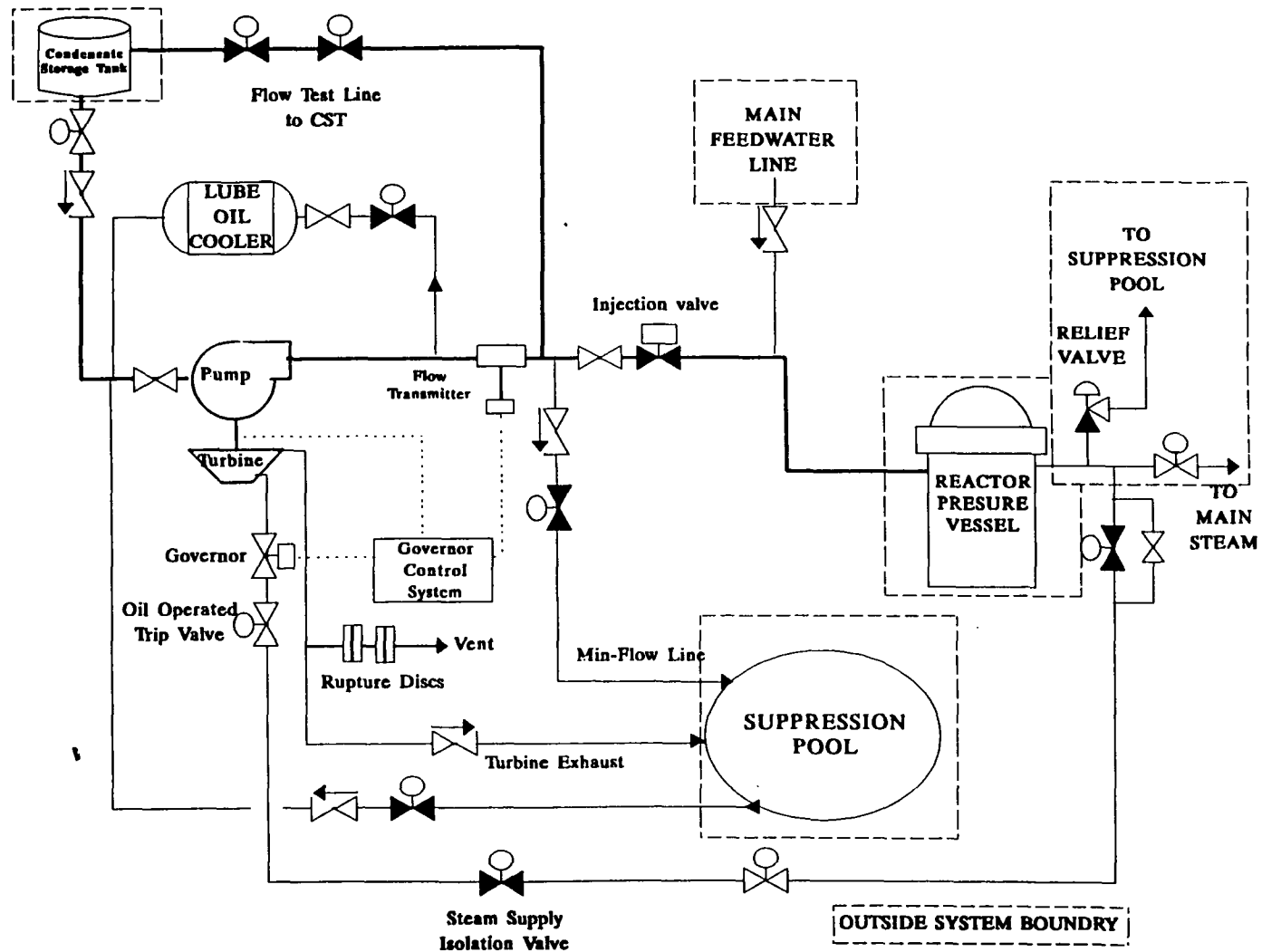


- A** The system was not operable as defined by applicable technical specifications or safety analysis reports.
- B** The safety function of the system was lost.
- C** The safety function of the system was lost and a corresponding demand count could be determined or estimated

