

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, *Code of Federal Regulations*, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	
1. Westinghouse Electric Company, LLC	3. License Number: SNM-1107
	Amendment 18
2. P.O. Box 355	4. Expiration Date: September 30, 2027
Pittsburgh, Pennsylvania 15230-0355	5. Docket No. 70-1151

6. Byproduct Source, and/or
Special Nuclear Material

7. Chemical and/or Physical
Form

8. Maximum amount that the Licensee
may possess at any one time
under this License

A. U-233

A. Any chemical or
physical form, limited
to laboratory use as
individual 1-gram
maximum quantities
in ventilated hoods,
glove boxes, or other
enclosures

A. _____ U-233

B. U-235 in uranium of
any enrichment

B. Any chemical or
physical form

B. _____ U-235

C. Uranium enriched
to $\leq 5\%$, including any
uranium daughter
isotopes

C. Any chemical or
physical form except
metal

C. _____ U-235

D. Pu-238/239

D. Sealed sources

D. _____

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1107

Docket or Reference Number

70-1151

Amendment 18

E. Transuranic elements
and fission products

E. Any

E. Not to exceed _____ alpha
per KgU (the limits on alpha and
gamma activity specified for
enriched processed UF₆ in
ASTM C996-96; Standard
Specification for Uranium
Hexafluoride Enriched to Less
than 5% U-235), and not to
exceed 5 grams of plutoniumF. Natural (or depleted)
UraniumF. Any chemical or physical
form except metal

F. _____

G. Depleted uranium

G. Flywheel

G. _____

H. Byproduct material

H. Surface contamination on
returned fuel assemblies,
fuel rods, equipment, and
associated miscellaneous
componentsH. _____ curies of any
byproduct material in the form of
contamination on nuclear fuel
assemblies, contaminated rods,
and equipment, with a maximum of
100 curies at any one time and not
to exceed Category 2 limits from
Table I-1 – Quantities of Concern
Threshold of Appendix I To
10 CFR Part 73

9. Authorized place of use: The licensee's existing facilities at Columbia, South Carolina.
10. This license shall be deemed to contain two sections: Safety Conditions and Safeguards Conditions. These sections are part of the license and the licensee is subject to compliance with all listed conditions in each section.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Date: November 2, 2015By: /RA/

Robert K. Johnson, Chief
Fuel Manufacturing Branch
Division of Fuel Cycle Safety, Safeguards,
and Environmental Review
Office of Nuclear Materials Safety
and Safeguards

NRC FORM 374A	U.S. NUCLEAR REGULATORY COMMISSION	Page 3 of 8
<p align="center">MATERIALS LICENSE SUPPLEMENTARY SHEET</p>		License Number SNM-1107
		Docket or Reference Number 70-1151
		Amendment 18

SAFETY CONDITIONS

- S-1 Authorized Use: For use in accordance with statements, representations, and conditions in the license application, dated June 27, 2007; or as revised, pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 70.32 or 10 CFR 70.72, and the supplements, dated July 18, 2007; September 13, 2007; April 21, 2008; June 10, 2008; September 30, 2008; August 30, 2010; October 4, 2010; September 7, 2010; November 22, 2010; December 15, 2010; February 22, 2011; April 8, 2011; and April 28, 2011; license application (Revision 2.0) dated June 29, 2012, June 29, 2012; November 9, 2012; April 5, 2012, July 6, 2012, October 4, 2012, November 13, 2012, and December 3, 2012; December 12, 2012, and January 15, 2013; Letters dated January 3, 2014, and March 24, 2014; July 2, 2014, July 18, 2014, and October 8, 2014; July 8, 2014; November 11, 2014, November 21, 2014, January 26, 2015, February 27, 2015, June 5, 2015, and June 18, 2015.
- S-2 The licensee shall maintain and execute the Site Emergency Plan for the Columbia Fuel Fabrication Facility, Revision 17, dated December 12, 2013, or as further revised by the licensee consistent with 10 CFR 70.32(i).
- S-3 Removed
- S-4 Removed
- S-5 Removed
- S-6 For nuclear criticality safety, as changes are implemented in the second Nuclear Criticality Safety Improvement Program (NCSIP-II) and other future such programs, the licensee shall incorporate justification for determining that accident sequences are incredible, specifically listing which item under Section 1.1.6.22 of the Application applies and a justification for using the item, documented in sufficient detail so that results can be reviewed.
- S-7 Removed
- S-8. Removed
- S-9 Notwithstanding the requirements in 10 CFR Part 30, Appendix A, Section II.C.1, Westinghouse shall demonstrate its continued eligibility to use the parent company guarantee within 120 days of the close of the fiscal year. This exemption remains in effect until either: (i) Westinghouse ceases to use a parent company guarantee from its current parent, Toshiba Corporation; (ii) Westinghouse fails to qualify for a parent company guarantee from its current parent, Toshiba Corporation; or (iii) the NRC's regulations in 10 CFR Part 30, Appendix A, Section II.C.1 are revised. If the submittal is sent by mail, Westinghouse is to use first class or overnight mail, postmarked no later than the date corresponding to the 120-day milestone.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1107

