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ATTENTION: "REPLACE" directions do not affect the Table of Contents, Therefore no TOC will be issued with the updated material.

TRM1 - TECHNICAL REQUIREMENTS MANUAL UNIT 1

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A001
NRR

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ANY DISCREPANCIES WITH THE MATERIAL PROVIDED, CONTACT DCS @ X3107 OR X3136 FOR ASSISTANCE. UPDATES FOR HARDCOPY MANUALS WILL BE DISTRIBUTED WITHIN 3 DAYS IN ACCORDANCE WITH DEPARTMENT PROCEDURES. PLEASE MAKE ALL CHANGES AND ACKNOWLEDGE COMPLETE IN YOUR NIMS INBOX UPON COMPLETION OF UPDATES. FOR ELECTRONIC MANUAL

Apr. 29, 2015

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USERS, ELECTRONICALLY REVIEW THE APPROPRIATE DOCUMENTS AND ACKNOWLEDGE COMPLETE IN YOUR NIMS INBOX.

SSES MANUAL

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Manual Title: TECHNICAL REQUIREMENTS MANUAL UNIT 1

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TRM1 text LOES
4/23/2015

2.0 PLANT PROGRAMS AND SETPOINTS

2.1 Plant Programs

Table 2.1-1 lists the Plant Programs required as a result of the plant Technical Specification or other Plant License condition. Each program listing identifies the program source and implementing documents and is intended to provide an index to these documents only.

Table 2.1-1
PLANT PROGRAMS
(Page 1 of 3)

PROGRAM DESIGNATION		PROGRAM SOURCE	IMPLEMENTING DOCUMENT	
2.1.1	Offsite Dose Calculation Manual (ODCM)	Tech Spec 5.5.1	ODCM-QA-001	ODCM Introduction
			ODCM-QA-002	ODCM Review and Revision Control
			ODCM-QA-003	Effluent Monitor Setpoints
			ODCM-QA-004	Airborne Effluent Dose Calculations
			ODCM-QA-005	Waterborne Effluent Dose Calculations
			ODCM-QA-006	Total Dose Calculations
			ODCM-QA-007	Radioactive Waste Treatment Systems
			ODCM-QA-008	Radiological Environmental Monitoring Program
			ODCM-QA-009	Dose Assessment Policy Statements
2.1.2	Primary Coolant Sources Outside Containment	Tech Spec 5.5.2	NSEP-QA-401	System Leakage Quantification Program
2.1.3	Not Used			

(continued)

Table 2.1-1
PLANT PROGRAMS
(Page 2 of 3)

PROGRAM DESIGNATION		PROGRAM SOURCE	IMPLEMENTING DOCUMENT	
2.1.4	Radioactive Effluent Controls	Tech Spec 5.5.4	TRO 2.1.1 ODCM TRO 3.6.1 Containment VENTING and PURGING TRO 3.11 Series RADIOACTIVE EFFLUENTS	
2.1.5	Component Cyclic or Transient Limit	Tech Spec 5.5.5	NEPM-QA-0901	Plant Transient and Fatigue Monitoring Program
2.1.6	Inservice Testing	Tech Spec 5.5.6	NDAP-QA-0423	Station Pump and Valve Testing Program
2.1.7	Ventilation Filter Testing	Tech Spec 5.5.7	NDAP-QA-0407	Filter Testing of HEPA and Charcoal Filtration Units
2.1.8	Explosive Gas and Storage Tank Radioactivity Monitoring	Tech Spec 5.5.8	NDAP-QA-1180 TRO 3.7.5.1 TRO 3.7.5.2	Storage Tank Radioactivity Monitoring Program Main Condenser Offgas Hydrogen Monitor Main Condenser Explosive Gas Mixture

(continued)

Table 2.1-1
PLANT PROGRAMS
(Page 3 of 3)

PROGRAM DESIGNATION		PROGRAM SOURCE	IMPLEMENTING DOCUMENT	
2.1.9	Diesel Fuel Oil Testing	Tech Spec 5.5.9	NDAP-QA-0633	Diesel Fuel Oil Testing
2.1.10	Technical Specification (TS) Bases Control	Tech Spec 5.5.10	NDAP-QA-0730	Controlling Changes to Licensing Documents
2.1.11	Safety Function Determination	Tech Spec 5.5.11	NDAP-QA-0312	Safety Function Determination Program
2.1.12	Primary Containment Leakage Rate Testing	Tech Spec 5.5.12	NDAP-QA-0412	Leakage Rate Test Program
2.1.13	Fire Protection	Item d; Tech Spec 5.4.1	NDAP-QA-0449	Fire Protection Program

3.2 Core Operating Limits Report (COLR)

3.2.1 Core Operating Limits Report (COLR)

TRO 3.2.1 The Core Operating Limits specified in the attached COLR shall be met.

APPLICABILITY: Specified in the referenced Technical Specifications.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Core Operating Limits not met.	A.1 Perform action(s) described in referenced Technical Specification.	Specified in referenced Technical Specifications.

TECHNICAL REQUIREMENT SURVEILLANCE

SURVEILLANCE	FREQUENCY
<div>-----NOTE----- No associated Surveillances. Surveillances are implemented in the applicable Technical Specifications. -----</div>	N/A

Susquehanna SES Unit 1 Cycle 19

CORE OPERATING LIMITS REPORT

**Nuclear Fuels
Engineering**

April 2015



**CORE OPERATING LIMITS REPORT
REVISION DESCRIPTION INDEX**

Rev. No.	Affected Sections	Description/Purpose of Revision
0	ALL	Issuance of this COLR is in support of Unit 1 Cycle 19 operation.
1	ALL	New COLR required due to Power/Flow Map revision.

**SUSQUEHANNA STEAM ELECTRIC STATION
Unit 1 Cycle 19
CORE OPERATING LIMITS REPORT**

Table of Contents

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5.0	<u>MINIMUM CRITICAL POWER RATIO (MCPR)</u>	9
6.0	<u>LINEAR HEAT GENERATION RATE (LHGR)</u>	24
7.0	<u>ROD BLOCK MONITOR (RBM) SETPOINTS AND OPERABILITY REQUIREMENTS</u>	32
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1.0 INTRODUCTION

This CORE OPERATING LIMITS REPORT for Susquehanna Unit 1 Cycle 19 is prepared in accordance with the requirements of Susquehanna Unit 1, Technical Specification 5.6.5. As required by Technical Specifications 5.6.5, core shutdown margin, the core operating limits, RBM setpoints, and OPRM setpoints presented herein were developed using NRC-approved methods and are established such that all applicable limits of the plant safety analysis are met.

2.0 DEFINITIONS

Terms used in this COLR but not defined in Section 1.0 of the Technical Specifications or Section 1.1 of the Technical Requirements Manual are provided below.

- 2.1 The AVERAGE PLANAR EXPOSURE at a specified height shall be equal to the total energy produced per unit length at the specified height divided by the total initial weight of uranium per unit length at that height.
- 2.2 The PELLET EXPOSURE shall be equal to the total energy produced per unit length of fuel rod at the specified height divided by the total initial weight of uranium per unit length of that rod at that height.
- 2.3 FDLRX is the ratio of the maximum LHGR calculated by the core monitoring system for each fuel bundle divided by the LHGR limit for the applicable fuel bundle type.
- 2.4 LHGRFAC_f is a multiplier applied to the LHGR limit when operating at less than 108 Mlbm/hr core flow. The LHGRFAC_f multiplier protects against both fuel centerline melting and cladding strain during anticipated system transients initiated from core flows less than 108 Mlbm/hr.
- 2.5 LHGRFAC_p is a multiplier applied to the LHGR limit when operating at less than RATED THERMAL POWER. The LHGRFAC_p multiplier protects against both fuel centerline melting and cladding strain during anticipated system transients initiated from partial power conditions.
- 2.6 MFLCPR is the ratio of the applicable MCPR operating limit for the applicable fuel bundle type divided by the MCPR calculated by the core monitoring system for each fuel bundle.
- 2.7 MAPRAT is the ratio of the maximum APLHGR calculated by the core monitoring system for each fuel bundle divided by the APLGHR limit for the applicable fuel bundle type.
- 2.8 OPRM is the Oscillation Power Range Monitor. The Oscillation Power Range Monitor (OPRM) will reliably detect and suppress anticipated stability related power oscillations while providing a high degree of confidence that the MCPR safety limit is not violated.
- 2.9 N_p is the OPRM setpoint for the number of consecutive confirmations of oscillation half-cycles that will be considered evidence of a stability related power oscillation.
- 2.10 S_p is the OPRM trip setpoint for the peak to average OPRM signal.
- 2.11 F_p is the core flow, in Mlbm / hr, below which the OPRM RPS trip is activated.

3.0 SHUTDOWN MARGIN

3.1 Technical Specification Reference

Technical Specification 3.1.1

3.2 Description

The SHUTDOWN MARGIN shall be equal to or greater than:

- a) 0.38% $\Delta k/k$ with the highest worth rod analytically determined

OR

- b) 0.28% $\Delta k/k$ with the highest worth rod determined by test

Since core reactivity will vary during the cycle as a function of fuel depletion and poison burnup, Beginning of Cycle (BOC) SHUTDOWN MARGIN (SDM) tests must also account for changes in core reactivity during the cycle. Therefore, the SDM measured at BOC must be equal to or greater than the applicable requirement from either 3.2.a or 3.2.b plus an adder, "R". The adder, "R", is the difference between the calculated value of maximum core reactivity (that is, minimum SDM) during the operating cycle and the calculated BOC core reactivity. If the value of "R" is zero (that is, BOC is the most reactive point in the cycle) no correction to the BOC measured value is required.

The SHUTDOWN MARGIN limits provided in 3.2a and 3.2b are applicable in MODES 1, 2, 3, 4, and 5. This includes core shuffling.

4.0 AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)

4.1 Technical Specification Reference

Technical Specification 3.2.1

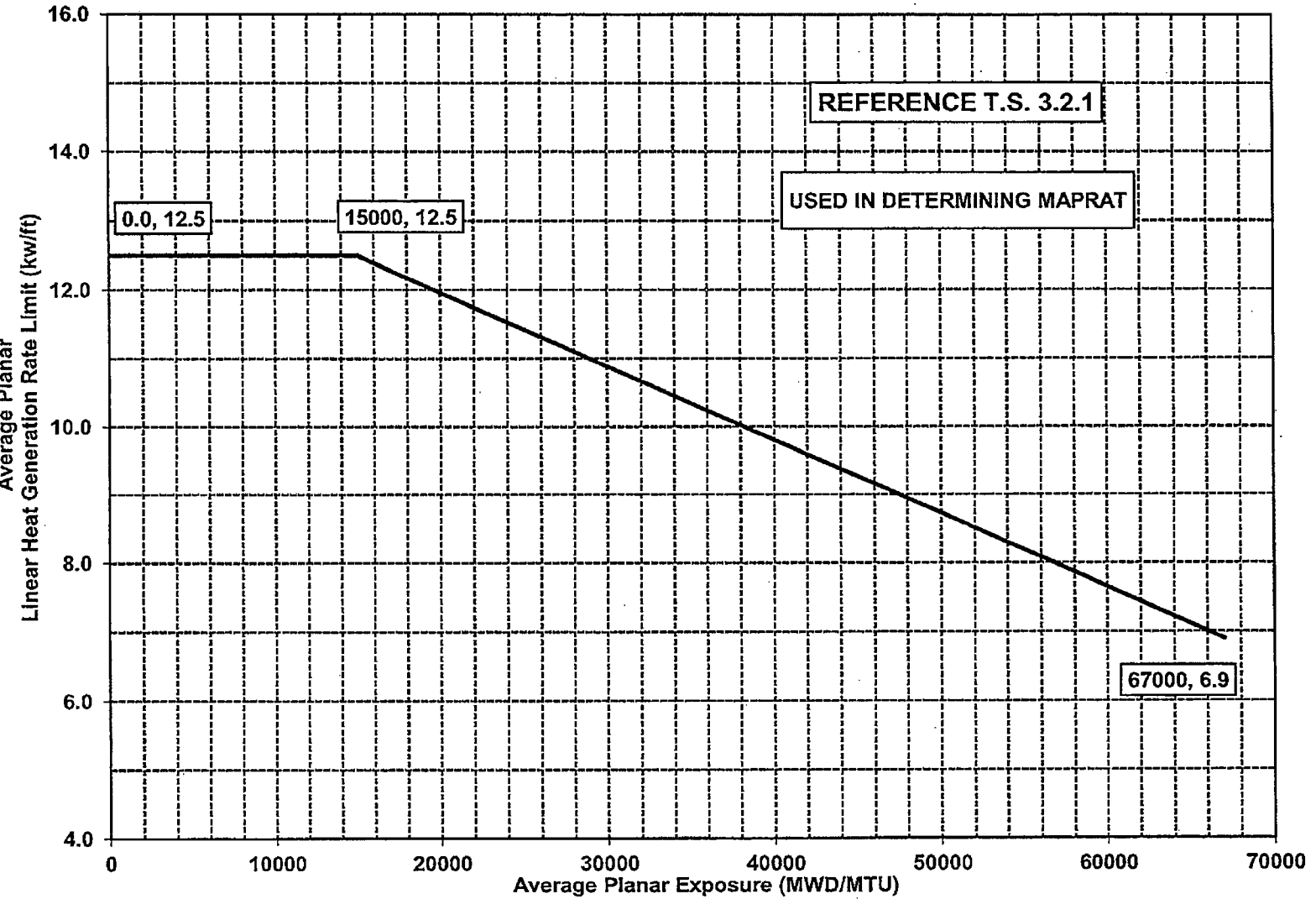
4.2 Description

The APLHGRs for ATRIUMTM-10 fuel shall not exceed the limit shown in Figure 4.2-1.

The APLHGR limits in Figure 4.2-1 are valid for Main Turbine Bypass Operable and Inoperable, EOC-RPT Operable and Inoperable, and Backup Pressure Regulator Operable and Inoperable in Two Loop operation. The APLHGR limits for Single Loop operation are provided in Section 8.0.

SSSES UNIT 1 CYCLE 19

SUSQUEHANNA UNIT 1
TRM/3.2-9
EFFECTIVE DATE 04/21/2015



AVERAGE PLANAR LINEAR HEAT GENERATION RATE LIMIT VERSUS
AVERAGE PLANAR EXPOSURE - TWO LOOP OPERATION
ATRIUM™-10 FUEL
FIGURE 4.2-1

5.0 MINIMUM CRITICAL POWER RATIO (MCPR)

5.1 Technical Specification Reference

Technical Specification 3.2.2, 3.3.4.1, 3.7.6, and 3.7.8

5.2 Description

The MCPR limit is specified as a function of core power, core flow, average scram insertion time per Section 5.3 and plant equipment operability status. The MCPR limits for all fuel types (ATRIUM™-10) shall be the greater of the Flow-Dependent or the Power-Dependent MCPR, depending on the applicable equipment operability status.

a) Main Turbine Bypass / EOC-RPT / Backup Pressure Regulator Operable

Figure 5.2-1: Flow-Dependent MCPR value determined from BOC to EOC

Figure 5.2-2: Power-Dependent MCPR value determined from BOC to EOC

b) Main Turbine Bypass Inoperable

Figure 5.2-3: Flow-Dependent MCPR value determined from BOC to EOC

Figure 5.2-4: Power-Dependent MCPR value determined from BOC to EOC

c) EOC-RPT Inoperable

Figure 5.2-5: Flow-Dependent MCPR value determined from BOC to EOC

Figure 5.2-6: Power-Dependent MCPR value determined from BOC to EOC

d) Backup Pressure Regulator Inoperable

Figure 5.2-7: Flow-Dependent MCPR value determined from BOC to EOC

Figure 5.2-8: Power Dependent MCPR value determined from BOC to EOC

The MCPR limits in Figures 5.2-1 through 5.2-8 are valid for Two Loop operation.

