



TMI-15-072
June 10, 2015

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Three Mile Island Nuclear Station, Unit 1
Renewed Facility Operating License No. DPR-50
NRC Docket No. 50-289

Subject: Supplement to License Amendment Request – Modify Reactor Coolant System Pressure Isolation Check Valve Technical Specification Maximum Allowable Leakage Limits

References: 1. Letter from James Barstow (Exelon Generation Company, LLC) to U.S. NRC, "License Amendment Request – Modify Reactor Coolant System Pressure Isolation Check Valve Technical Specification Maximum Allowable Leakage Limits," dated October 30, 2014

Pursuant to 10 CFR 50.90, Exelon Generation Company, LLC (Exelon) is submitting a supplement to the October 30, 2014, (Reference 1) request for an amendment to the Technical Specifications (TS) for Three Mile Island Nuclear Station (TMI), Unit 1.

In the Reference 1 submittal, two items believed to be typographical errors were proposed for correction. It was subsequently determined that the errors exist in the Exelon Electronic Document Management System (EDMS) but do not exist on the controlled copies of the TMI Technical Specifications. Therefore, the proposed correction of typographical errors in TS 3.1.6.10.a (LIPS) and TS Table 3.1.6.1 (CF-VSA) is being withdrawn. The errors in the EDMS file have been entered into Exelon's Corrective Action Program for correction.

Attachment 1 provides a new mark-up of TS Page 3-15b. Attachment 2 is the final clean TS Page 3-15b for this license amendment request. With the exception of the typographical error, no other changes affected TS Section 3.1.6.10.a. Therefore, TS Page 3-13 has been removed from this submittal. No other revisions have been made to the proposed changes in the October 30, 2014, submittal.

Exelon has determined that the information provided in this supplemental letter does not impact the conclusions of the No Significant Hazards Consideration or Environmental Consideration as stated in Reference 1.

There are no regulatory commitments contained in this submittal.

In accordance with 10 CFR 50.91, "Notice for public comment; State consultation," paragraph (b), Exelon is notifying the Commonwealth of Pennsylvania of this application for license amendment by transmitting a copy of this letter and its attachments to the designated State Official.

Should you have any questions concerning this letter, please contact Frank Mascitelli at (610) 765-5512.

I declare under penalty of perjury that the foregoing is true and correct. Executed on the 10th day of June 2015.

Respectfully,



David P. Helker
Manager - Licensing and Regulatory Affairs
Exelon Generation Company, LLC

Attachment: 1. Proposed Technical Specification Marked-Up Page, Revised
2. Proposed Technical Specification – Clean Page

cc: USNRC Region I, Regional Administrator
USNRC Project Manager, TMI, Unit 1
USNRC Senior Resident Inspector, TMI, Unit 1
Director, Bureau of Radiation Protection, PA Department of Environmental Resources
Chairman, Board of County Commissioners, Dauphin County, PA
Chairman, Board of Supervisors, Londonderry Township, PA
R. R. Janati, Commonwealth of Pennsylvania

Attachment 1

Three Mile Island Nuclear Station, Unit 1

Supplement to License Amendment Request
Modify Reactor Coolant System Pressure Isolation Check Valve
Technical Specification Maximum Allowable Leakage Limits

Proposed Technical Specification Marked-Up Page, Revised

TABLE 3.1.6.1

PRESSURE ISOLATION CHECK VALVES BETWEEN
THE PRIMARY COOLANT SYSTEM & LPIS

<u>System</u>	<u>Valve No.</u>	<u>Maximum(a) Allowable Leakage</u>
Low Pressure Injection		(<5.0 GPM for all valves)
Train A	CF-V5A DH-V22A	(<5.0 GPM for all valves) ≤ 5.0 GPM
Train B	CF-V5B DH-V22B	(<5.0 GPM for all valves) ≤ 5.0 GPM

Footnote:

(a)

1. Leakage rates less than or equal to 1.0 gpm are considered acceptable.
2. Leakage rates greater than 1.0 gpm but less than or equal to 5.0 gpm are considered acceptable if the latest measured rate has not exceeded the rate determined by the previous test by an amount that reduces the margin between measured leakage rate and the maximum permissible rate of 5.0 gpm by 50% or greater.
3. Leakage rates greater than 1.0 gpm but less than or equal to 5.0 gpm are considered unacceptable if the latest measured rate exceeded the rate determined by the previous test by an amount that reduces the margin between measured leakage rate and the maximum permissible rate of 5.0 gpm by 50% or greater.
4. Leakage rates greater than 5.0 gpm are considered unacceptable.

Attachment 2

Three Mile Island Nuclear Station, Unit 1

Supplement to License Amendment Request
Modify Reactor Coolant System Pressure Isolation Check Valve
Technical Specification Maximum Allowable Leakage Limits

Proposed Technical Specification – Clean Page

TABLE 3.1.6.1

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THE PRIMARY COOLANT SYSTEM & LPIS

<u>System</u>	<u>Valve No.</u>	<u>Maximum Allowable Leakage</u>
Low Pressure Injection		
Train A	CF-V5A	≤5.0 GPM
	DH-V22A	≤5.0 GPM
Train B	CF-V5B	≤5.0 GPM
	DH-V22B	≤5.0 GPM