Docket or Reference Number

70-1151

Amendment 18

S-10 Notwithstanding the requirements of 10 CFR Part 20.1703(c)(5), Westinghouse may use a licensed health care professional to determine the medical fitness of personnel at the Columbia Fuel Fabrication Facility to use respiratory protection equipment. The respiratory protection program must be designed by, and under the supervision of, a physician. Though the physician need not administer each determination personally, the physician is ultimately responsible for the fitness determination. The physician is to be involved in the supervision of the fitness program, the review of overall results, individual cases that fall outside certain predetermined parameters, and supervision of personnel performing the tests.

S-11 The increase in the possession limit, authorized by Amendment 18, is for the storage of UF₆ cylinders.

SAFEGUARDS CONDITIONS**SECTION 1.0 – MATERIAL CONTROL AND ACCOUNTING**

- SG-1.1 The licensee shall follow Chapters 1.0 through 9.0 of its "Fundamental Nuclear Material Control Plan for the Columbia Fuel Fabrication Facility," which has been revised as indicated by Revision 45, dated June 8, 2015. Any further revision to this plan shall be made only in accordance with, and pursuant to, either the provisions of 10 CFR 70.32(c) or 70.34.
- SG-1.2 Operations involving special nuclear material which are not referenced in the Plan identified in Condition SG-1.1 shall not be initiated until an appropriate safeguards plan has been approved by the NRC.
- SG-1.3 Notwithstanding the requirements of the Fundamental Nuclear Material Control Plan identified in License Condition SG-1.1, the licensee may use: (1) a single standard for measurement control (including daily control limit monitoring and bias corrections) for any linear-response tube or rod scales, in any initially demonstrated to be linear over its range of use within the discrimination of the scale by calculating a bias at four levels across the range of use and demonstrating that the four results are not statistically different, and (2) that the continued linearity of response of the scales is verified by monthly calibration against at least four traceable standards covering the range of use.
- SG-1.4 Notwithstanding the requirement of Section 2.1.1, Block 6.b, of NUREG/BR-0006, which is incorporated via 10 CFR 74.15, to complete receiver's measurements of scrap receipts (following recovery processing) within 60 days of receipt, in cases in which the 60-day limit for confirmatory measurements cannot be met for UF₆ heels when Block 6.b Action Code N (of DOE/NRC Form 741) is used to book such receipts, the licensee shall complete receiver's measurements relative to recovering and measuring UF₆ heels no later than the next physical inventory.
- SG-1.5 With respect to Section 5.1.4 (b) of the Plan identified by Condition SG-1.1, "*allowed number*" within the phrase "*allowed number of defects*" is hereby specified as being:
- (i) up to two defects when each item within a batch of items has an assigned value equal to or less than 50 grams of U-235;

NRC FORM 374A	U.S. NUCLEAR REGULATORY COMMISSION	Page 5 of 8
<p align="center">MATERIALS LICENSE SUPPLEMENTARY SHEET</p>		License Number SNM-1107
		Docket or Reference Number 70-1151
		Amendment 18

(ii) no more than one defect when each item within a batch of items has an assigned value of less than 500 grams of U-235, but one or more items has an assigned value in excess of 50 grams of U-235; and

(iii) zero defects when any item within a batch of items contains 500 or more grams of U-235.

SG-1.6 Notwithstanding the first paragraph of Section 7.1 of the Plan identified by Condition SG-1.1, the licensee shall conduct shipper-receiver comparisons on all special nuclear materials (SNM) received (regardless of whether booked on the basis of receiver's or shipper's values), except for those materials identified in Section 7.1 of NUREG-1065 (Revision 2) as being exempted from shipper-receiver comparisons.