The MCPR limits for Single Loop operation are provided in Section 8.0.

5.3 Average Scram Time Fraction

If the average measured scram times are greater than the Realistic Scram times listed in Table 5.3-1 then the MCPR operating limits corresponding to the Maximum Allowable Average Scram Insertion Time must be implemented. Determining MCPR operating limits based on interpolation between scram insertion times is not permitted. The evaluation of scram insertion time data, as it relates to the attached table should be performed per Reactor Engineering procedures.

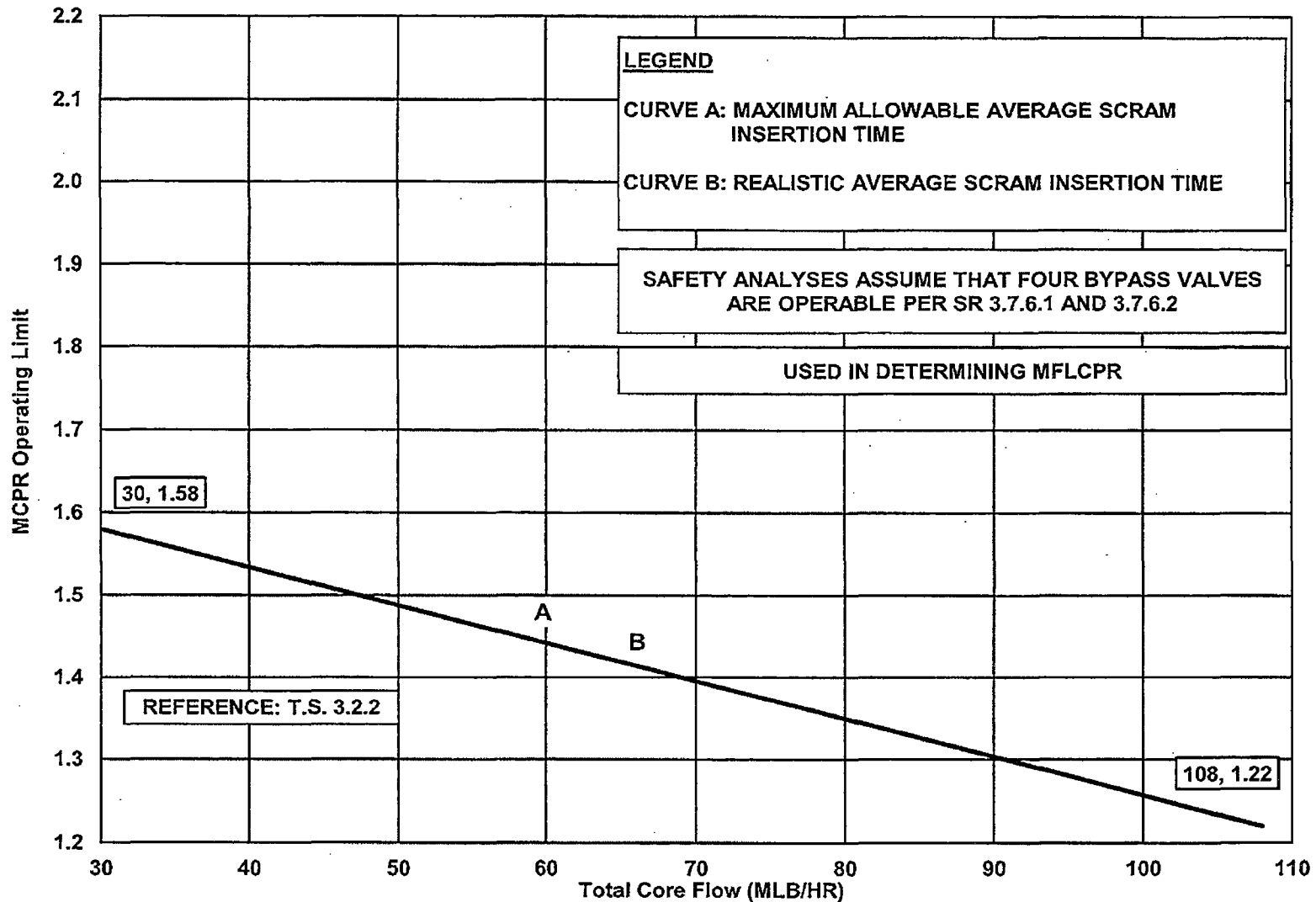
Main Turbine Bypass / EOC-RPT / Backup Pressure Regulator Operable

SSS UNIT 1 CYCLE 19

SUSQUEHANNA UNIT 1

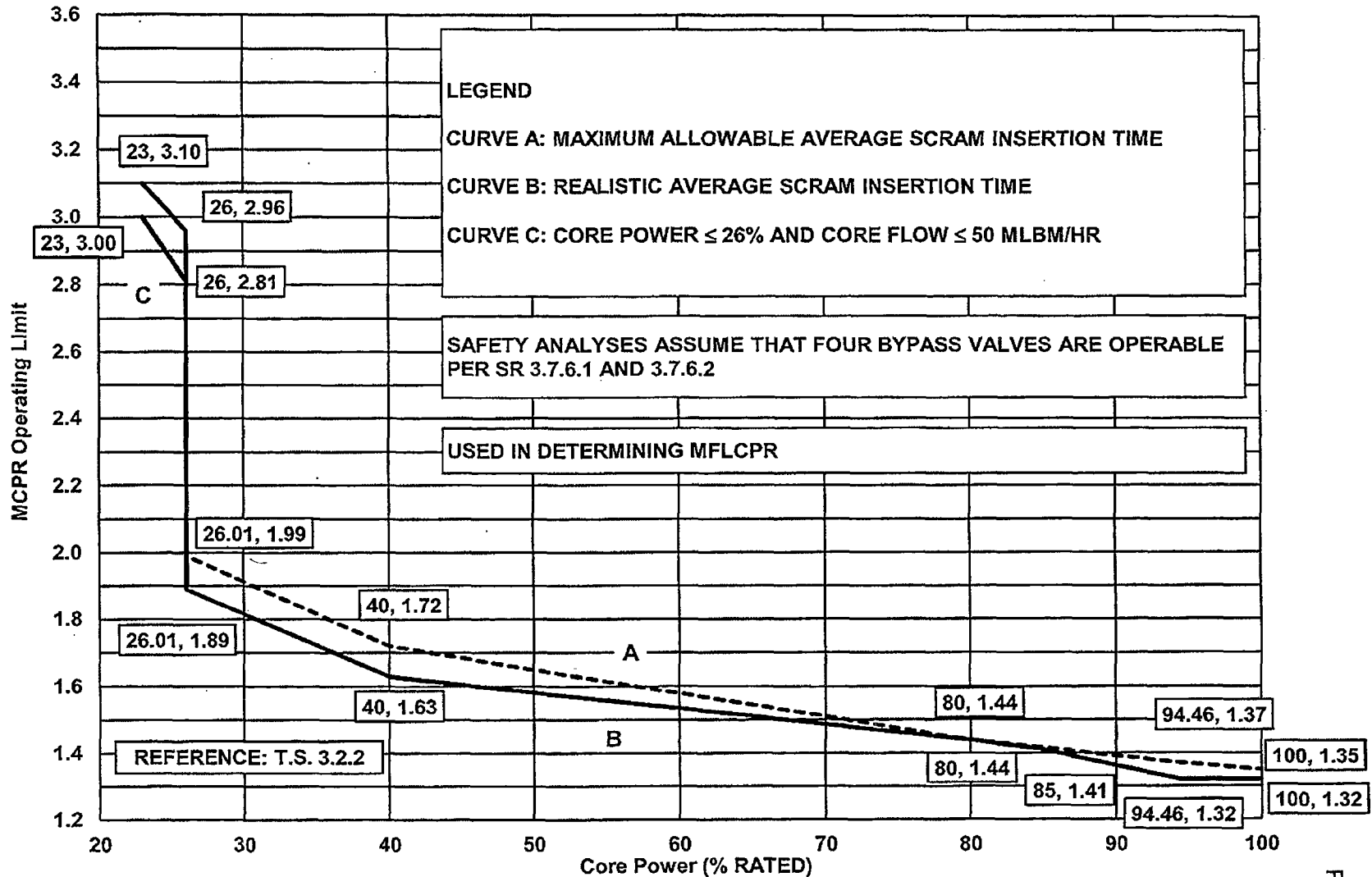
TRM/3.2-13

EFFECTIVE DATE 04/21/2015



MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
 MAIN TURBINE BYPASS / EOC-RPT / BACKUP PRESSURE REGULATOR OPERABLE
 TWO LOOP OPERATION (BOC TO EOC)
 FIGURE 5.2-1

SSSES UNIT 1 CYCLE 19



MCPR OPERATING LIMIT VERSUS CORE POWER
MAIN TURBINE BYPASS / EOC-RPT / BACKUP PRESSURE REGULATOR OPERABLE
TWO LOOP OPERATION (BOC TO EOC)
FIGURE 5.2-2

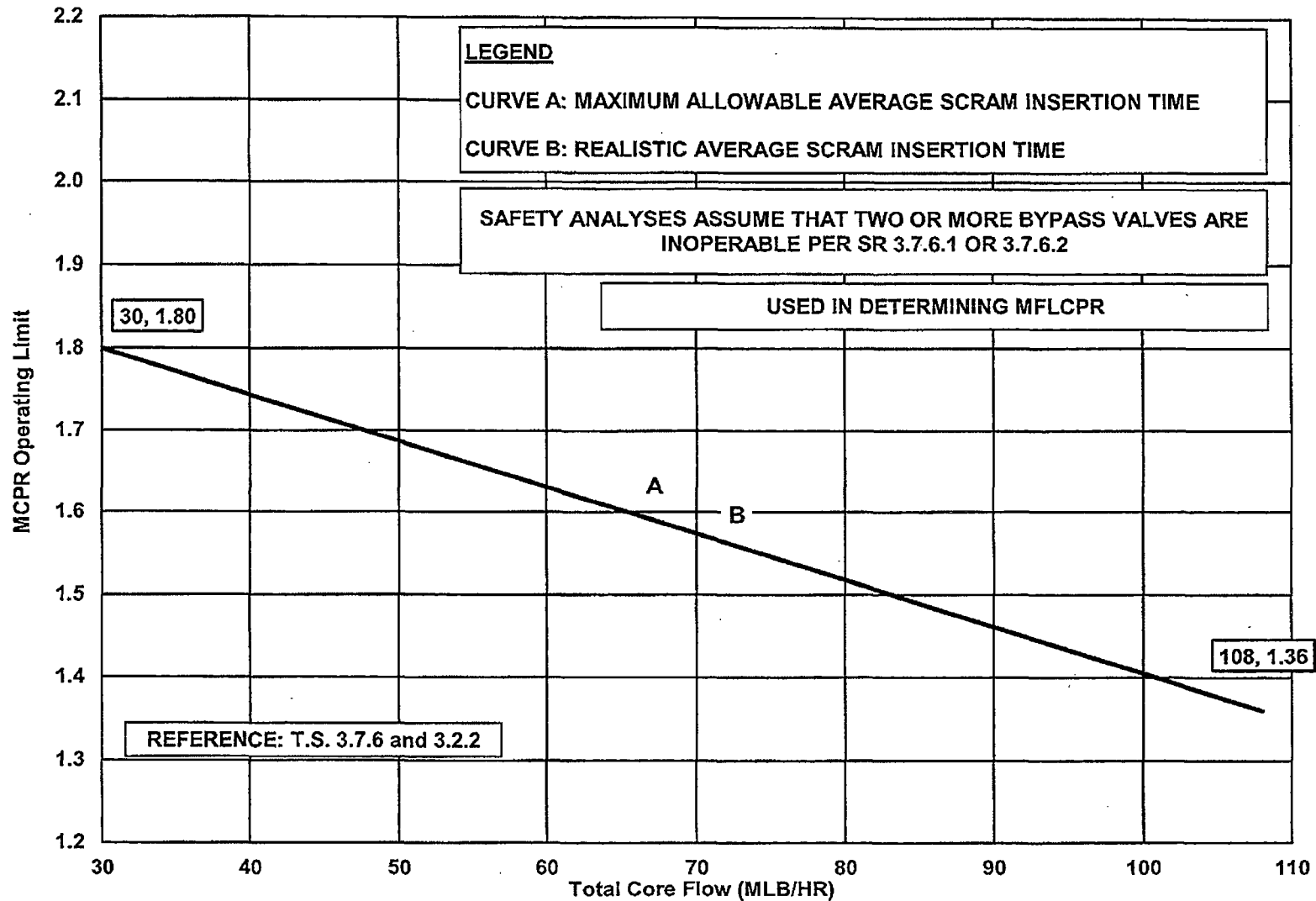
Main Turbine Bypass Inoperable

SSES UNIT 1 CYCLE 19

SUSQUEHANNA UNIT 1

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EFFECTIVE DATE 04/21/2015



MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
MAIN TURBINE BYPASS INOPERABLE
TWO LOOP OPERATION (BOC TO EOC)
FIGURE 5.2-3

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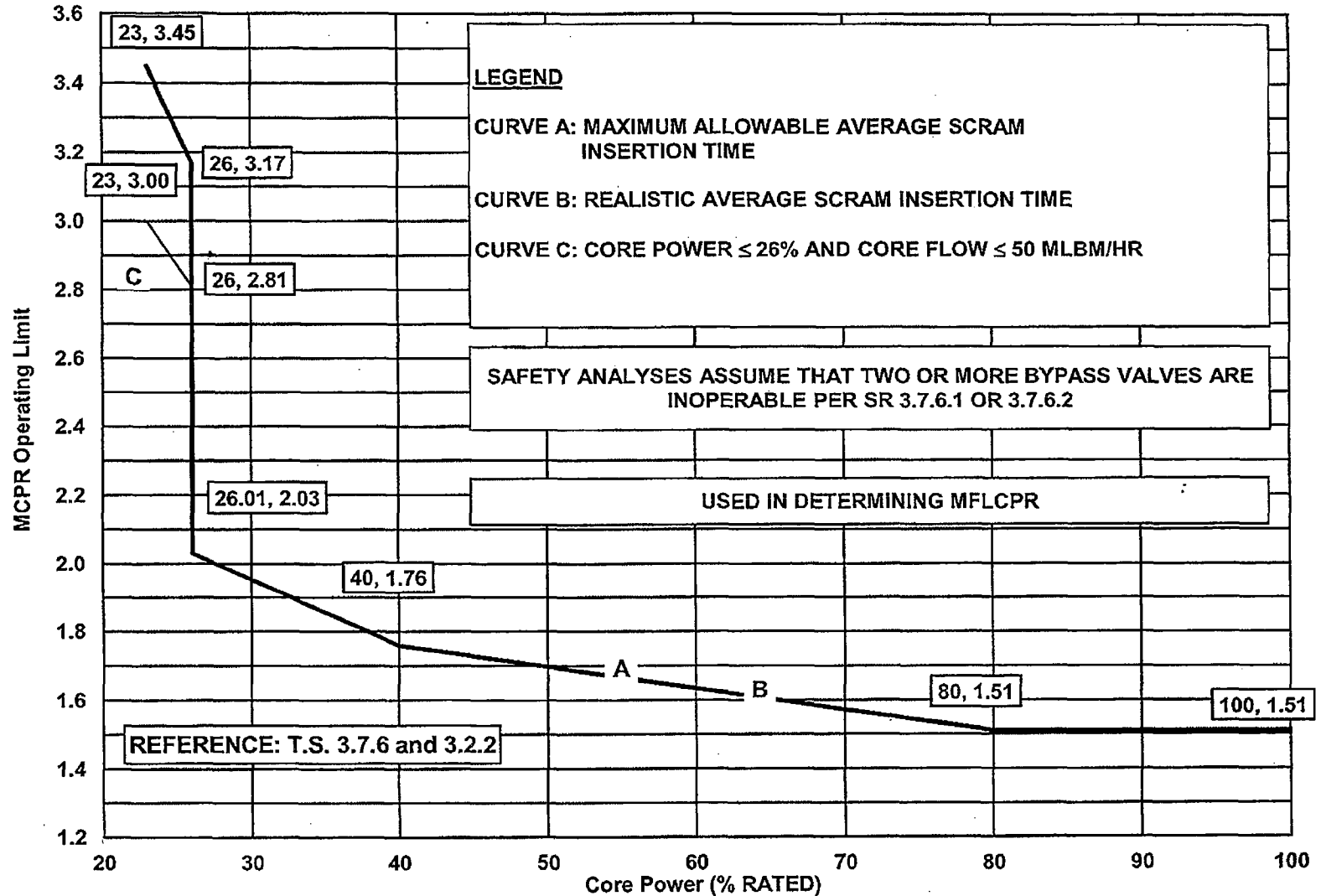
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SUSQUEHANNA UNIT 1