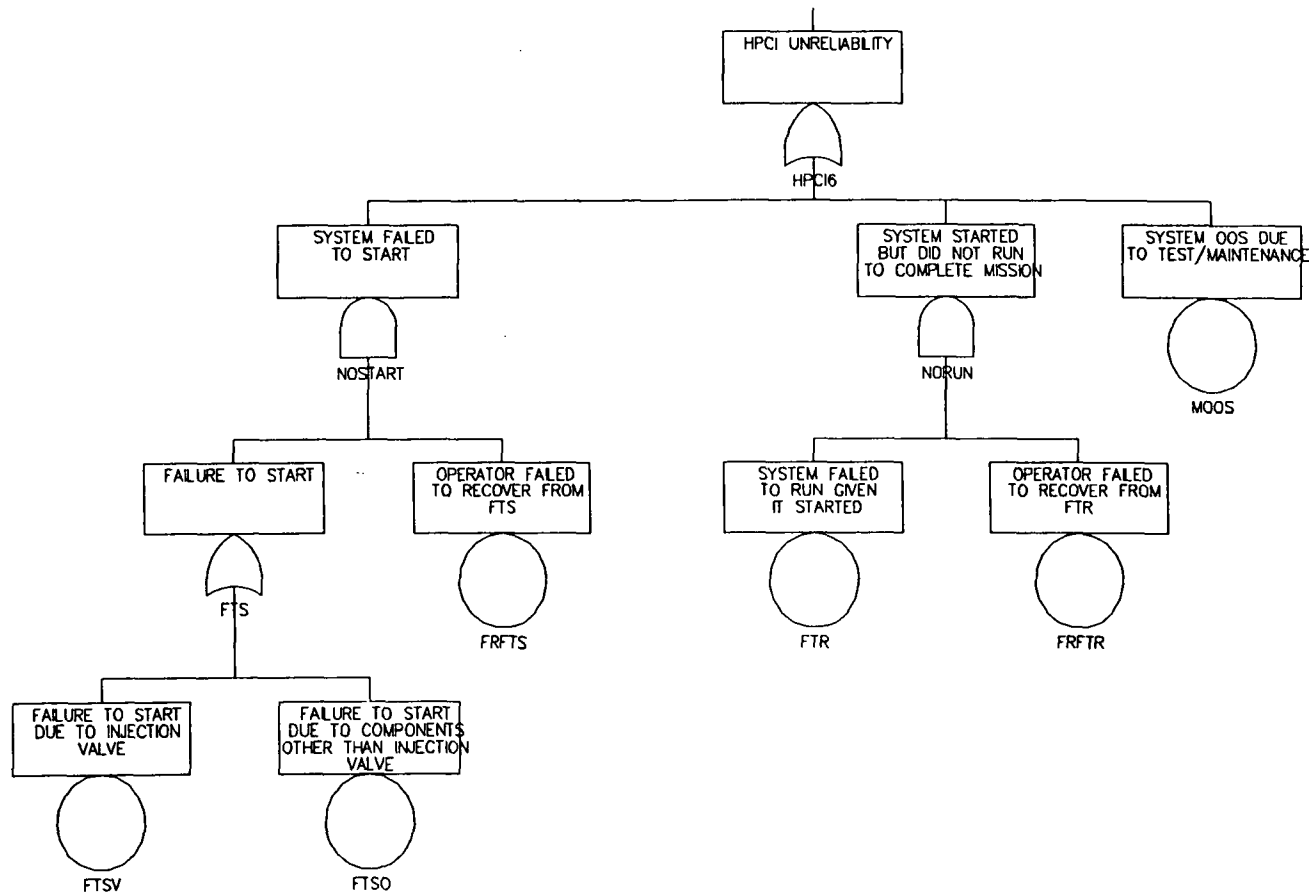
BAYESIAN TECHNIQUES EMPLOYED

- **Simple Bayes**
- **Empirical Bayes**
- **Constrained non-informative priors**

BWR HIGH PRESSURE COOLANT INJECTION SYSTEM BOUNDARIES



HPCI UNRELIABILITY EVALUATION MODEL



$$\text{Unreliability} = \text{Prob}\{ [(FTSV \text{ or } FTSO) \text{ and } FRFTS] \text{ or } [FTR \text{ and } FRFTR] \text{ or } MOOS \}$$

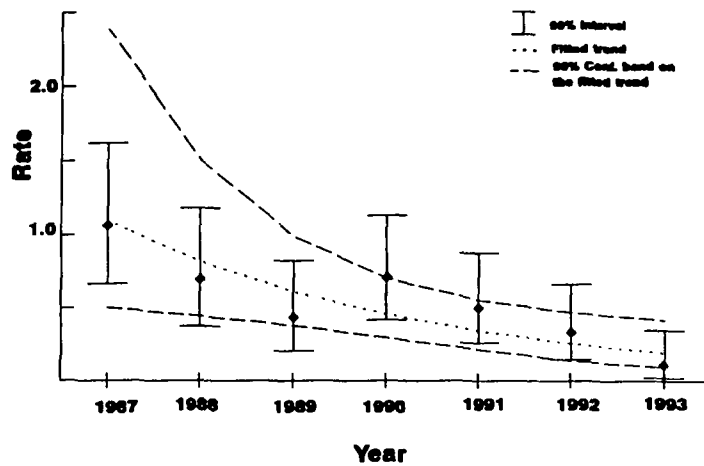
HPCI BASIC EVENT FAILURE DATA AND BAYESIAN PROBABILITY INFORMATION

Basic event	Failures	Demands	Bayes
			Mean and 90% interval
Maintenance or testing out of service (MOOS)	1	63	(0.0028, 0.023, 0.060)
Failure to start, other than injection valve (FTSO)	11	170	(0.0001, 0.060, 0.24)
Failure to start, injection valve (FTSV)	1	59	(0.0030, 0.025, 0.064)
Failure to recover from FTS (FRFTS)	0	5	(0.0004, 0.083, 0.31)
Failure to run (FTR)	7	167	(0.017 , 0.042, 0.076)
Failure to recover from FTR (FRFTR)	2	3	(0.24 , 0.63 , 0.94)
Failure of injection valve to reopen (FRO)	3	19	(0.046 , 0.20 , 0.41)

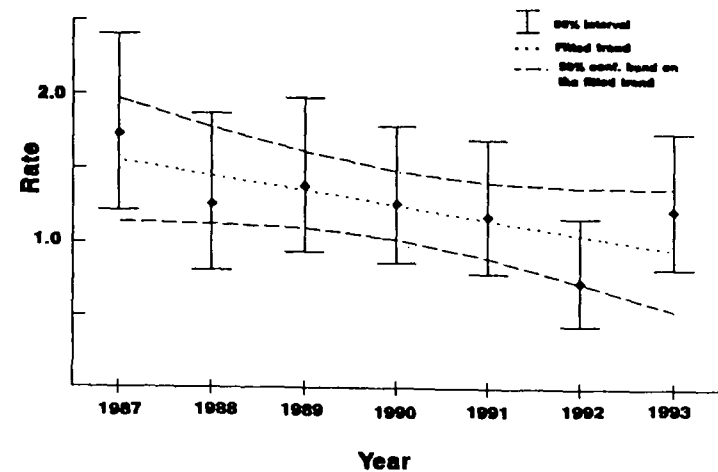
HPCI SYSTEM UNRELIABILITY, WITH RECOVERY ACTIONS

Contributor	Contributor probability	Percentage contribution
FTR*FRFTR	0.026	46
MOOS	0.023	41
FTSO*FRFTS	0.0050	9
FTSV*FRFTS	0.0021	4
Unreliability (90% interval)	0.056 (0.021,0.11)	100

