Exelon Generation

10 CFR 50.59
10 CFR 72.48

RA-15-042

May 26, 2015

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

Oyster Creek Nuclear Generating Station
Renewed Facility Operating License No. DPR-16
NRC Docket No. 50-219

Subject: Biennial 10 CFR 50.59 and 10 CFR 72.48 Change Summary Reports – January 1, 2013 through December 31, 2014

Enclosed are the Oyster Creek Nuclear Generating Station 10 CFR 50.59 and 10 CFR 72.48 Change Summary Reports for regulatory commitments changed during the period of January 1, 2013 through December 31, 2014.

There are no regulatory commitments contained in this submittal.

Please contact Thomas Cappuccino at (609) 971-4430 if any further information or assistance is needed.

Sincerely,

Gary L. Stathes

Garey L. Stathes
Vice President
Oyster Creek Nuclear Generating Station

Enclosure

cc: Administrator, USNRC Region I
USNRC Senior Project Manager, Oyster Creek
USNRC Senior Resident Inspector, Oyster Creek

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# Exelon Generation Company, LLC
# Oyster Creek Nuclear Generating Station
# Enclosure to RA-15-42

Docket No. 50-219

## 2013 – 2014
## Biennial 10 CFR 50.59 and 10 CFR 72.48 Change Summary Reports

These summary reports are issued pursuant to reporting requirements for Oyster Creek Nuclear Generating Station (OCNGS). These reports address tests, experiments, and changes to the facility and/or procedures as they are described in the Final Safety Analysis Report for the OCNGS station and the Final Safety Analysis Report for the Standardized NUHOMS Horizontal Modular Storage System (TN-61BT Spent Fuel Cask at OCNGS). These reports summarize the three (3) tests, experiments, and changes that were implemented between January 1, 2013 and December 31, 2014 under 10 CFR 50.59. There were no tests, experiments, or changes implemented by OCNGS under 10 CFR 72.48.

## Item # 1

**Evaluation Number:** OC-2013-E-0001, Rev.0
**PORC Review Meeting No. (Date):** 13-08 (08/30/13)
**Activity/Document No.:** Temporary Modification / ECR # 13-00378, Rev.0

**Title:** Trunnion Room Temperature Main Steam Line Isolation

**Description of Activity:**

ECR OC 13-00378 allows Operations to install a jumper across three of the four temperature switches in any one of four Reactor Protection System (RPS) high temperature isolation circuits. This is a temporary jumper that is installed as a compensatory measure to reduce the challenges with regard to spurious isolations. This temporary jumper is to be removed after repairs have been made to restore the system to its design condition no later than 1R25.

**Reason for Activity:**

RPS 1 and 2 have been experiencing spurious half scram isolations due to an intermittent ground in one of the main steam line temperature sensors, which are located in the Condenser Bay. Steam leaks in the Condenser Bay are believed to be the cause of the intermittent grounding. Therefore, the purpose of the jumper is to prevent spurious initiation of the affected RPS sub-channel.

**Effect of Activity:**

Implementing ECR OC 13-00378 protects against a full reactor scram due to simultaneous spurious half isolations in RPS 1 and RPS 2 Trunnion Room high temperature isolation circuits. Implementing ECR OC 13-00378 does not prevent a full reactor scram if valid trip signals are received on the Trunnion Room high temperature circuits. Implementing ECR OC 13-00378 does not have any impact on any plant design basis or safety analysis as described in the UFSAR.

**Summary of Conclusion for the Activity's 50.59 Review:**

Implementing ECR OC 13-00378 does not result in more than a minimal increase in the frequency of an occurrence of an accident previously evaluated in the UFSAR, nor does it result in more than a minimal increase in the likelihood of occurrence of a malfunction of an SSC important to safety previously evaluated in the UFSAR. Implementing ECR OC 13-00378 does not result in more than a minimal increase in the consequences of an accident previously evaluated in the UFSAR, nor does it result in more than a minimal increase in the consequences of a malfunction of an SSC important to safety previously evaluated in the UFSAR. Implementing ECR OC 13-00378 does not create the possibility for an accident of a different type than any previously evaluated in the UFSAR, nor does it create a possibility for a malfunction of an SSC important to safety with a different result than any previously evaluated in the UFSAR. Implementing ECR OC 13-00378 does not result in a design basis limit for a fission product barrier as described in the UFSAR being exceeded or altered. Implementing ECR OC 13-00378

does not result in a departure from a method of evaluation described in the UFSAR used in establishing the design bases or in the safety analyses for Oyster Creek.

Therefore, NRC approval is not required prior to implementing ECR OC 13-00378.

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## Item # 2

**Evaluation Number:** OC-2013-E-0002, Rev.0
**PORC Review Meeting No. (Date):** 13-10 (09/23/13)
**Activity/Document No.:** Temporary Modification / ECR # 13-00378, Rev.2

**Title:** TCCP to Jumper Out TS-IB0010E and TS-IB0010F

**Description of Activity:**

This activity is a Temporary Configuration Change Package (TCCP) to jumper out temperature switches TS IB0010E, MSL TUNNEL HI TEMPERATURE SWITCH CLOSES MSIV'S, and TS-IB0010F, MSL TUNNEL HI TEMPERATURE SWITCH CLOSES MSIV'S. The field wires in the switch housings will be lifted and connected together, effectively jumpering out each temperature switch.