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EFFECTIVE DATE 04/21/2015



MCPR OPERATING LIMIT VERSUS CORE POWER
MAIN TURBINE BYPASS INOPERABLE
TWO LOOP OPERATION (BOC to EOC)
FIGURE 5.2-4

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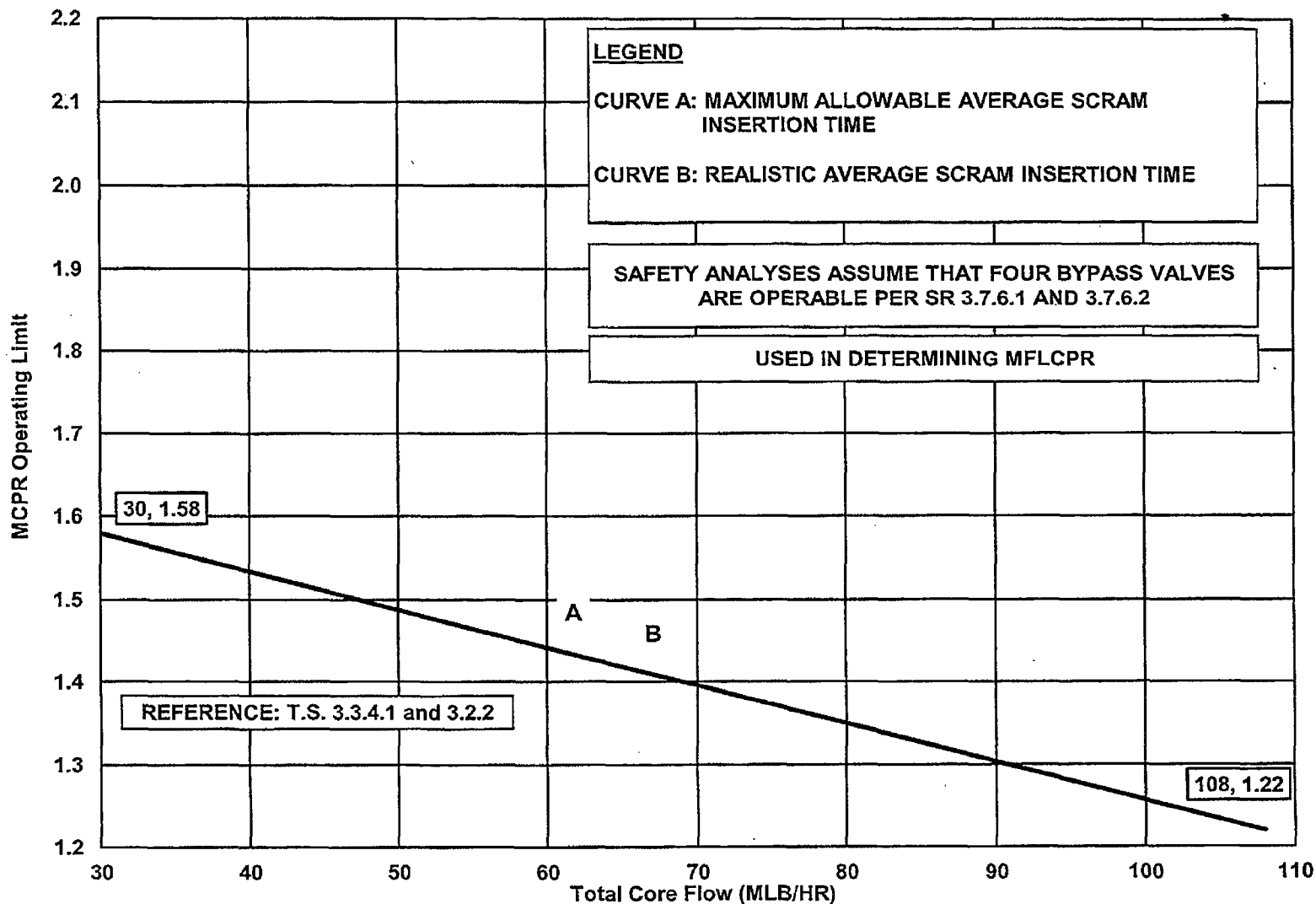
EOC-RPT Inoperable

SSSES UNIT 1 CYCLE 19

SUSQUEHANNA UNIT 1

TRM/3.2-19

EFFECTIVE DATE 04/21/2015



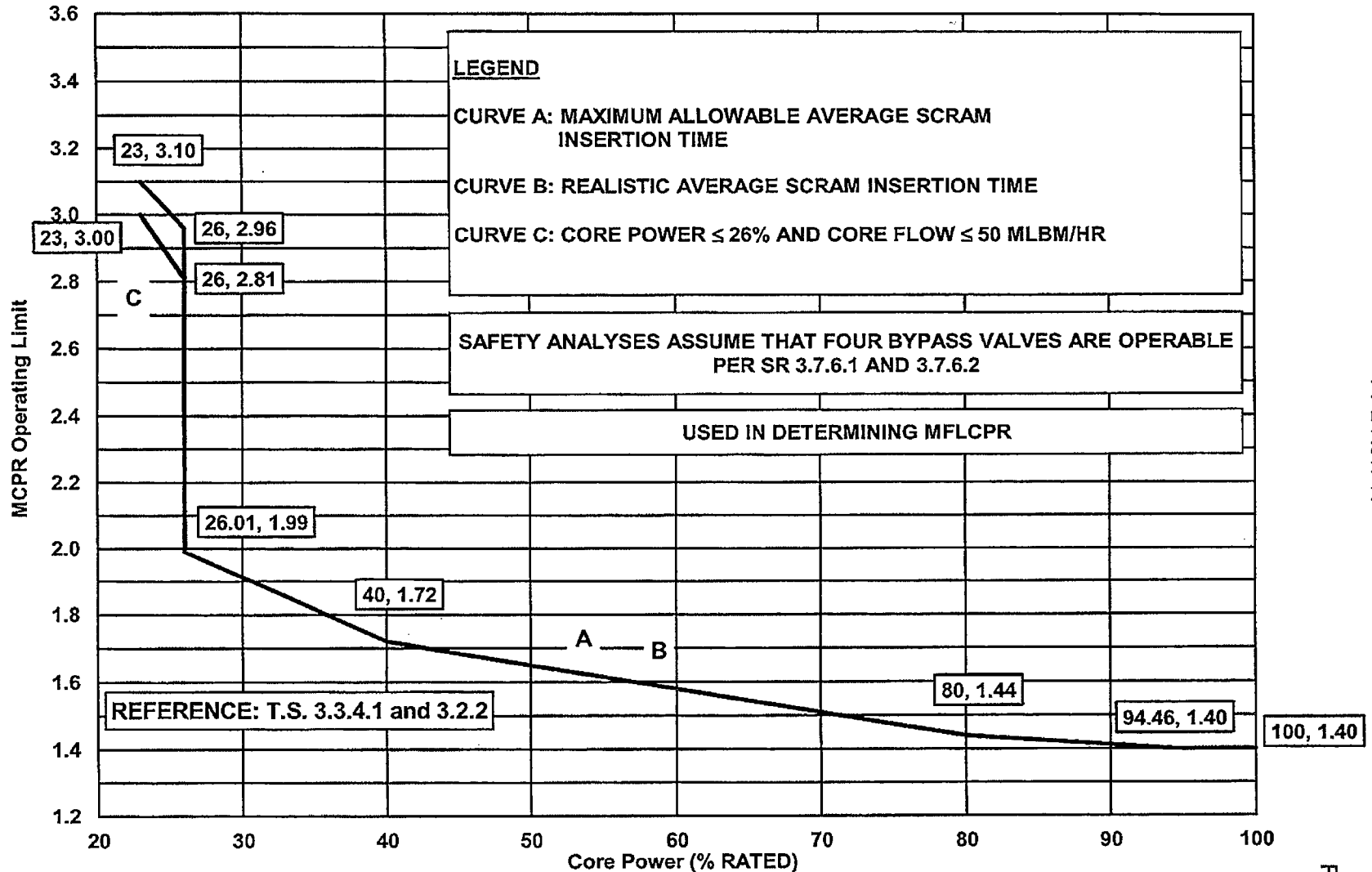
MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
EOC-RPT INOPERABLE
TWO LOOP OPERATION (BOC TO EOC)
FIGURE 5.2-5

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EFFECTIVE DATE 04/21/2015



MCPR OPERATING LIMIT VERSUS CORE POWER
EOC-RPT INOPERABLE
TWO LOOP OPERATION (BOC to EOC)
FIGURE 5.2-6

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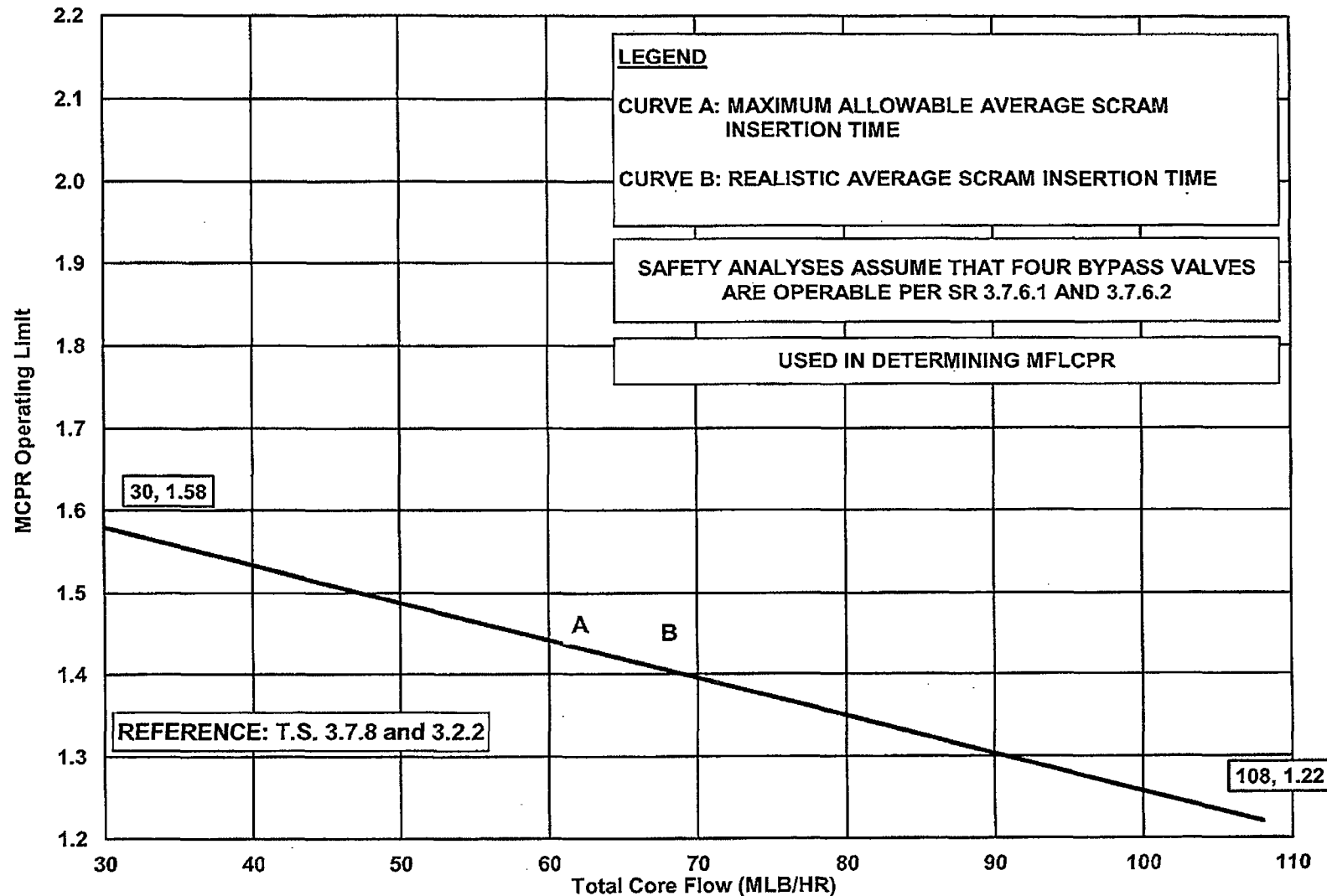
Backup Pressure Regulator Inoperable

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SUSQUEHANNA UNIT 1

TRM/3.2-22

EFFECTIVE DATE 04/21/2015

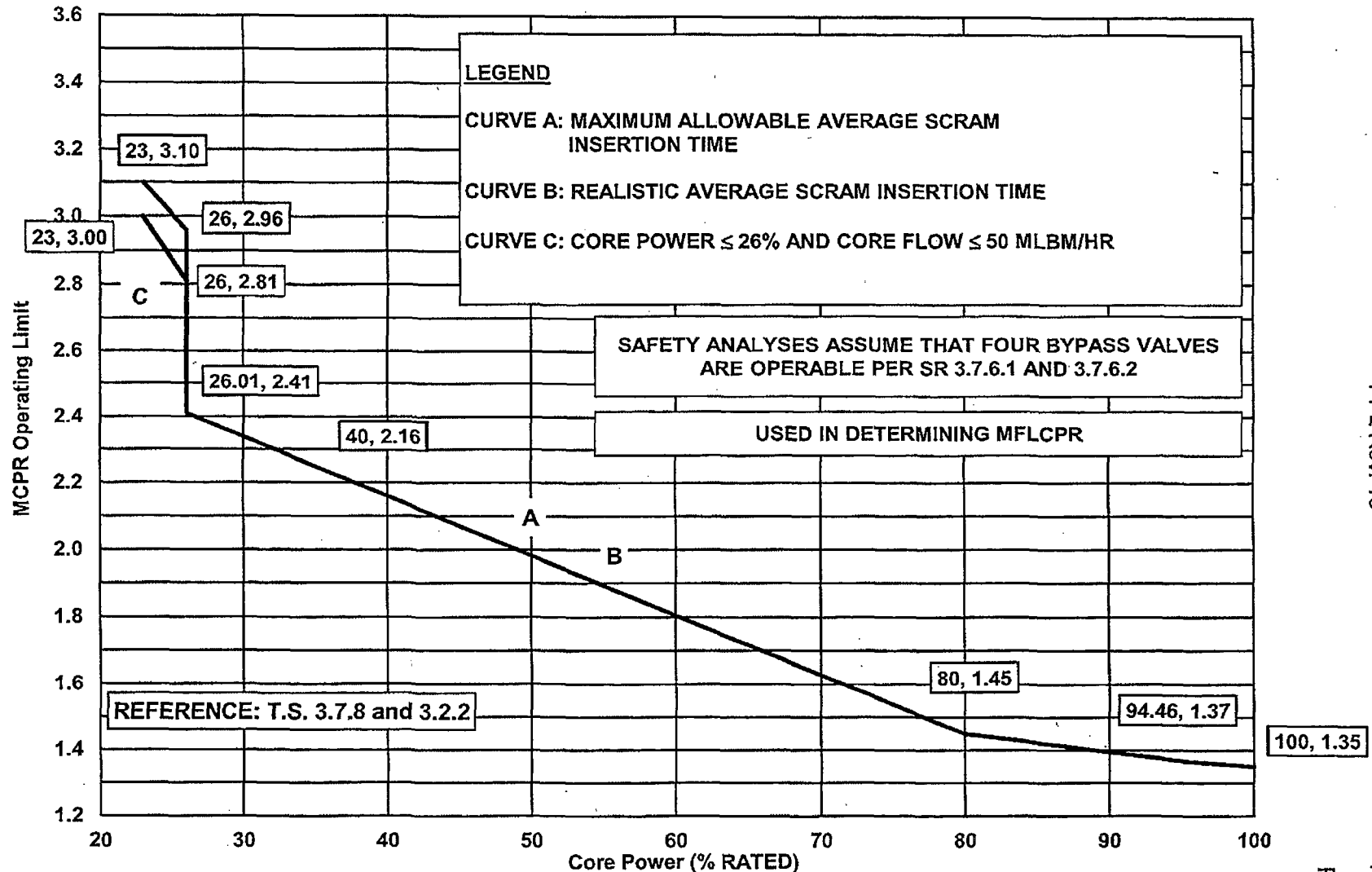


MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
BACKUP PRESSURE REGULATOR INOPERABLE
TWO LOOP OPERATION (BOC TO EOC)
FIGURE 5.2-7