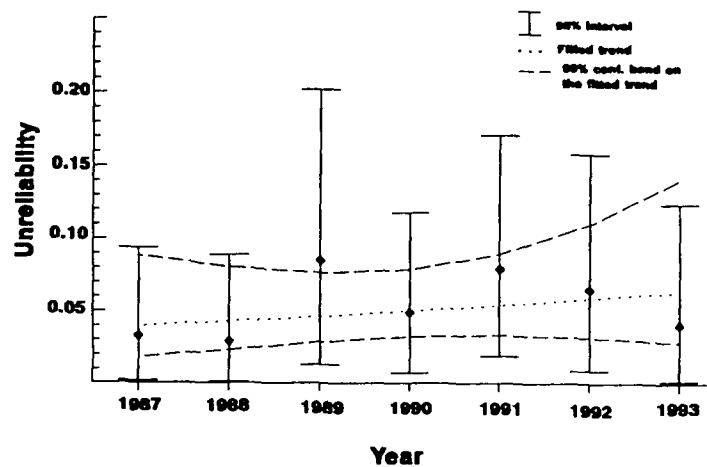
HPCI SYSTEM TRENDS



System unplanned demand rate

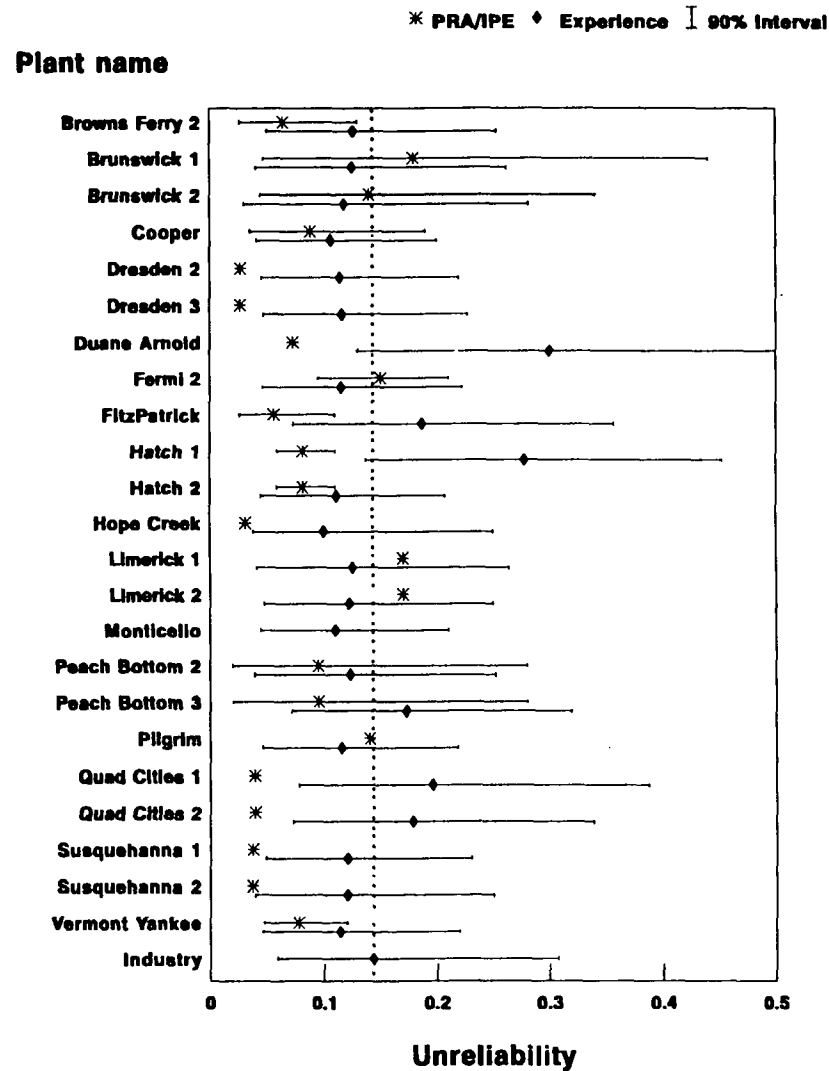


System failure rate

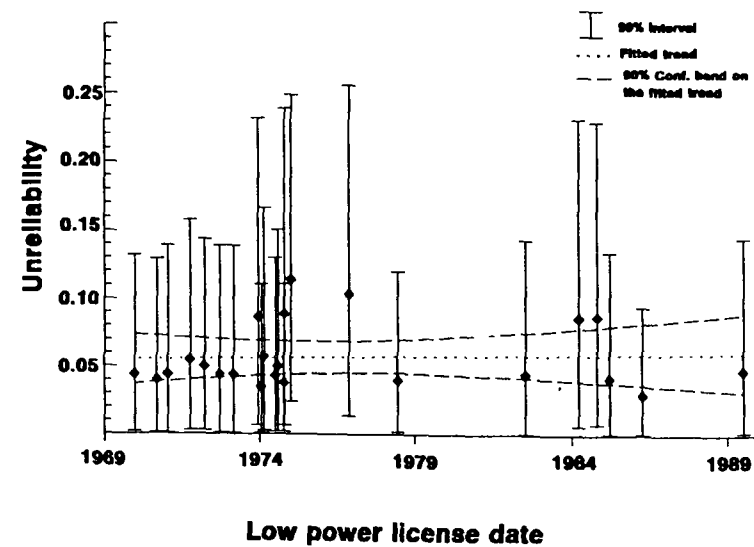
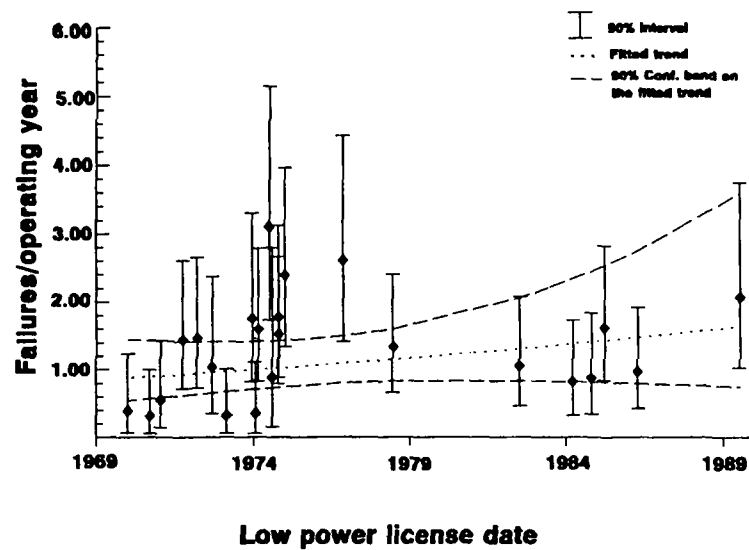


System unreliability

HPCI UNRELIABILITY COMPARISON ACTUAL EXPERIENCE VERSUS PRA/IPE VALUES



HPCI TRENDS PER YEAR OF OPERATION