TS-IB0010E and TS-IB0010F are described in the UFSAR, in the attached 50.59 Screening, and in the attached 50.59 Evaluation as "Trunnion Room High Temperature" switches. Both switches are located in the Main Steam Line Tunnel in the Condenser Bay.

**Reason for Activity:**

RPS System I has been experiencing spurious half isolations. Troubleshooting isolated TS-IB0010E and TS-IB0010F. This TCCP is to keep the switches isolated until repairs can be performed in a plant outage.

**Effect of Activity:**

This TCCP does not affect plant operations. It does reduce the number of temperature switches for high Trunnion Room temperature in RPS System 1 from 8 to 6. It reduces the TSs in each RPS sub-channel from 4 to 3. This is acceptable per Technical Specification, which requires only one TS in each sub-channel. However, the TCCP does require a 50.59 Evaluation because it involves a change to an SSC that adversely affects a UFSAR described function.

**Summary of Conclusion for the Activity's 50.59 Review:**

This activity requires a 50.59 Evaluation because it involves a change to an SSC that adversely affects a UFSAR described function.

This activity does not result in more than a minimal increase in the frequency of occurrence of an accident previously evaluated in the UFSAR. It does not result in more than a minimal increase in the likelihood of occurrence of a malfunction of an SSC important to safety previously evaluated in the UFSAR. It does not result in more than a minimal increase in the consequences of an accident previously evaluated in the UFSAR. It does not result in more than a minimal increase in the consequences of a malfunction of an SSC important to safety previously evaluated in the UFSAR. It does not create a possibility for an accident of a different type than any previously evaluated in the UFSAR. It does not create a possibility for a malfunction of an SSC important to safety with a different result than any previously evaluated in UFSAR. It does not result in a design basis limit for a fission product barrier as described in the UFSAR being exceeded or altered. It does not result in a departure from a method of evaluation described in the UFSAR used in establishing the design bases or in the safety analyses.

Therefore, prior NRC approval before implementation of ECR 13-00378 Rev 2 is not required.

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## Item # 3

**Evaluation Number:** OC-2014-E-0002, Rev.0
**PORC Review Meeting No. (Date):** 14-07 (07/09/14)
**Activity/Document No.:** Temporary Modification / ECR # 14-00340, Rev.0

**Title:** Gag for Damper DM-838-11

**Description of Activity:**

ECR OC 14-00340 installs a gag to maintain Damper DM-838-11 in the open position during normal plant operation. Damper DM-838-11 is the lower inlet damper to Drywell Recirculation Fan RF-1-4.

**Reason for Activity:**

During a recent inspection of the drywell recirculation fans, it was observed that Damper DM-838-11 was found closed and not functioning. See IR 1679878. A closed damper to a drywell recirculation fan reduces the fan's ability to cool the drywell effectively.

**Effect of Activity:**

Installing a gag on Damper DM-838-11 ensures that the damper is open fully during fan operation and improves cooling of the drywell during plant operation. Installing this gag also prevents Damper DM-838-11 from closing should Drywell Recirculation Fan RF-1-4 trip.

**Summary of Conclusion for the Activity's 50.59 Review:**

If a 50.59 Screening was performed, Questions 1 and 2 would have been answered 'Yes' since ECR OC 14-00340 does remove an automatic function to close Damper DM-838-11 resulting from a trip of Drywell Recirculation Fan RF-1-4.

Implementing ECR OC 14-00340 does not result in more than a minimal increase in the frequency of an accident previously evaluated in the UFSAR, nor does it does not result in more than a minimal increase in the likelihood of occurrence of a malfunction of an SSC important to safety previously evaluated in the UFSAR. Implementing ECR OC 14-00340 does not result in more than a minimal increase in the consequences of an accident previously evaluated in the UFSAR, nor does result in more than a minimal increase in the consequences of a malfunction of an SSC important to safety previously evaluated in the UFSAR. Implementing ECR OC 14-00340 does not create a possibility for an accident of a different type than any previously evaluated in the UFSAR, nor does it create a possibility for a malfunction of an SSC important to safety with a different result than any previously evaluated in UFSAR. Implementing ECR OC 14-00340 does not result in a design basis limit fission product barrier (DBLFPB) as described in the UFSAR being exceeded or altered, nor does it result in a departure from a method of evaluation described in the UFSAR used in establishing the design bases or in the safety analyses.

ECR OC 14-00340 does not impact the Oyster Creek Technical Specifications and Operating License.

Therefore, ECR OC 14-00340 can be implemented without obtaining previous NRC approval.

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**Note:** Evaluation # OC-2014-E-0001 was never issued. The proposed activity (Temporary modification / ECR # 14-00323) was voided (cancelled) before ever being approved for installation. As such, this Evaluation was not required.