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MCPR OPERATING LIMIT VERSUS CORE POWER
BACKUP PRESSURE REGULATOR INOPERABLE
TWO LOOP OPERATION (BOC to EOC)
FIGURE 5.2-8

Table 5.3-1**Average Scram Time Fraction Table For Use With Scram Time Dependent
MCPR Operating Limits**

Control Rod Position	Average Scram Time to Position (seconds)		
45	0.470		0.520
39	0.630		0.860
25	1.500		1.910
5	2.700		3.440
Average Scram Insertion Time	Realistic		Maximum Allowable

6.0 LINEAR HEAT GENERATION RATE (LHGR)

6.1 Technical Specification Reference

Technical Specification 3.2.3, 3.3.4.1, 3.7.6, and 3.7.8

6.2 Description

The maximum LHGR for ATRIUM™-10 fuel shall not exceed the LHGR limit determined from Figure 6.2-1. The LHGR limit in Figure 6.2-1 is valid for Main Turbine Bypass Operable and Inoperable, EOC-RPT Operable and Inoperable, and Backup Pressure Regulator Operable and Inoperable.

To protect against both fuel centerline melting and cladding strain during anticipated system transients initiated from reduced power and flow conditions, power and flow dependent LHGR limit multipliers are provided. The following figures are applicable to EOC-RPT Operable and Inoperable and Backup Pressure Regulator Operable and Inoperable:

a) Main Turbine Bypass Operable

Figure 6.2-2: Flow-Dependent LHGR Limit Multiplier

Figure 6.2-3: Power-Dependent LHGR Limit Multiplier

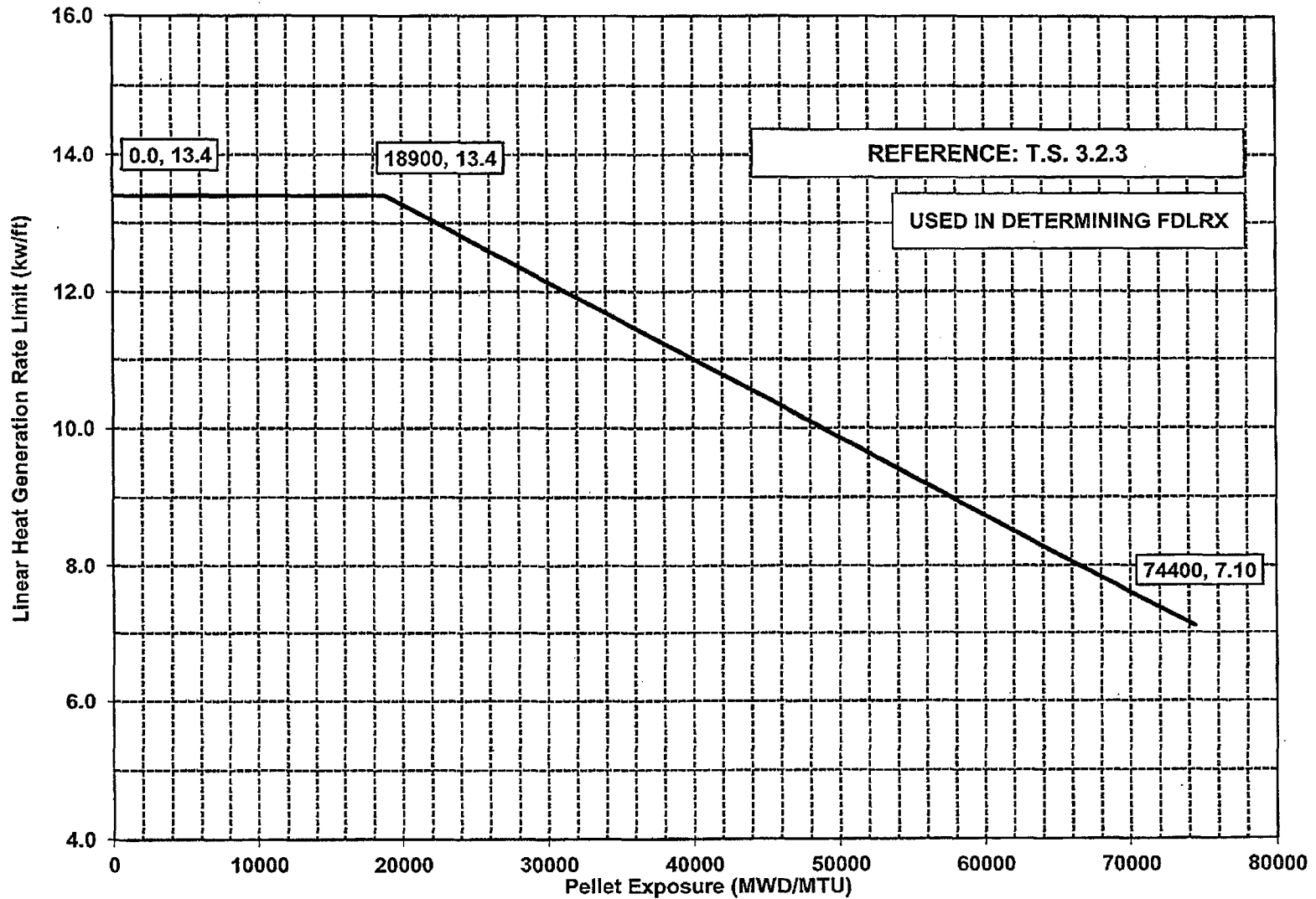
b) Main Turbine Bypass Inoperable

Figure 6.2-4: Flow-Dependent LHGR Limit Multiplier

Figure 6.2-5: Power-Dependent LHGR Limit Multiplier

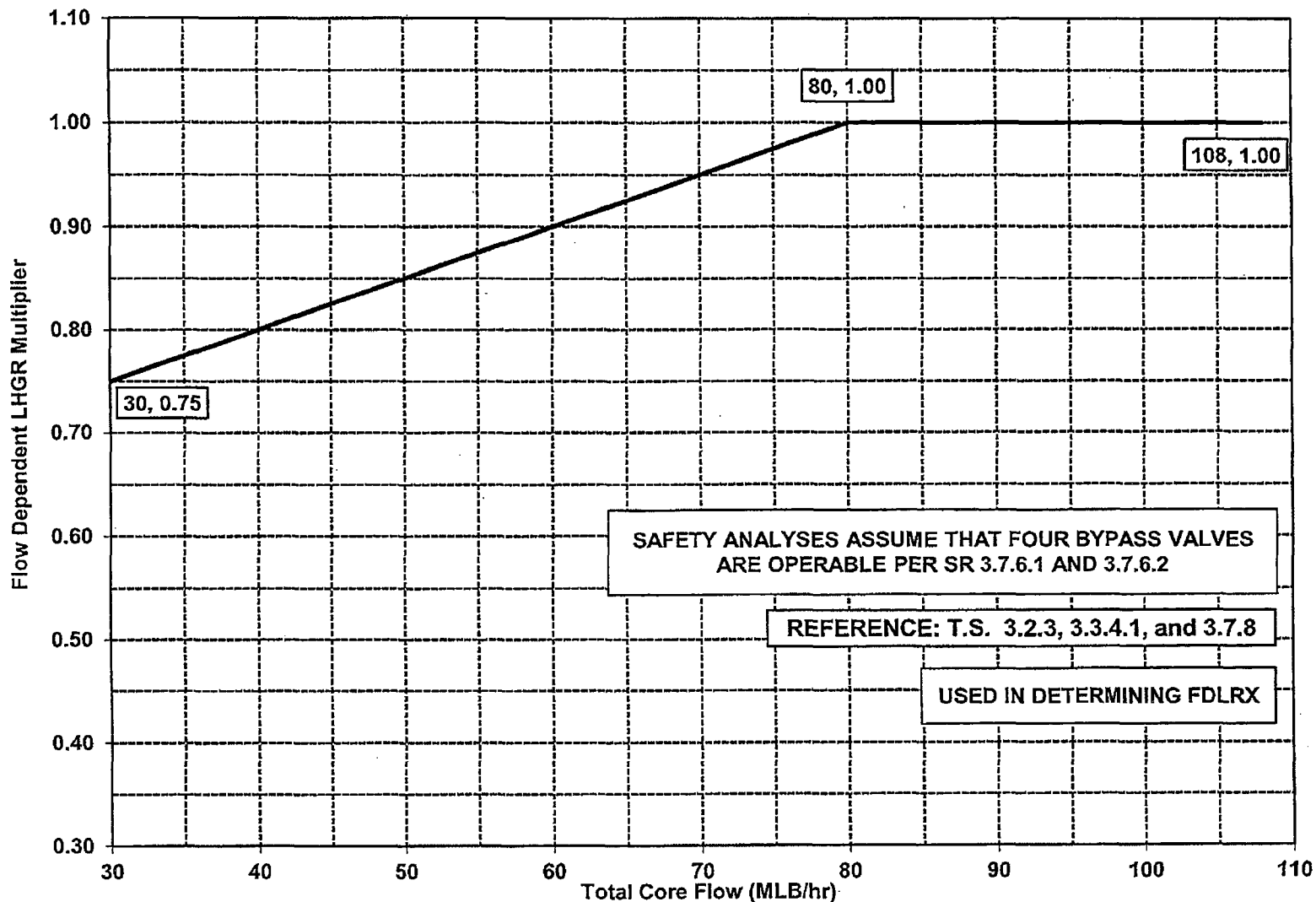
The LGHR limit and LHGR limit multipliers in Figures 6.2-1 through 6.2-5 are valid for both Two Loop and Single Loop operation.

SSS UNIT 1 CYCLE 19



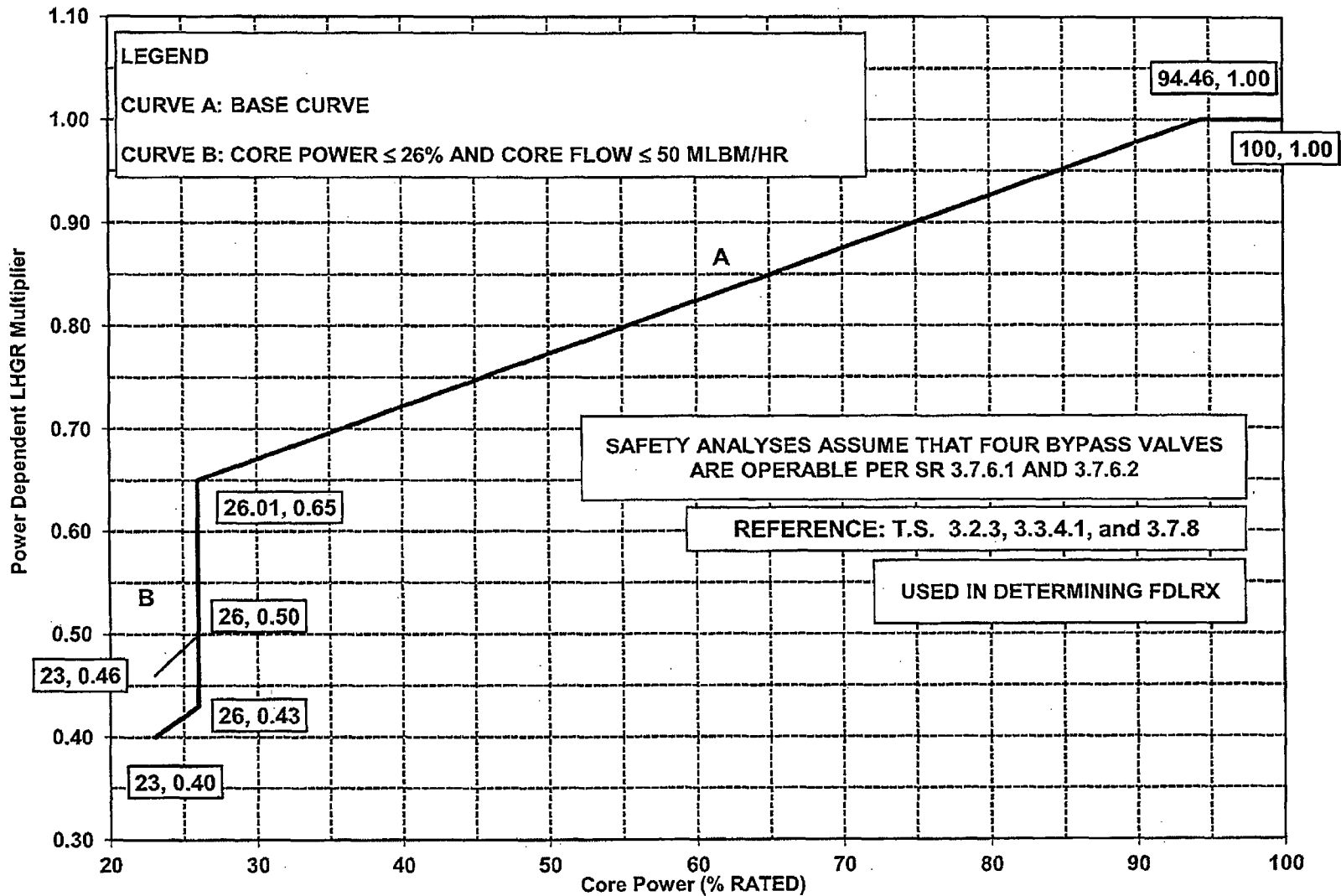
LINEAR HEAT GENERATION RATE LIMIT VERSUS PELLETS EXPOSURE
 ATRIUM™-10 FUEL
 FIGURE 6.2-1