HPCI SUBSYSTEM FAILURE CONTRIBUTIONS (BY METHOD OF DISCOVERY)

Subsystem	Method of discovery		
	Unplanned demand	Surveillance test	Operational occurrence
Turbine and Turbine Control Valves	58%	61%	32%
Instrumentation and Control	0%	28%	60%
Coolant Piping and Valves	42%	11%	8%

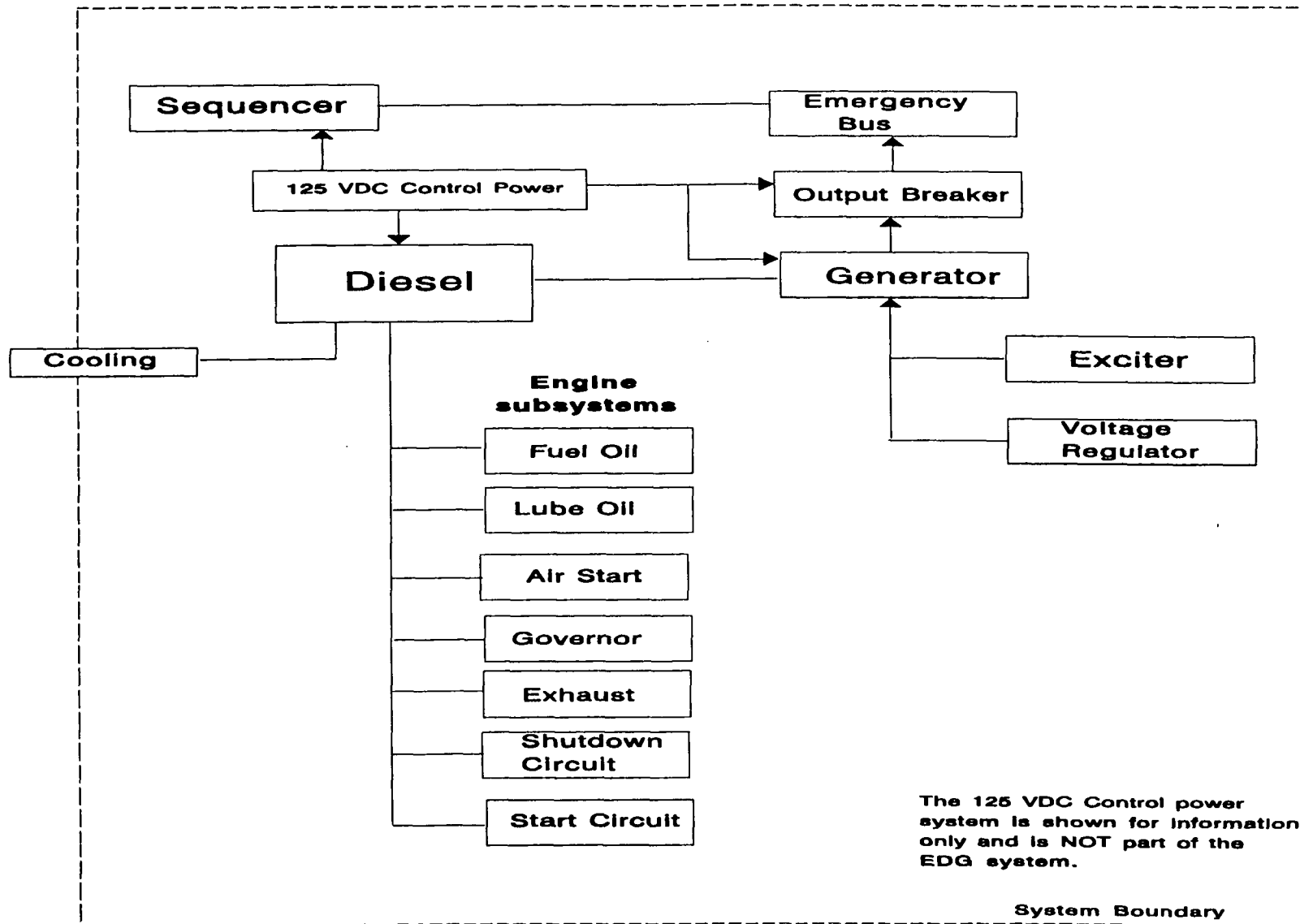
HPCI FAILURE MODES (BY METHOD OF DISCOVERY)