Main Turbine Bypass Operable



FLOW DEPENDENT LHGR LIMIT MULTIPLIER
MAIN TURBINE BYPASS OPERABLE
ATRIUM™-10 FUEL
FIGURE 6.2-2

SSS UNIT 1 CYCLE 19

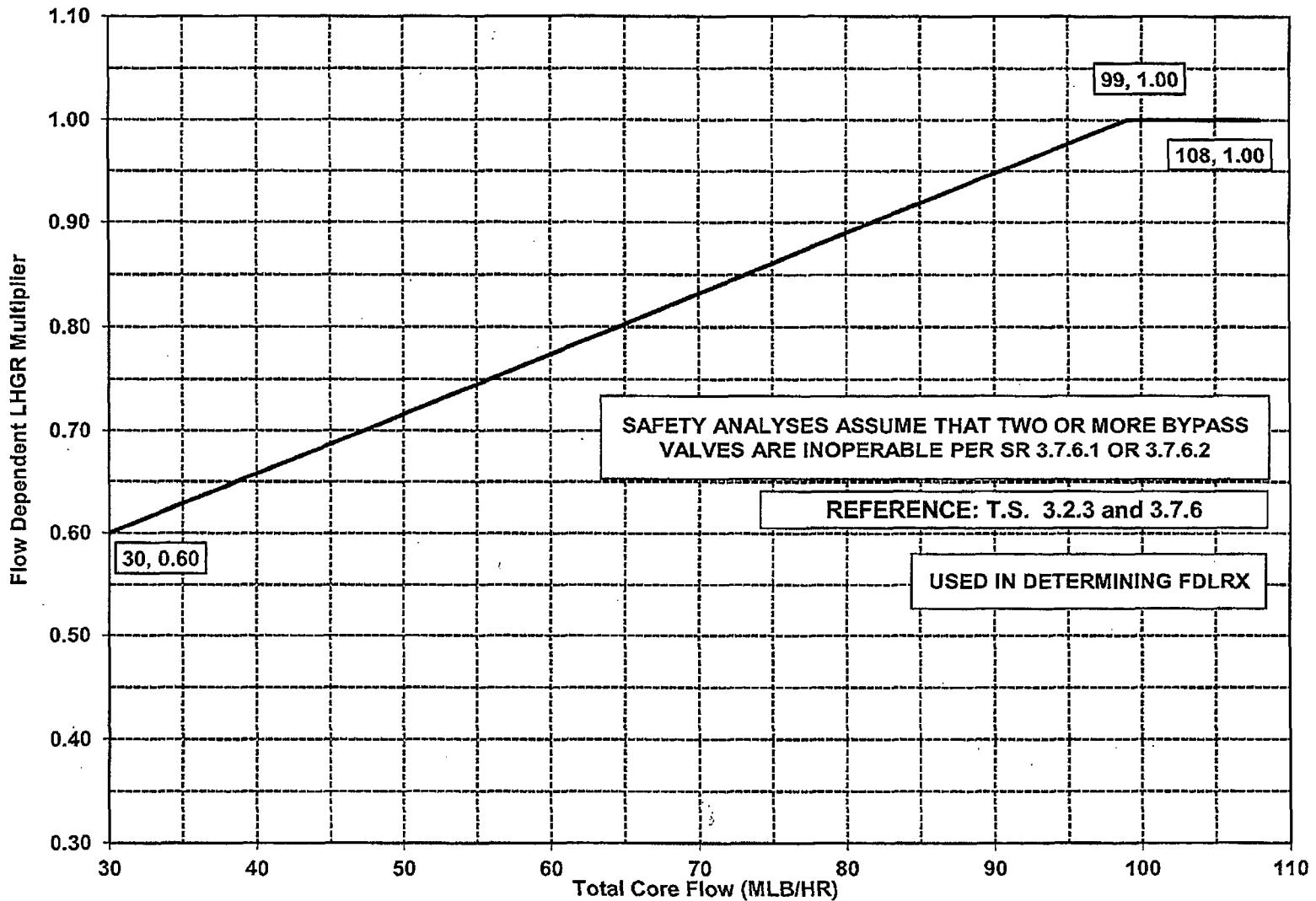


POWER DEPENDENT LHGR LIMIT MULTIPLIER
 MAIN TURBINE BYPASS OPERABLE
 ATRIUM™-10 FUEL
 FIGURE 6.2-3

Main Turbine Bypass Inoperable

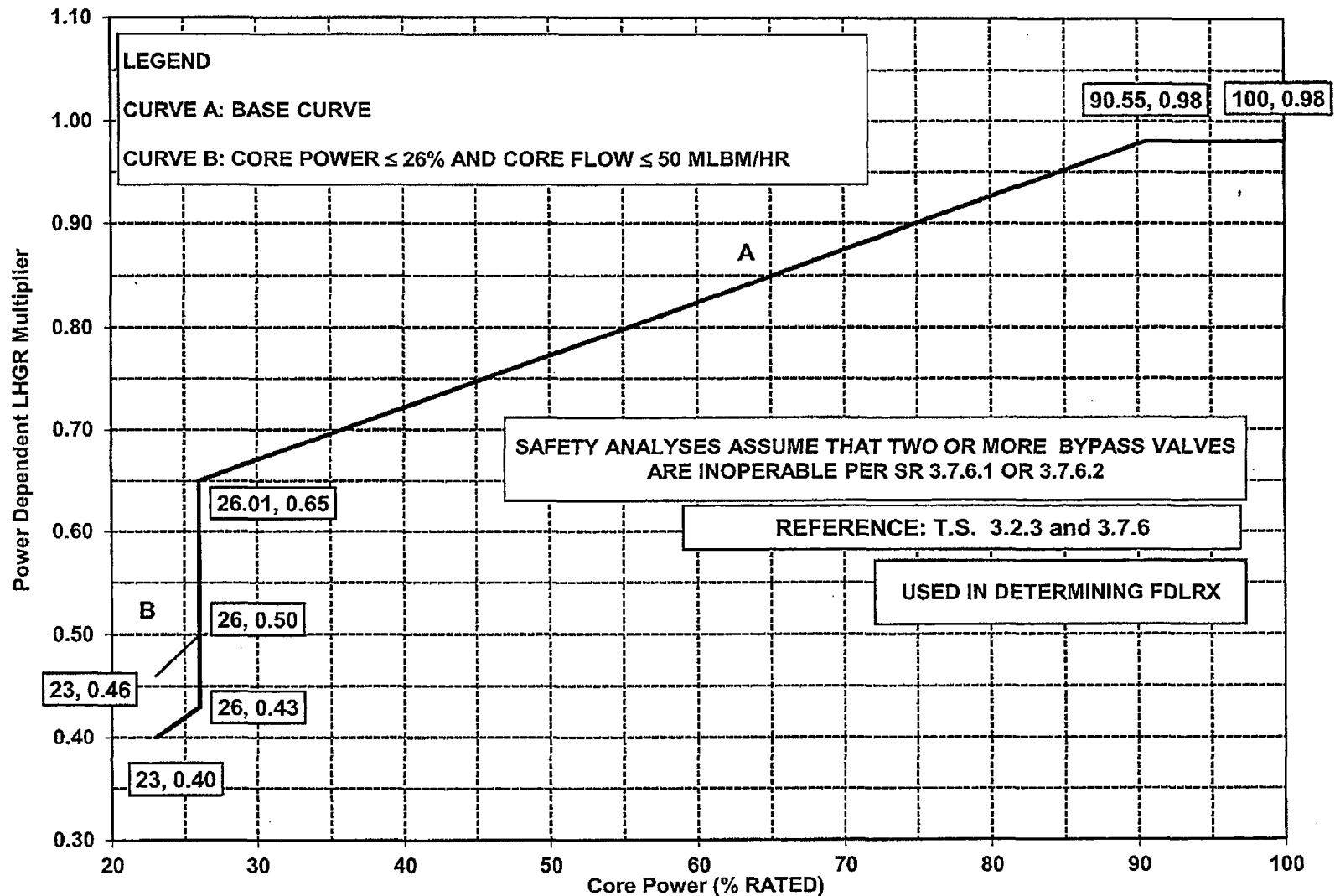
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TRM/3.2-31
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FLOW DEPENDENT LHGR LIMIT MULTIPLIER
MAIN TURBINE BYPASS INOPERABLE
ATRIUM™-10 FUEL
FIGURE 6.2-4

SSS UNIT 1 CYCLE 19



POWER DEPENDENT LHGR LIMIT MULTIPLIER
MAIN TURBINE BYPASS INOPERABLE
ATRIUM™-10 FUEL
FIGURE 6.2-5

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7.0 ROD BLOCK MONITOR (RBM) SETPOINTS AND OPERABILITY REQUIREMENTS

7.1 Technical Specification Reference

Technical Specification 3.3.2.1

7.2 Description

The RBM Allowable Value and Trip Setpoints for;

- a) Low Power Range Setpoint,
- b) Intermediate Power Range Setpoint,
- c) High Power Range Setpoint,
- d) Low Power Range - Upscale,
- e) Intermediate Power Range - Upscale, and
- f) High Power Range - Upscale

shall be established as specified in Table 7.2-1. The RBM setpoints are valid for Two Loop and Single Loop Operation, Main Turbine Bypass Operable and Inoperable, EOC-RPT Operable and Inoperable, and Backup Pressure Regulator Operable and Inoperable.

The RBM system design objective is to block erroneous control rod withdrawal initiated by the operator before fuel design limits are violated. If the full withdrawal of any control rod would not violate a fuel design limit, then the RBM system is not required to be operable. Table 7.2-2 provides RBM system operability requirements to ensure that fuel design limits are not violated.

Table 7.2-1
RBM Setpoints

Function	Allowable Value ⁽¹⁾	Nominal Trip Setpoint
Low Power Range Setpoint	28.0	24.9
Intermediate Power Range Setpoint	63.0	61.0
High Power Range Setpoint	83.0	81.0
Low Power Range - Upscale	123.4	123.0
Intermediate Power Range - Upscale	117.4	117.0
High Power Range - Upscale	107.6	107.2

- (1) Power setpoint function (Low, Intermediate, and High Power Range Setpoints) determined in percent of RATED THERMAL POWER. Upscale trip setpoint function (Low, Intermediate, and High Power Range - Upscale) determined in percent of reference level.

Table 7.2-2
RBM System Operability Requirements

Thermal Power (% of Rated)	MCPR ^(2,3)
≥ 28 and < 90	< 1.76
≥ 90 and < 95	< 1.47
≥ 95	< 1.70

- (2) Applicable to Main Turbine Bypass Operable and Inoperable, EOC-RPT Operable and Inoperable, and Backup Pressure Regulator Operable and Inoperable.
- (3) Applicable to both Two Loop and Single Loop Operation.

8.0 RECIRCULATION LOOPS - SINGLE LOOP OPERATION

8.1 Technical Specification Reference

Technical Specification 3.2.1, 3.2.2, 3.2.3, 3.3.4.1, 3.4.1, 3.7.6, and 3.7.8

8.2 Description

APLHGR

The APLHGR limit for ATRIUM™-10 fuel shall be equal to the APLHGR Limit from Figure 8.2-1.

The APLHGR limits in Figure 8.2-1 are valid for Main Turbine Bypass Operable and Inoperable, EOC-RPT Operable and Inoperable, and Backup Pressure Regulator Operable and Inoperable in Single Loop operation.

Minimum Critical Power Ratio Limit

The MCPR limit is specified as a function of core power, core flow, and plant equipment operability status. The MCPR limits for all fuel types (ATRIUM™-10) shall be the greater of the Flow-Dependent or the Power-Dependent MCPR, depending on the applicable equipment operability status.

a) Main Turbine Bypass / EOC-RPT / Backup Pressure Regulator Operable

Figure 8.2-2: Flow-Dependent MCPR value determined from BOC to EOC

Figure 8.2-3: Power-Dependent MCPR value determined from BOC to EOC

b) Main Turbine Bypass Inoperable

Figure 8.2-4: Flow-Dependent MCPR value determined from BOC to EOC

Figure 8.2-5: Power-Dependent MCPR value determined from BOC to EOC

c) EOC-RPT Inoperable

Figure 8.2-6: Flow-Dependent MCPR value determined from BOC to EOC

Figure 8.2-7: Power-Dependent MCPR value determined from BOC to EOC

d) Backup Pressure Regulator Inoperable

Figure 8.2-8: Flow-Dependent MCPR value determined from BOC to EOC

Figure 8.2-9: Power-Dependent MCPR value determined from BOC to EOC

The MCPR limits in Figures 8.2-2 through 8.2-9 are valid only for Single Loop operation.

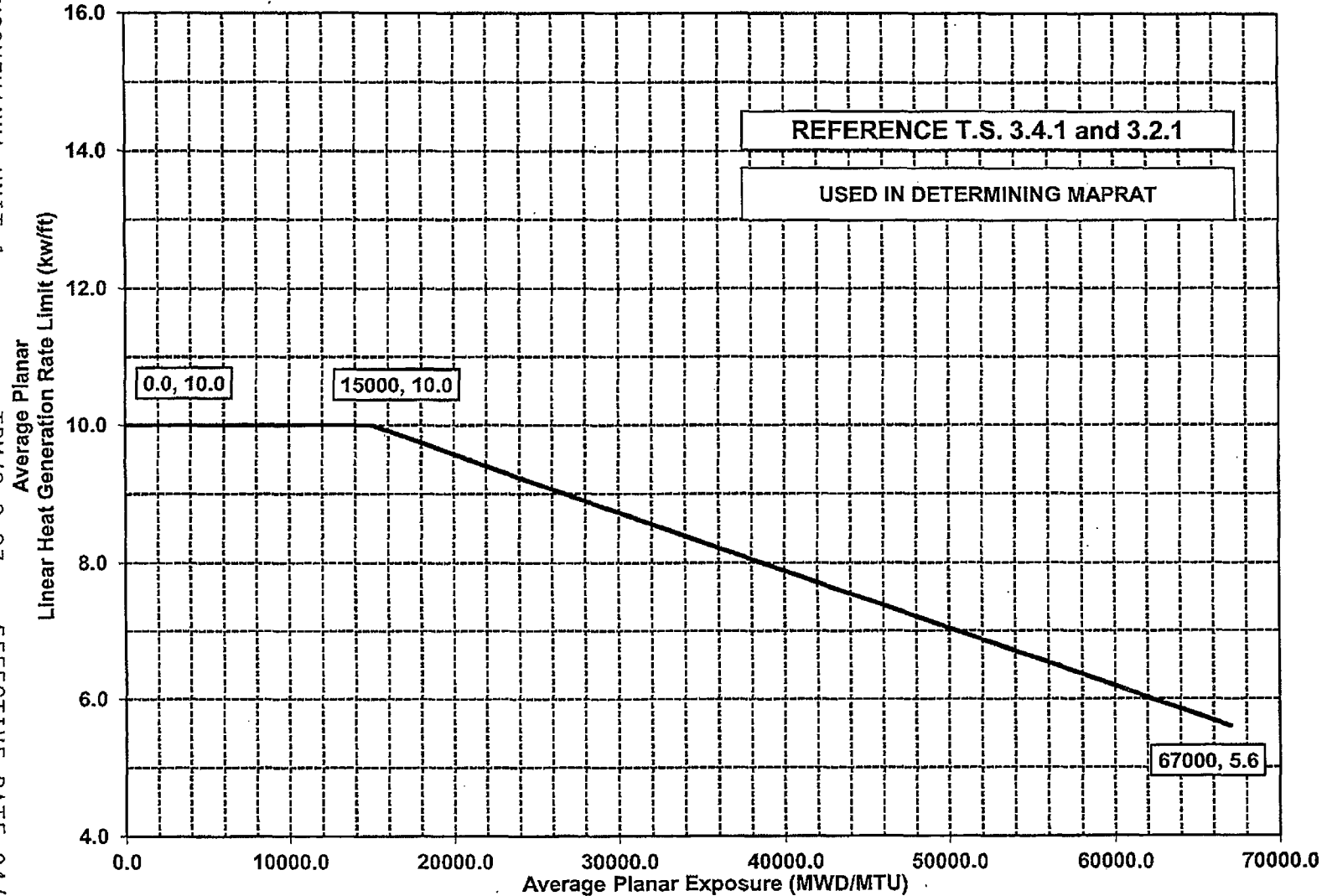
Linear Heat Generation Rate Limit

The LHGR limits for Single Loop Operation are defined in Section 6.0.

RBM Setpoints and Operability Requirements

The RBM setpoints and operability requirements for Single Loop Operation are defined in Section 7.0.

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AVERAGE PLANAR LINEAR HEAT GENERATION RATE LIMIT VERSUS
AVERAGE PLANAR EXPOSURE - SINGLE LOOP OPERATION
ATRIUM™-10 FUEL
FIGURE 8.2-1

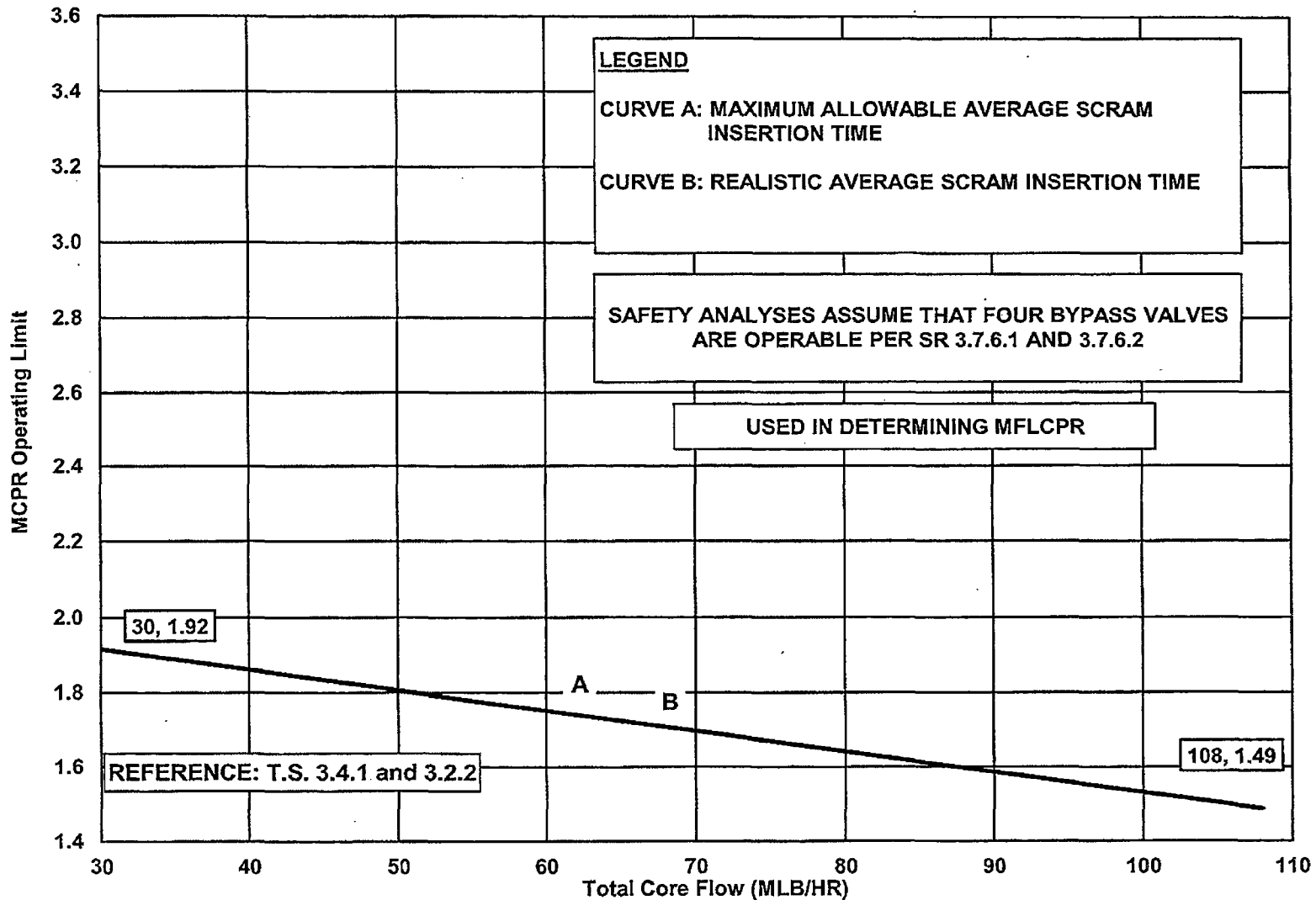
Main Turbine Bypass / EOC-RPT / Backup Pressure Regulator Operable

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SUSQUEHANNA UNIT 1

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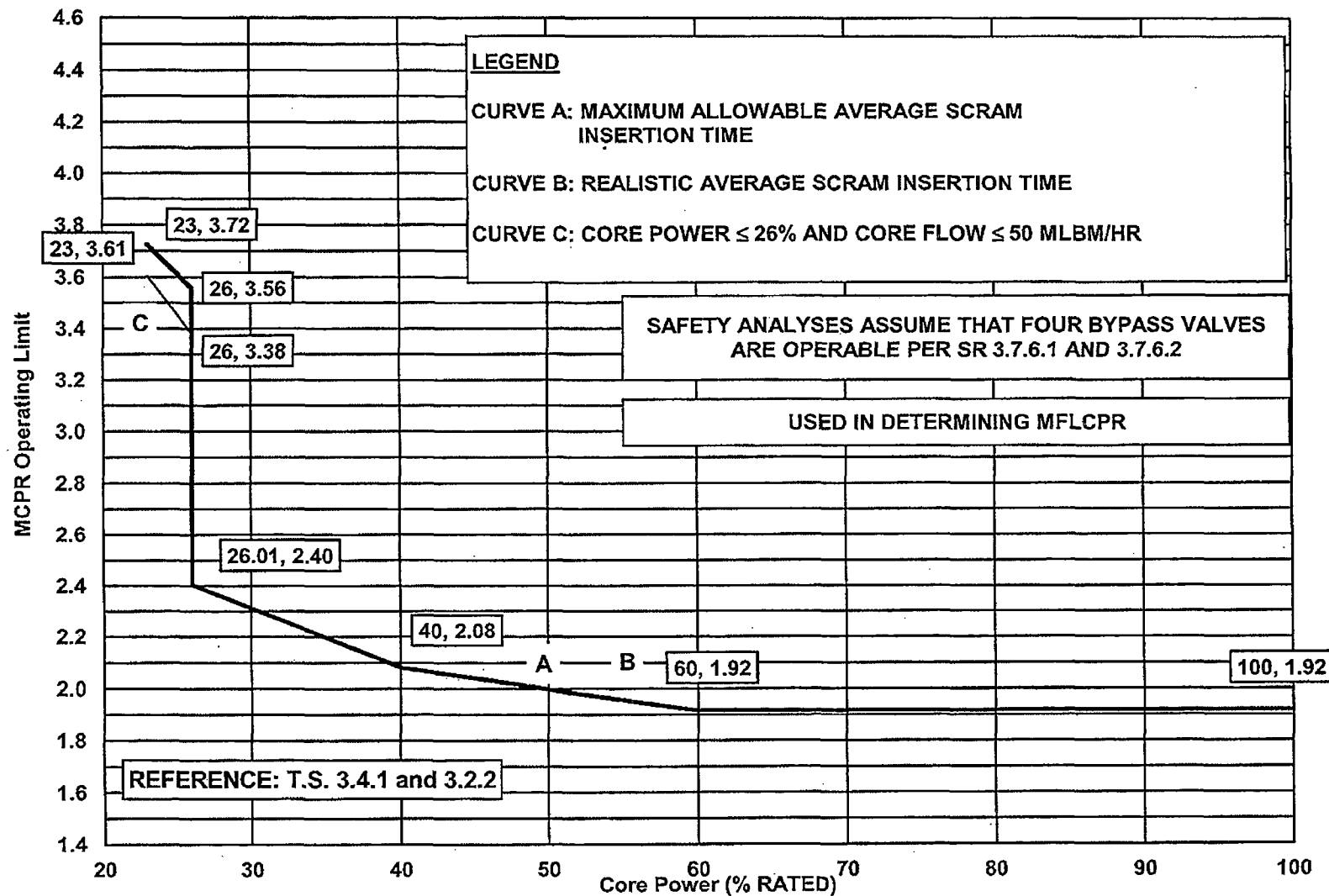


MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
MAIN TURBINE BYPASS / EOC-RPT / BACKUP PRESSURE REGULATOR OPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-2

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MCPR OPERATING LIMIT VERSUS CORE POWER
MAIN TURBINE BYPASS / EOC-RPT / BACKUP PRESSURE REGULATOR OPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-3

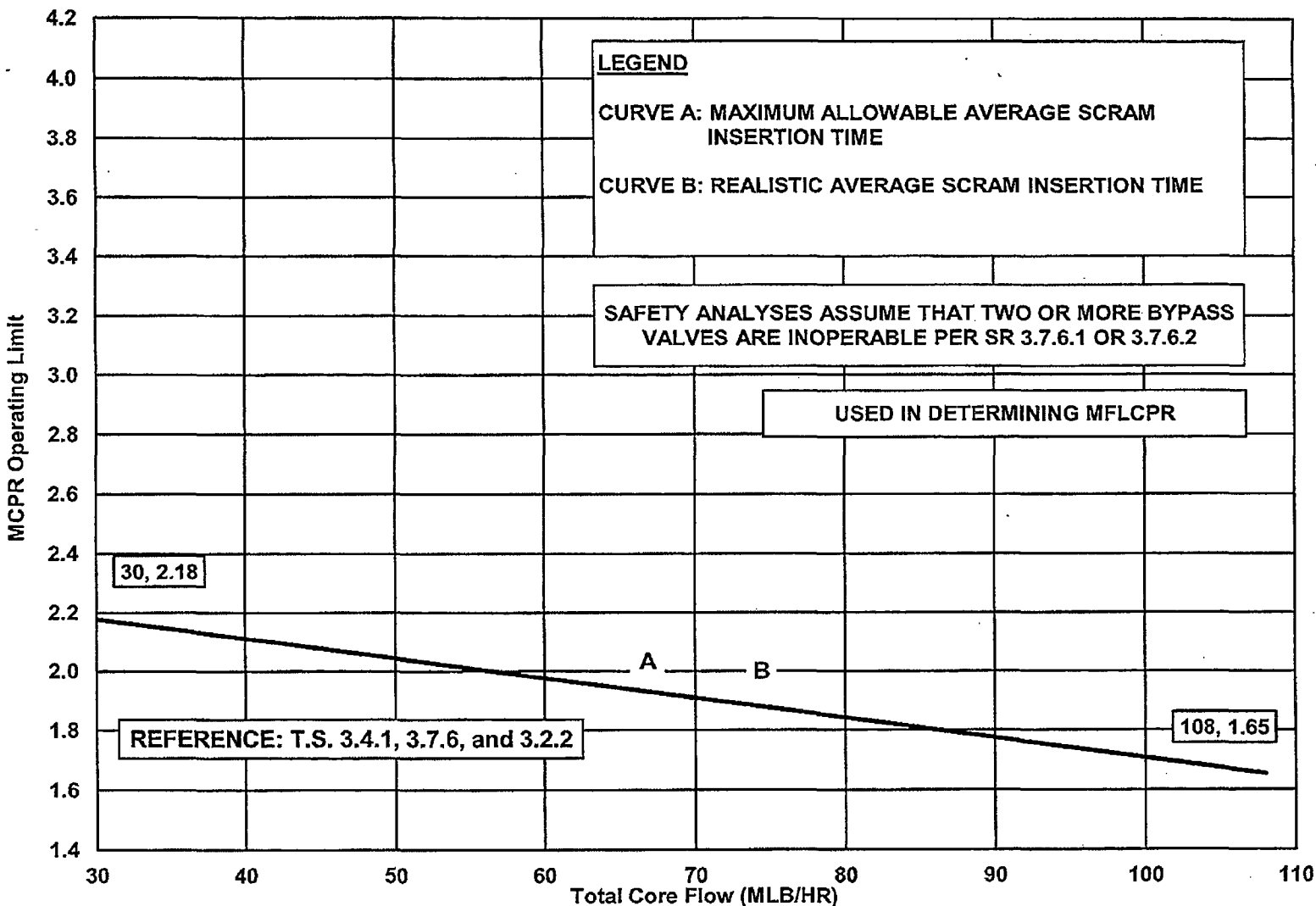
Main Turbine Bypass Inoperable

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SUSQUEHANNA UNIT 1

TRM/3.2-42

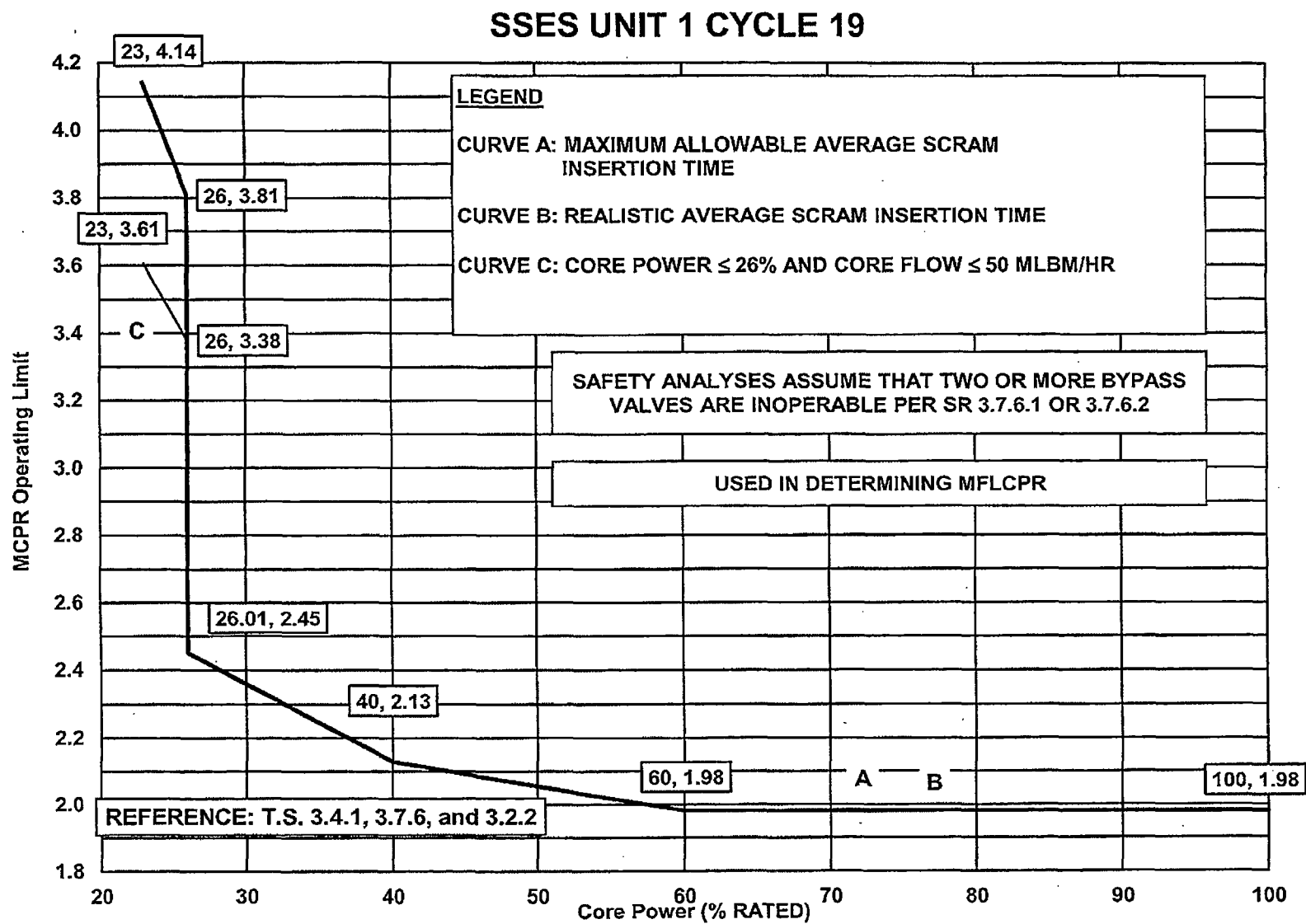
EFFECTIVE DATE 04/21/2015



MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
MAIN TURBINE BYPASS INOPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-4