Failure mode (exclude MOOS)	Unplanned demand	Surveillance test	Operational occurrence
Fail-to-start	45%	74%	87%
Fail-to-run	55%	26%	13%

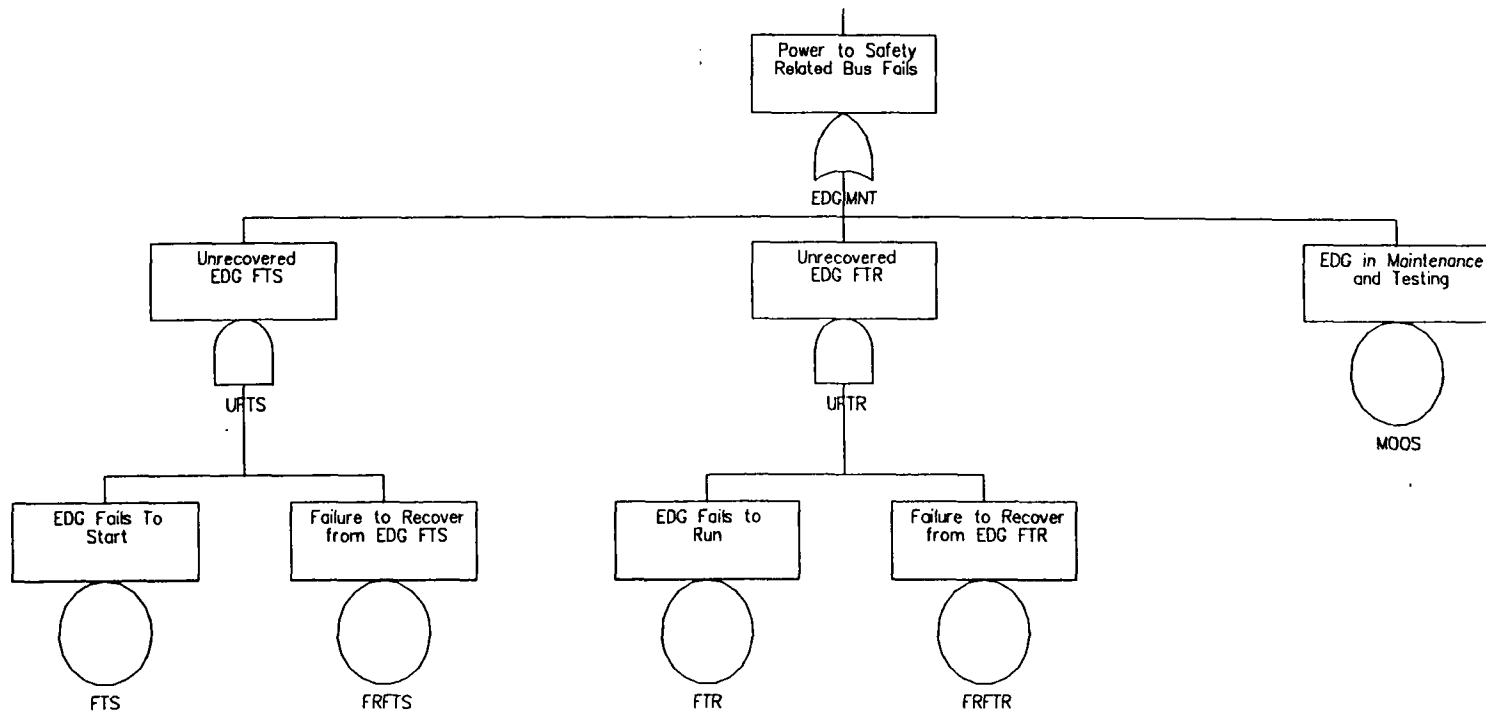
HPCI INSIGHTS

- No discernible trend in reliability
- Failure rate and unplanned demand rate both decreasing
- No significant variation in reliability or failure rates due to the age of the plants
- Generally comparable results to PRA/IPE values with some exceptions which require further investigation
- Failure differences between actual demands and routine surveillance or inspections
- All failures-to-start from actual demands were recovered
- Maintenance or testing out of service was important factor in unreliability but data limited
- Injection MOVs and turbine failure-to-run are dominant failure modes

EMERGENCY DIESEL GENERATOR TRAIN BOUNDARIES



EDG UNRELIABILITY EVALUATION MODEL



Unreliability = Prob{ [FTS and FRFTS] or [FTR and FRFTR] or MOOS }

FTR = FTR per hour X mission time

RELIABILITY DATA REPORTABILITY

- **August 1977 - Regulatory Guide 1.108 Revision 1 "Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems at Nuclear Power Plants"**
- **July 1993 - Regulatory Guide 1.9 Revision 3 "Selection, Design, Qualification, and Testing of Emergency Diesel Generator Units Used as Class 1E Onsite Electric Power Systems at Nuclear Power Plants"**
- **Statistical data differences between plants reporting under Regulatory Guide 1.108 and those not under Regulatory Guide 1.108 - data not poolable**
- **Lack of resources to gather plant-specific data**
- **Reliability data rule information required**