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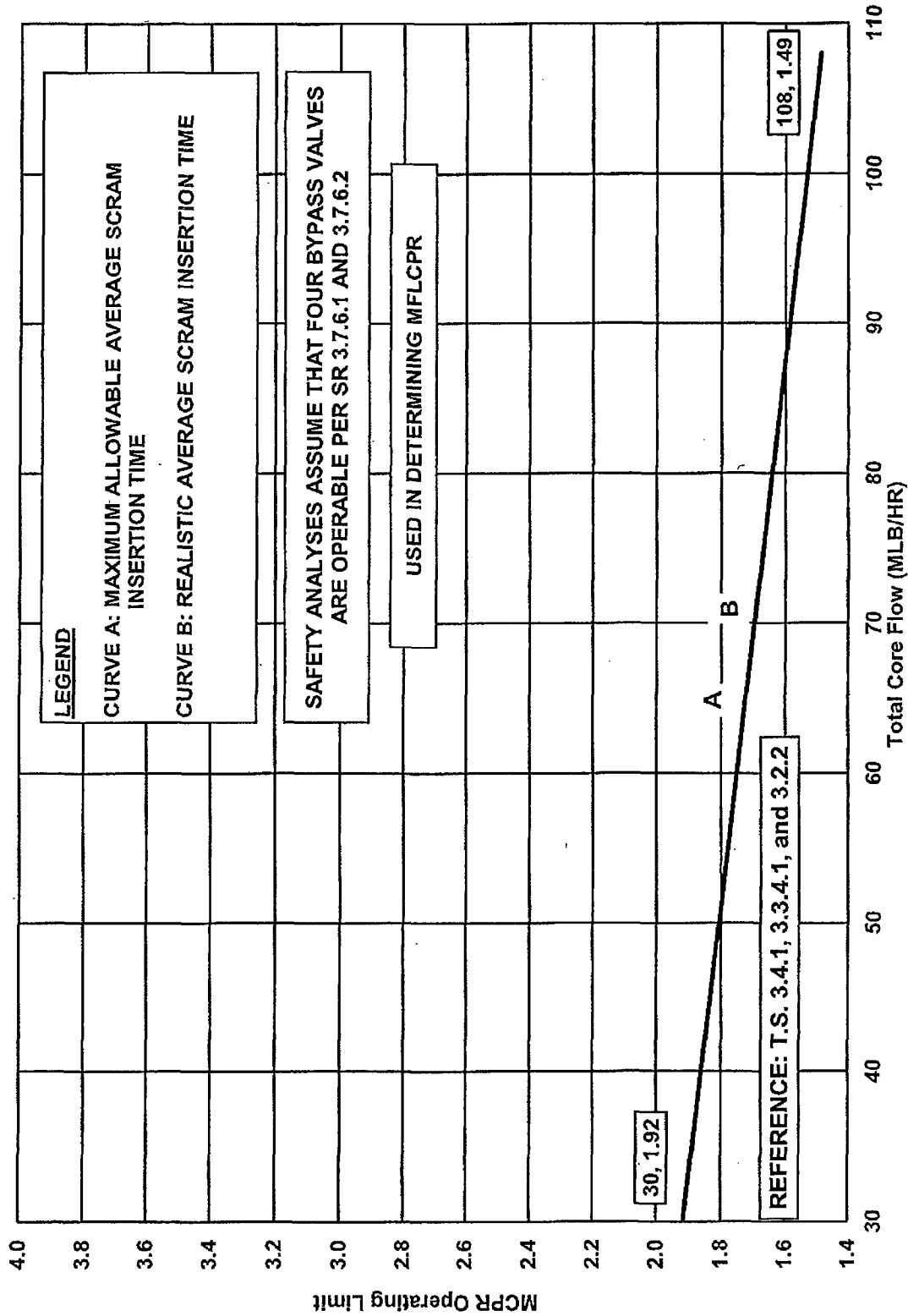
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MCPR OPERATING LIMIT VERSUS CORE POWER
MAIN TURBINE BYPASS INOPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-5

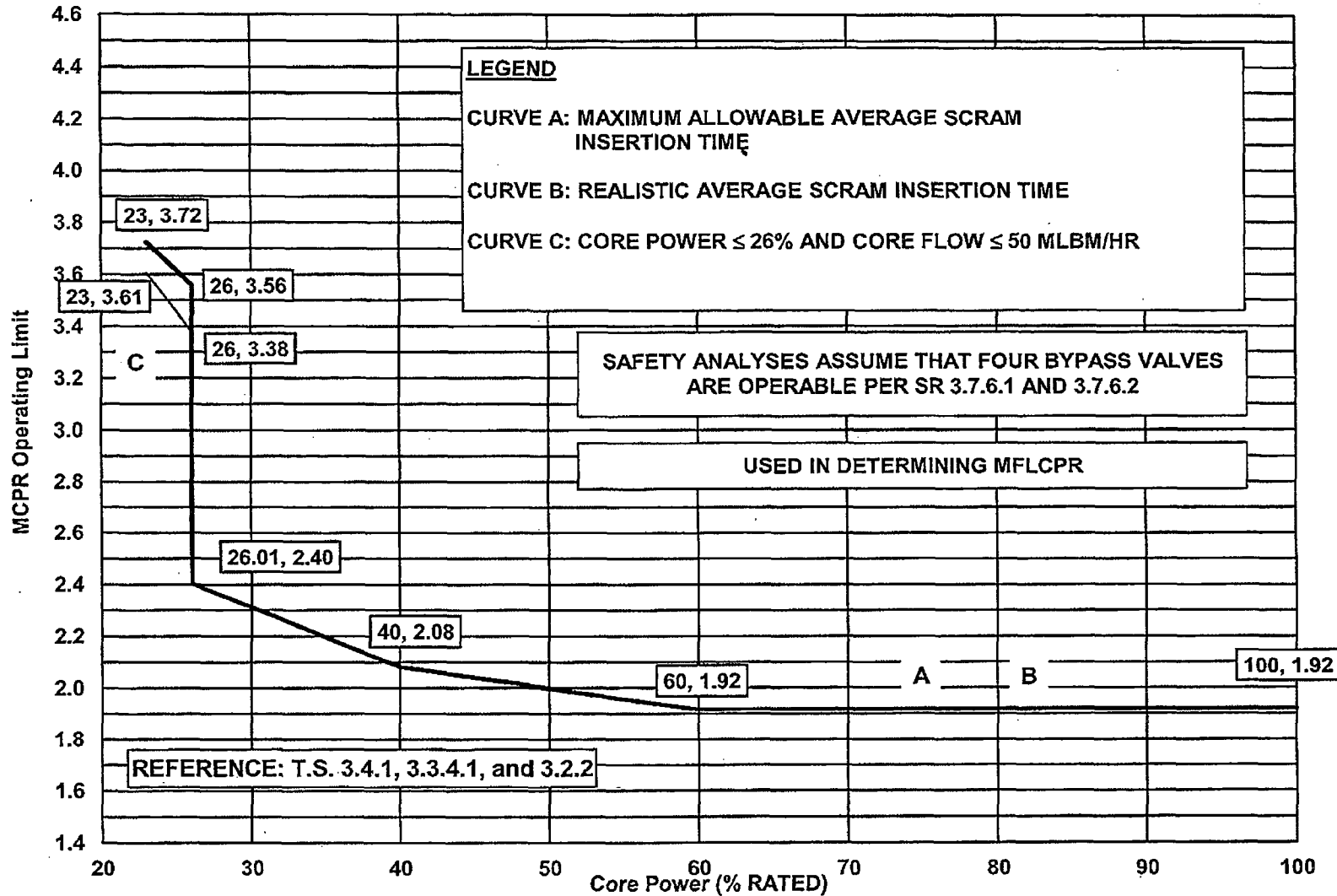
EOC-RPT Inoperable

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MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
EOC-RPT INOPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-6

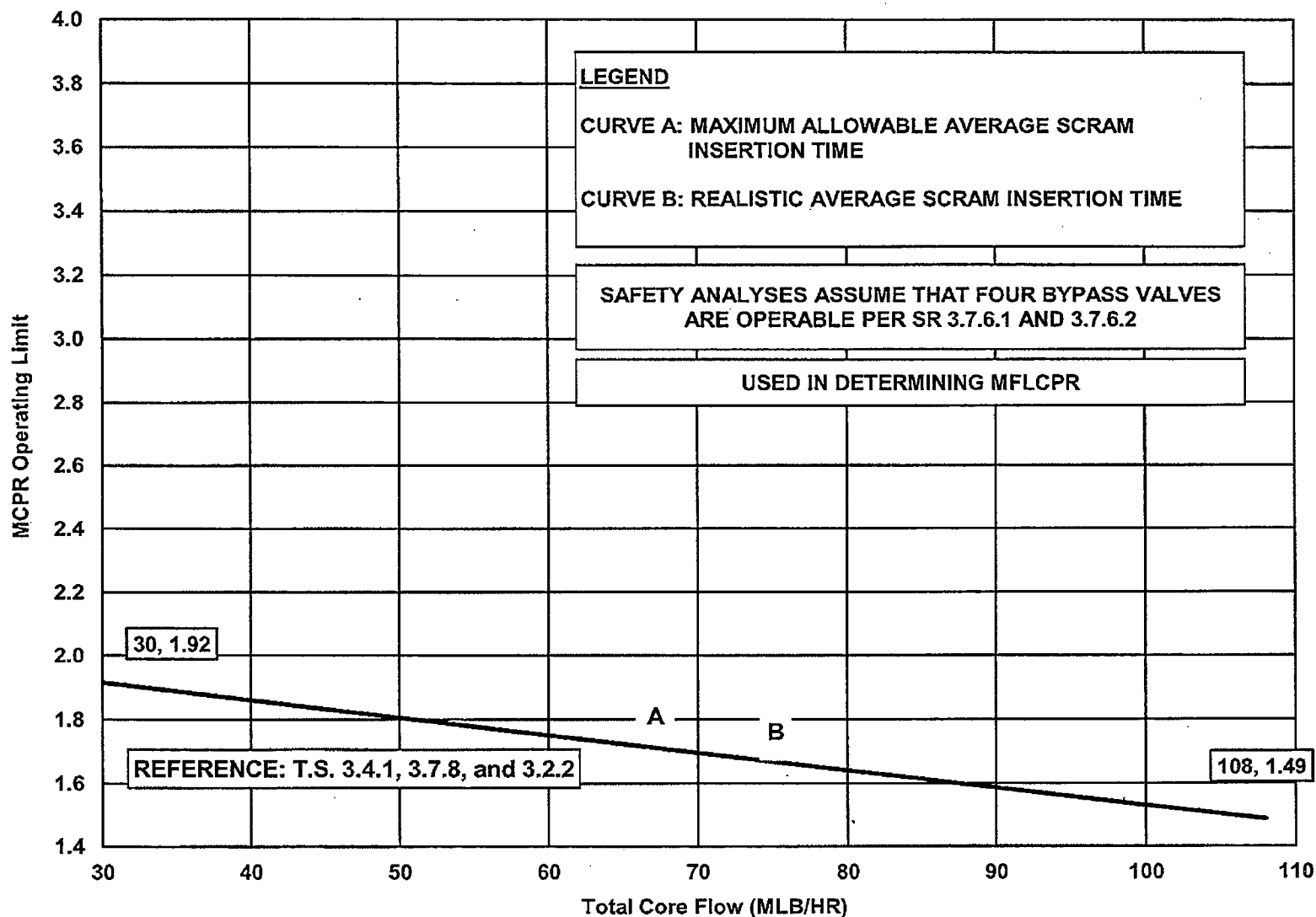
SSSES UNIT 1 CYCLE 19



MCPR OPERATING LIMIT VERSUS CORE POWER
EOC-RPT INOPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-7

Backup Pressure Regulator Inoperable

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MCPR OPERATING LIMIT VERSUS TOTAL CORE FLOW
BACKUP PRESSURE REGULATOR INOPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-8

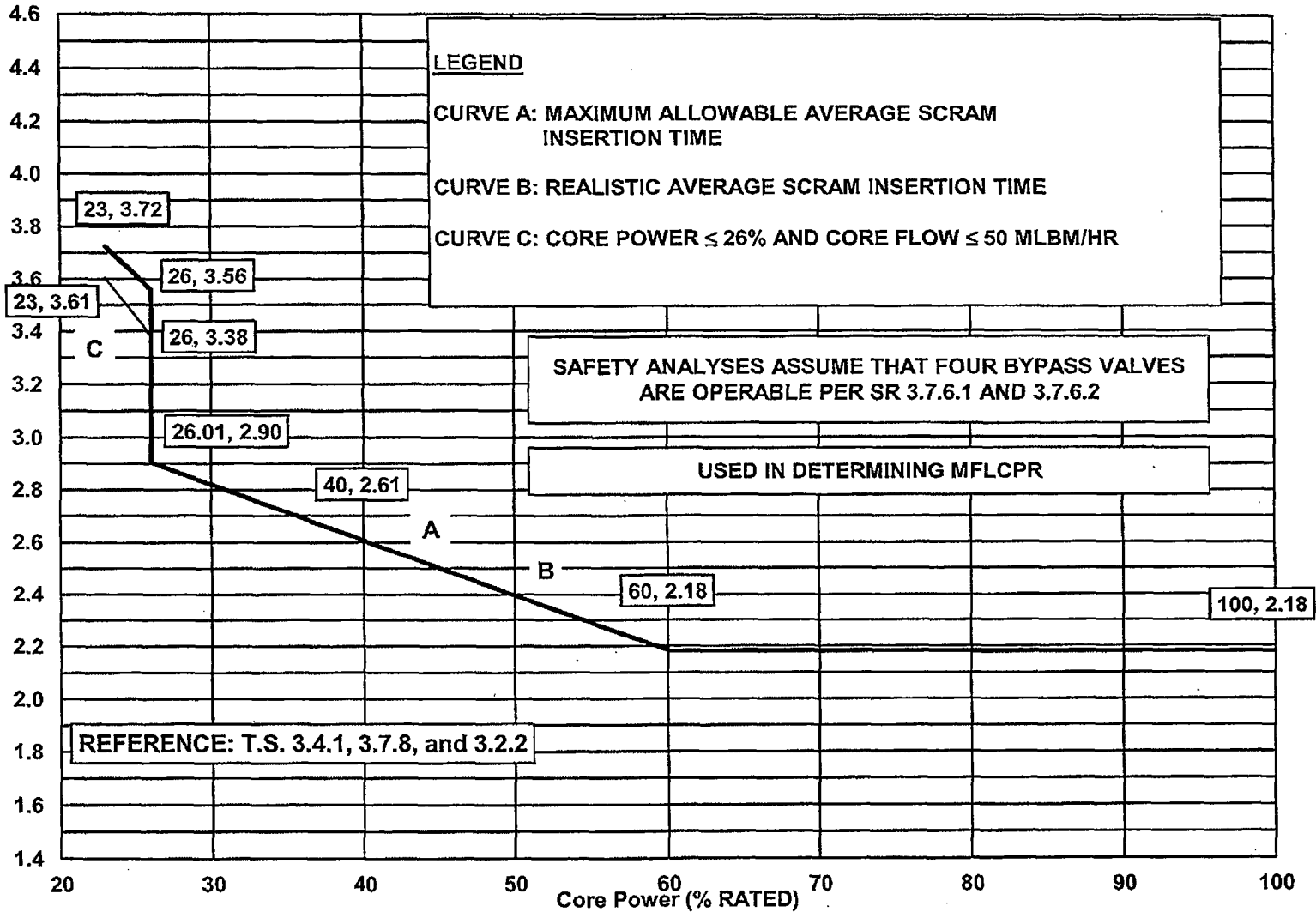
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SUSQUEHANNA UNIT 1

TRM/3.2-49

EFFECTIVE DATE 04/21/2015

MCPR Operating Limit



MCPR OPERATING LIMIT VERSUS CORE POWER
BACKUP PRESSURE REGULATOR INOPERABLE
SINGLE LOOP OPERATION (BOC to EOC)
FIGURE 8.2-9

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9.0 POWER / FLOW MAP

9.1 Technical Specification Reference

Technical Specification 3.3.1.1

9.2 Description

Monitor reactor conditions to maintain THERMAL POWER / core flow outside of Stability Regions I and II of the Power / Flow map, Figure 9.1.