PRELIMINARY RESULTS

EDG BASIC EVENT FAILURE DATA AND BAYESIAN PROBABILITY INFORMATION

[Plants reporting per Regulatory Guide 1.108]

Basic event	Failures	Demands	Bayes
			Mean and 90% interval
Maintenance or testing out of service (MOOS)	3	112	(9.7E-3, 3.1E-2, 6.2E-2)
Failure to start (FTS)	19	1545	(5.0E-4, 1.2E-2, 3.9E-2)
Failure to recover from FTS (FRFTS)	2	2	(4.3E-1, 8.3E-1, 1.0E-0)
Failure to run 0-0.5 hr (FTR)/hr	12	844	(4.2E-7, 2.5E-2, 1.2E-1)
Failure to run 0.5-14 hr (FTR)/hr	15	654	(5.0E-8, 1.8E-3, 8.7E-3)
Failure to run 14-24 Hr (FTR)/hr	1	639	(2.8E-5, 2.5E-4, 6.7E-4)
Failure to recover from FTR (FRFTR)	0	3	(6.0E-4, 1.3E-2, 4.4E-1)

PRELIMINARY RESULTS

**EDG TRAIN UNRELIABILITY,
WITH RECOVERY ACTIONS**

[Nominal 8 hour mission time]

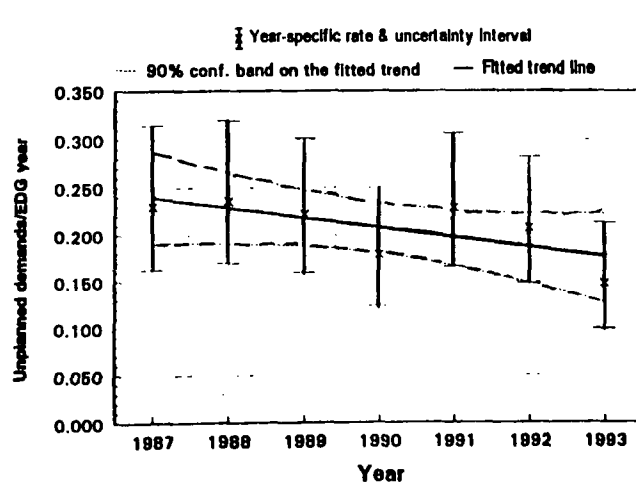
[Plants reporting per Regulatory Guide 1.108]

Contributor	Contributor probability	Percentage contribution
FTR*FRFTR	0.004	9
MOOS	0.030	68
FTS*FRFTS	0.010	23
Unreliability (90% interval)	0.044 (0.016,0.083)	100

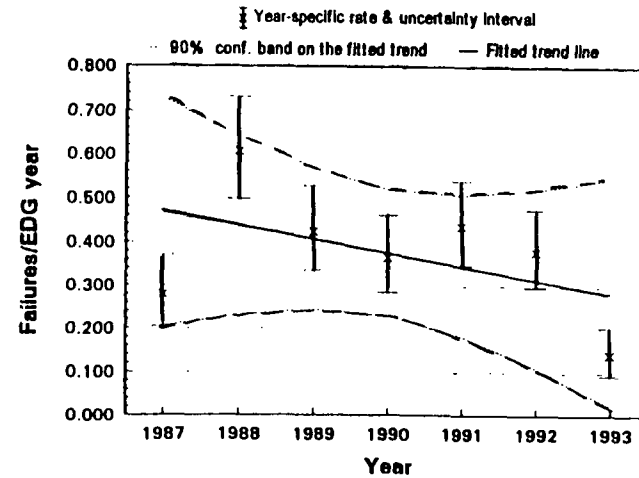
PRELIMINARY RESULTS

EDG TRAIN TRENDS

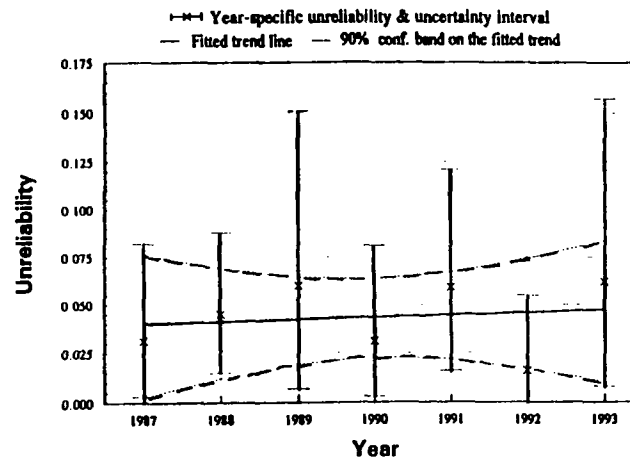
[Plants reporting per R.G. 1.108]



Train unplanned demand rate



Train failure rate

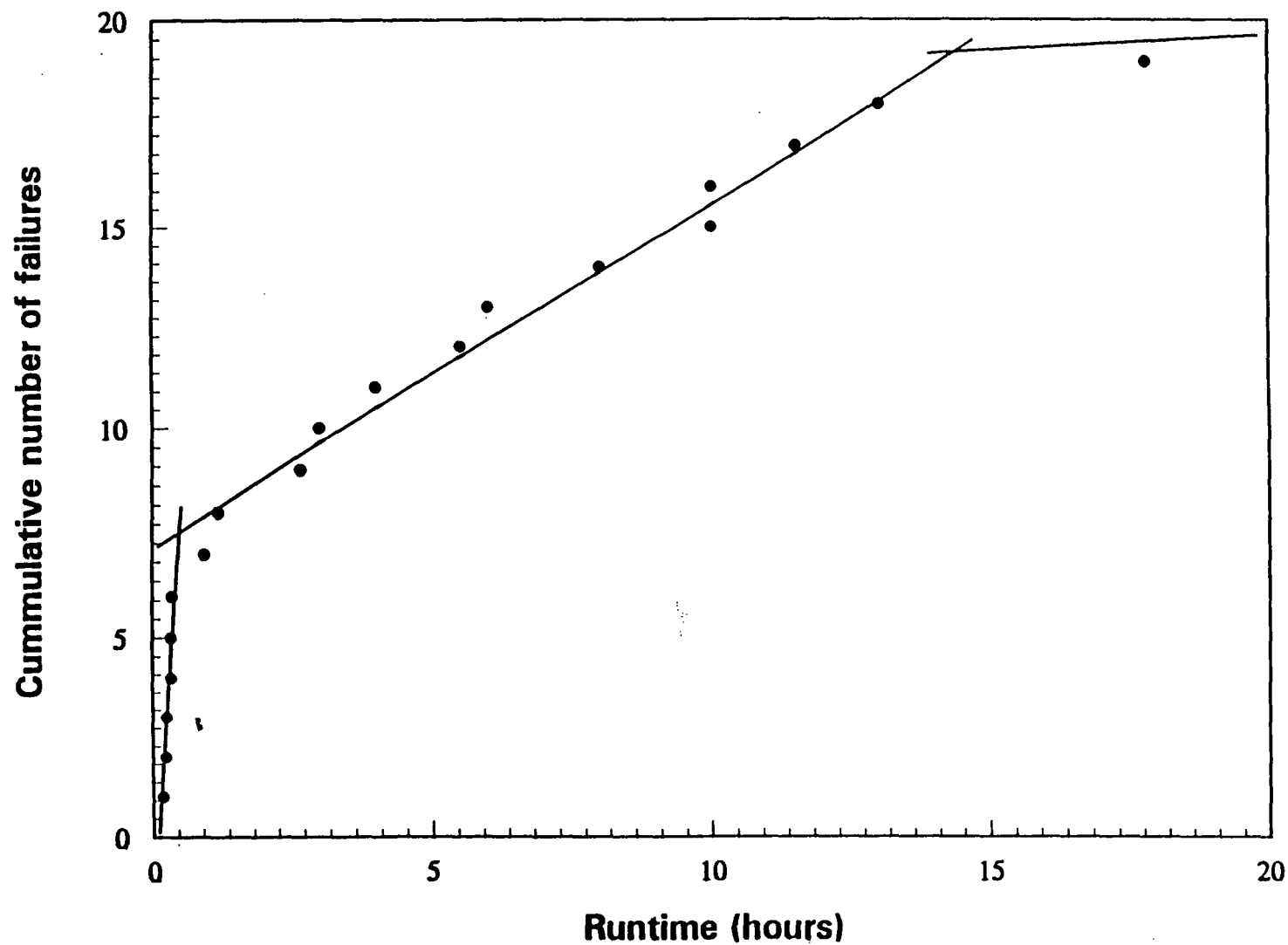


Train unreliability

PRELIMINARY RESULTS

EDG FAILURE-TO-RUN TREND

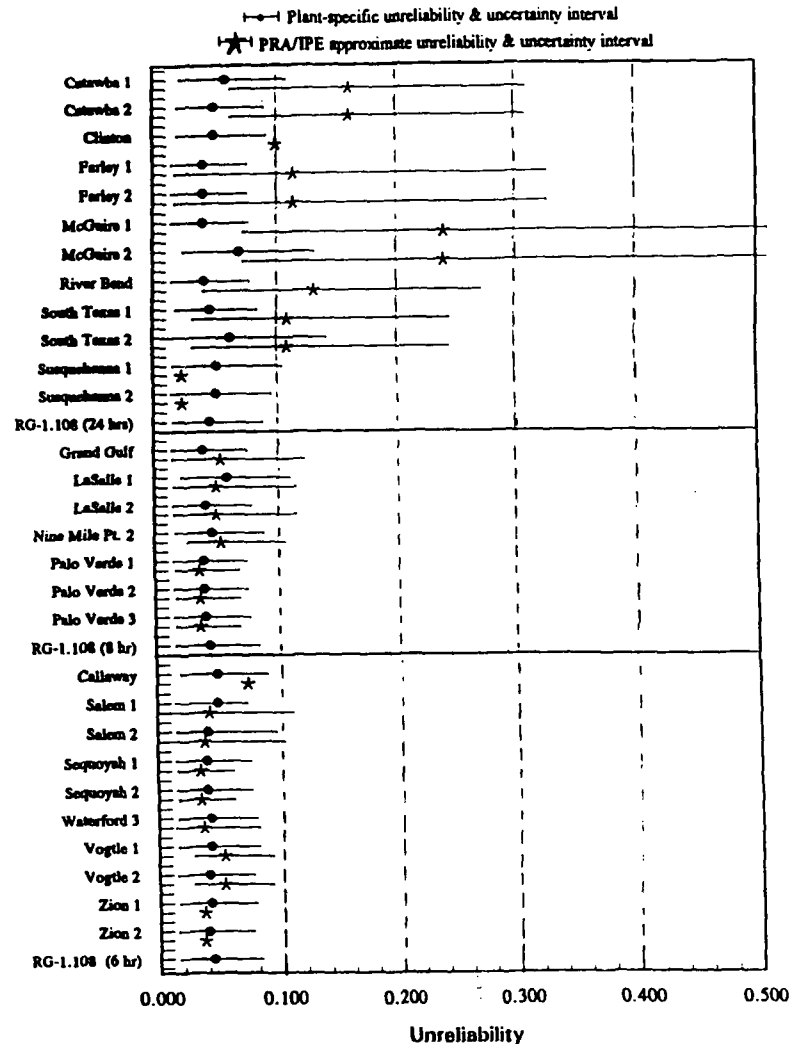
[Plants reporting per R.G. 1.108]