If the OPRM Instrumentation is OPERABLE per TS 3.3.1.1, Region I of the Power / Flow map is considered an immediate exit region.

If the OPRM Instrumentation is inoperable per TS 3.3.1.1, Region I of the Power / Flow map is considered an immediate scram region.

Region II of the Power / Flow map is considered an immediate exit region regardless of the operability of the OPRM Instrumentation.

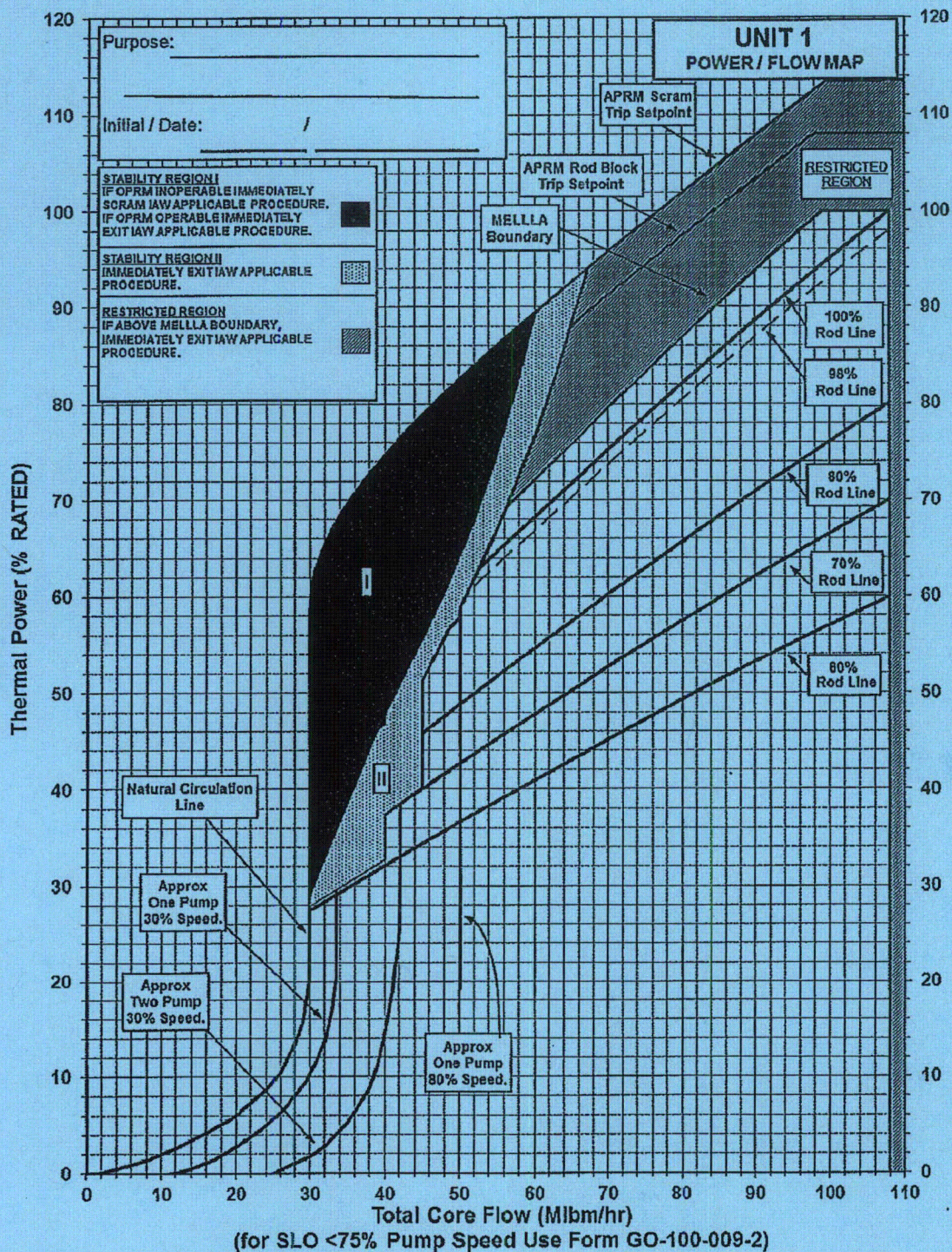


Figure 9.1
SSES Unit 1 Cycle 19 Power / Flow Map

10.0 OPRM SETPOINTS

10.1 Technical Specification Reference

Technical Specification 3.3.1.1

10.2 Description

Setpoints for the OPRM Instrumentation are established that will reliably detect and suppress anticipated stability related power oscillations while providing a high degree of confidence that the MCPR Safety limit is not violated. The setpoints are described in Section 2.0 and are listed below:

$$S_p = 1.11$$

$$N_p = 15$$

$$F_p = 60 \text{ Mlbm / hr}$$

11.0 REFERENCES

- 11.1 The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
1. XN-NF-81-58(P)(A), Revision 2 and Supplements 1 and 2, "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model," Exxon Nuclear Company, March 1984.
 2. EMF-2361(P)(A), Revision 0, "EXEM BWR-2000 ECCS Evaluation Model," Framatome ANP, May 2001.
 3. EMF-2292(P)(A), Revision 0, "ATRIUM™-10: Appendix K Spray Heat Transfer Coefficients," Siemens Power Corporation, September 2000.
 4. XN-NF-84-105(P)(A), Volume 1 and Volume 1 Supplements 1 and 2, "XCOBRA-T: A Computer Code for BWR Transient Thermal-Hydraulic Core Analysis," Exxon Nuclear Company, February 1987.
 5. XN-NF-80-19(P)(A), Volume 1 and Supplements 1 and 2, "Exxon Nuclear Methodology for Boiling Water Reactors: Neutronic Methods for Design and Analysis," Exxon Nuclear Company, March 1983.
 6. XN-NF-80-19(P)(A), Volumes 2, 2A, 2B, and 2C "Exxon Nuclear Methodology for Boiling Water Reactors: EXEM BWR ECCS Evaluation Model," Exxon Nuclear Company, September 1982.
 7. XN-NF-80-19(P)(A), Volume 3 Revision 2 "Exxon Nuclear Methodology for Boiling Water Reactors Thermex: Thermal Limits Methodology Summary Description," Exxon Nuclear Company, January 1987.
 8. XN-NF-80-19(P)(A), Volume 4, Revision 1, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads," Exxon Nuclear Company, June 1986.
 9. XN-NF-85-67(P)(A), Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel," Exxon Nuclear Company, Inc., September 1986.
 10. ANF-524(P)(A), Revision 2 and Supplements 1 and 2, "Advanced Nuclear Fuels Corporation Critical Power Methodology for Boiling Water Reactors," November 1990.
 11. NE-092-001A, Revision 1, "Licensing Topical Report for Power Uprate With Increased Core Flow," Pennsylvania Power & Light Company, December 1992 and NRC SER (November 30, 1993).

12. ANF-89-98(P)(A) Revision 1 and Supplement 1, "Generic Mechanical Design Criteria for BWR Fuel Designs," Advanced Nuclear Fuels Corporation, May 1995.
13. EMF-2209(P)(A), Revision 3, "SPCB Critical Power Correlation," AREVA NP, September 2009.
14. EMF-85-74(P)(A), Revision 0, Supplement 1(P)(A) and Supplement 2(P)(A), "RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model," Siemens Power Corporation, February 1998.
15. EMF-2158(P)(A), Revision 0, "Siemens Power Corporation Methodology for Boiling Water Reactors: Evaluation and Validation of CASMO-4/Microburn-B2," Siemens Power Corporation, October 1999.
16. EMF-CC-074(P)(A), Volume 4, Revision 0, "BWR Stability Analysis - Assessment of STAIF with Input from MICROBURN-B2," Siemens Power Corporation, August 2000.
17. NEDO-32465-A, "BWROG Reactor Core Stability Detect and Suppress Solutions Licensing Basis Methodology for Reload Applications," August 1996.
18. ANF-913(P)(A), Volume 1 Revision 1 and Volume 1 Supplements 2, 3, and 4, "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analyses," Advanced Nuclear Fuels Corporation, August 1990.
19. ANF-1358(P)(A), Revision 3, "The Loss of Feedwater Heating Transient in Boiling Water Reactors," Framatome ANP, September 2005.

3.11 Radioactive Effluents

3.11.2 Gaseous Effluents

3.11.2.6 Radioactive Gaseous Effluent Monitoring Instrumentation

TRO 3.11.2.6 The radioactive gaseous effluent monitoring instrumentation channels shown in Table 3.11.2.6-1 shall be OPERABLE with their setpoints established in accordance with the ODCM to ensure that the limits of Requirement 3.11.2.1 are not exceeded.

APPLICABILITY: According to Table 3.11.2.6-1

ACTIONS

- NOTE -----
1. Separate condition entry is allowed for each channel.
 2. The provisions of TRO 3.0.6 are not applicable.
-

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Radioactive gaseous effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required to ensure that the limits of Requirement 3.11.2.1 are not exceeded	A.1 Suspend the release of radioactive gaseous effluents monitored by the affected channel	Immediately
	<u>OR</u> A.2 Declare the channel inoperable	Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Reactor Building Ventilation System Noble Gas Activity Monitor low range channel inoperable	B.1 Take grab samples	Once per 8 hours while release is in progress.
	<u>AND</u>	
	B.2 Analyze grab samples for isotopic activity to the required LLDs for principal noble gas gamma emitters (Table 3.11.2.1-1)	Within 24 hours of grab sample
	<u>AND</u>	
	B.3 Restore monitoring instrumentation	30 days
C. Deleted		
D. Reactor Building Ventilation Monitoring System Effluent System Flow Rate Monitor or Sampler Flow Rate Monitor inoperable	D.1 Estimate flow rate	Once per 4 hours while release is in progress
	<u>AND</u>	
	D.2 Restore monitoring instrumentation	30 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
E. Turbine Building Ventilation System Noble Gas Activity Monitor low range channel inoperable	E.1 Verify mechanical vacuum pump is not in operation	Immediately
	<u>AND</u>	
	E.2 Take grab samples	Once per 8 hours while release is in progress
	<u>AND</u>	
	E.3 Analyze grab samples for isotopic activity to the required LLDs for the principal noble gas gamma emitters (Table 3.11.2.1-1)	Within 24 hours after sample
	<u>AND</u>	
	E.4 Restore monitoring instrumentation	30 days
F. Deleted		
G. Turbine Building Ventilation Monitoring System Effluent System Flow Rate Monitor or Sampler Flow Rate Monitor inoperable	G.1 Estimate flow rate	Once per 4 hours while release is in progress.
	<u>AND</u>	
	G.2 Restore monitoring instrumentation	30 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
H. Standby Gas Treatment System Noble Gas Activity Monitor low range channel inoperable	H.1 Take grab samples <u>AND</u> H.2 Analyze grab samples for isotopic activity to the required LLDs for principal noble gas gamma emitters (Table 3.11.2.1-1) <u>AND</u> H.3 Restore monitoring instrumentation	Once per 4 hours during operation of SGTS 30 days
I. Deleted		
J. SGTS Ventilation Monitoring System Effluent flow rate monitor or sample flow rate monitor inoperable	J.1 Estimate flow rate <u>AND</u> J.2 Restore monitoring instrumentation	Once per 4 hours during operation of SGTS 30 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
K. Required Actions and Completion Times not met for Conditions B through J	K.1 Explain why this inoperability was not corrected in a timely manner	In the next Radioactive Effluent Release Report per TS Section 5.6

TECHNICAL REQUIREMENT SURVEILLANCE

----- NOTE -----

Refer to Table 3.11.2.6-1 to determine which TRSs apply for each Monitoring Function.

SURVEILLANCE	FREQUENCY
TRS 3.11.2.6.1 Perform CHANNEL CHECK	24 hours
TRS 3.11.2.6.2 Deleted	
TRS 3.11.2.6.3 Perform Source Check	31 days
TRS 3.11.2.6.4 Perform CHANNEL FUNCTIONAL TEST	92 days
TRS 3.11.2.6.5 Perform CHANNEL CALIBRATION	24 months

TABLE 3.11.2.6-1 (Page 1 of 3)
RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

FUNCTION	APPLICABILITY	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS
1. REACTOR BUILDING VENTILATION MONITORING SYSTEM			
a. Noble Gas Activity Monitor (Low Range)	At all Times	1	TRS 3.11.2.6.1 TRS 3.11.2.6.3 TRS 3.11.2.6.4 TRS 3.11.2.6.5
b. Deleted			
c. Deleted			
d. Effluent System Flow Rate Monitor	At all Times	1	TRS 3.11.2.6.1 TRS 3.11.2.6.4 TRS 3.11.2.6.5
e. Sampler Flow Rate Monitor	At all Times	1	TRS 3.11.2.6.1 TRS 3.11.2.6.4 TRS 3.11.2.6.5

(continued)

TABLE 3.11.2.6-1 (Page 2 of 3)
RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

FUNCTION	APPLICABILITY	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS
2. TURBINE BUILDING VENTILATION MONITORING SYSTEM			
a. Noble Gas Activity Monitor (Low Range)	At all Times	1	TRS 3.11.2.6.1 TRS 3.11.2.6.3 TRS 3.11.2.6.4 TRS 3.11.2.6.5
b. Deleted			
c. Deleted			
d. Effluent System Flow Rate Monitor	<u>At all Times</u>	1	TRS 3.11.2.6.1 TRS 3.11.2.6.4 TRS 3.11.2.6.5
e. Sampler Flow Rate Monitor	At all Times	1	TRS 3.11.2.6.1 TRS 3.11.2.6.4 TRS 3.11.2.6.5

(continued)

TABLE 3.11.2.6-1 (Page 3 of 3)
RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

FUNCTION	APPLICABILITY	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS
3. STANDBY GAS TREATMENT SYSTEM (STGS) MONITOR			
a. Noble Gas Activity Monitor (Low Range)	During operation of SGTS ^(a)	1	TRS 3.11.2.6.1 TRS 3.11.2.6.3 TRS 3.11.2.6.4 TRS 3.11.2.6.5
b. Deleted			
c. Deleted			
d. Effluent System Flow Rate Monitor	During operation of SGTS ^(a)	1	TRS 3.11.2.6.1 TRS 3.11.2.6.4 TRS 3.11.2.6.5
e. Sampler Flow Rate Monitor	During operation of SGTS ^(a)	1	TRS 3.11.2.6.1 TRS 3.11.2.6.4 TRS 3.11.2.6.5
^(a) TRO 3.0.4.c is applicable.			