PRELIMINARY RESULTS

EDG UNRELIABILITY COMPARISONS ACTUAL EXPERIENCE VERSUS PRA/IPE VALUES

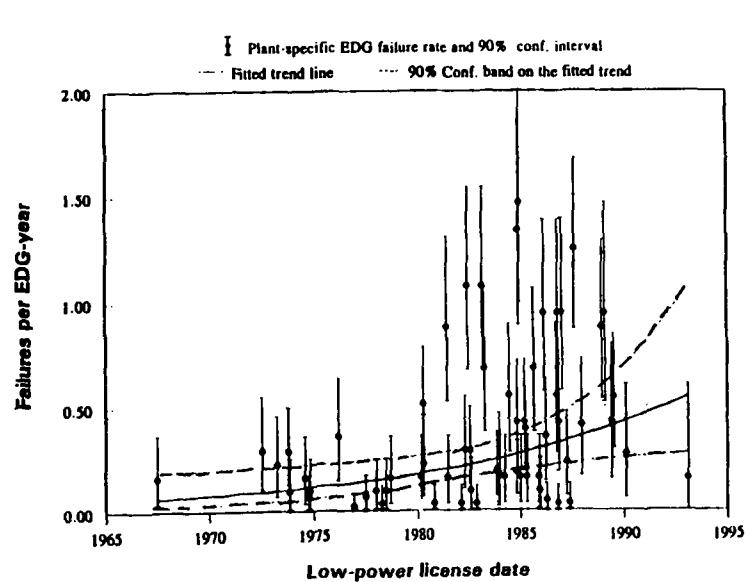
[Plants reporting per R.G. 1.108]



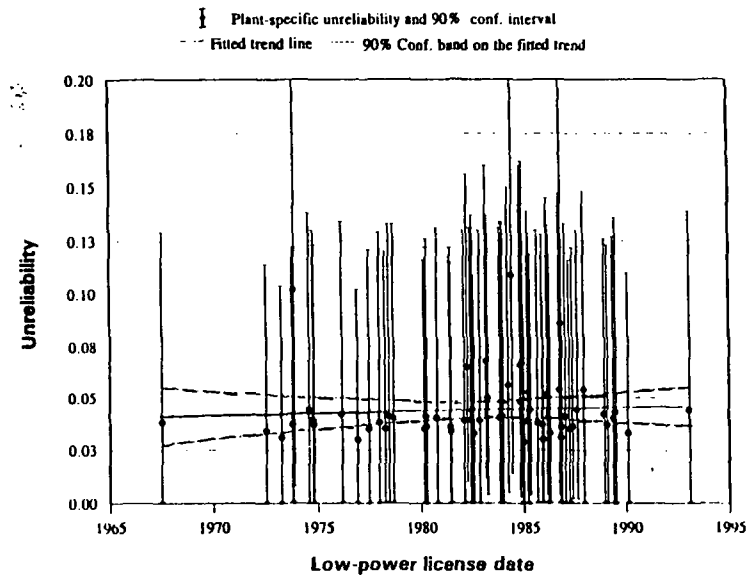
PRELIMINARY RESULTS

EDG TRENDS PER YEAR OF OPERATION

[Plants reporting per R.G. 1.108]



Train failure rate



Train Unreliability

PRELIMINARY RESULTS

EDG INSIGHTS

[Plants reporting per Regulatory Guide 1.108]

- No discernible trend in reliability
- Failure rate and unplanned demand rate both decreasing
- Higher failure rates for plants licensed from 1980 to 1990
- Three distinct failure-to-run rates
- General agreement with PRAs and IPEs
- Failure differences between actual demands and routine surveillance or inspections
- Actual demand failures-to-start not easily recoverable

PRELIMINARY RESULTS

EDG INSIGHTS (continued)

- No common cause failures of multiple diesels observed during actual unplanned demands
- Demand reliability (i.e., FTS and FTR) consistent with station blackout rule for plants reporting per Regulatory Guide 1.108
- MOOS much higher than earlier data used in station blackout rule
- MOOS significantly higher during shutdown than during operation for both Regulatory Guide 1.108 and non-Regulatory Guide 1.108 plants
- Unplanned demand data indicates reliability of plants not reporting per Regulatory Guide 1.108 consistent with Regulatory Guide 1.108 plants

USES

- Programmatic risk assessment of licensee performance
- Revise PI program to incorporate risk-significance indicators
- Independent check of the effectiveness of industry and Agency programs in reducing risk and increasing reliability using actual operating experience
- Input to the inspection and licensing programs of the Office of Nuclear Reactor Regulation (NRR)
- Updating simplified plant models used in Agency risk-significance estimations to make more plant specific

PROGRAM PLAN

Fiscal Year 1996

- Complete EDG, RCIC, IC, and initiating events reports
- Plan reliability rule database
- Issue PI report annually
- Complete CCF and update ASP and SCSS databases

Fiscal Year 1997

- Update FY-1995 and FY-1996 reports
- Complete RPS, BWR and PWR low pressure, PWR high pressure system reports
- Apply system report information to simplified plant models
- Construct and implement reliability rule database

Beyond Fiscal Year 1997

- Update previous analyses
- Incorporate reliability and availability data rule information
- Further reduce contractor support by performing tasks in-house