SECTION 2.0 – PHYSICAL PROTECTION OF SNM OF LOW STRATEGIC SIGNIFICANCE

SG-2.1 The licensee shall follow the physical protection plan entitled, "Physical Security Plan," Revision 45, dated June 8, 2015, and as it may be further revised in accordance with the provisions of 10 CFR 70.32(e).

SECTION 3.0 – INTERNATIONAL SAFEGUARDS

SG-3.1 The licensee shall follow Codes 1 through 6 of Transitional Facility Attachment No. 5A dated August 31, 1988, to the U.S./IAEA Safeguards Agreement. Such Transitional Facility Attachment shall be interpreted in accordance with Conditions SG-3.1.1 through SG-3.1.7.

SG-3.1.1 With respect to Transitional Facility Attachment Code 2:

The reference design information is that dated by the licensee on October 14, 1985. Information on the Facility also includes other facility information submitted via Concise Notes in accordance with 10 CFR 75.11(c).

SG-3.1.2 With respect to Transitional Facility Attachment Code 2.2:

Substantive changes to the information provided in the Columbia Plant Design Information Questionnaire (DIQ) means those changes requiring amendment of the Transitional Facility Attachment. Such changes shall be provided by letter to the NRC's Office of Nuclear Material Safety and Safeguards at least 70 days in advance of implementation.

Non-substantive changes to the information in the DIQ means those changes not requiring amendment of the Transitional Facility Attachment. Such changes shall be provided by Concise Note (from DOE/NRC-740M) within 30 days after the change is completed.

The types of modifications with respect to which information is required under 10 CFR 75.11,

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1107

Docket or Reference Number

70-1151

Amendment 18

to be submitted in advance, are those items stated in Code 2.2, specifically:

- (a) Any change in the purpose of type of facility means:

Any deviation from the described activities involving SNM and any change to the maximum enrichment and/or quantities of U-235 currently authorized by License No. SNM-1107, and/or as described in Paragraph 5 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c). Included also is any deviation from the described SNM production activities described in Paragraph 6 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

- (b) Any changes in the layout of the facility which affects safeguards implementation of the provisions of the Protocol means:

Any change in the existing facility and/or site layout or new addition affecting any activity involving SNM as described in Paragraphs 10 and 11 (per the referenced attachments of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c). Included also is any modification to, or deviation from, the data provided in Paragraphs 13 and 14 (per the referenced attachments) of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

- (c) Any change that makes the selected Key Measurement Points (KMPs) (as described in Code 3.1.2) inadequate for the Agency's accounting purpose means:

Any change to the KMPs as described in Code 3.1.2 of the Westinghouse-Columbia Transitional Facility Attachment to the U.S./IAEA Safeguards Agreement, or as modified in accordance with 10 CFR 75.11(c), that results in any KMP alteration affecting the purpose of KMPs as stipulated by 10 CFR 75.4(m).

- (d) Any change in the physical inventory procedures that would adversely affect the inventory taking for the agency's accounting purposes means:

Any change to the description data contained in Paragraph 34 (per the referenced attachments) of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c), that would not permit the agency to conclude an SNM material balance for the Westinghouse-Columbia facility.

- (e) Introduction of a significantly less accurate analytical method for accounting purposes means:

Any recalculation of the Relative Errors-Random and Systematic as listed in Attachment 36.2 referenced in Paragraph 36 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c), that results in the estimates of the random and systematic errors being affected by a factor of two or more.

NRC FORM 374A	U.S. NUCLEAR REGULATORY COMMISSION	Page 7 of 8
<p align="center">MATERIALS LICENSE SUPPLEMENTARY SHEET</p>		License Number SNM-1107
		Docket or Reference Number 70-1151
		Amendment 18

- (f) Decrease in the frequency of calibrating measuring equipment if it significantly decreases the accuracy of the materials accounting system means:

Any change that result in the estimates of the systematic error being affected by a factor of two or more.

- (g) Any change in the statistical procedures used to combine individual measurement error estimates to obtain limits of error for shipper/receiver (S/R) differences and material unaccounted for (MUF) means:

Any deviation from (or modification of) the equations and/or calculations outlined in Attachments 37.1, 37.2, and 37.3 referenced in Paragraph 37 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

SG-3.1.3 With respect to Transitional Facility Attachment Code 3.1.2:

KMP* – This is a KMP in which all shipper receiver differences (SRDs) must be recorded and reported, even if numerically zero. SRDs are computed and reported by the Nuclear Materials Management and Safeguards System upon receipt of the receiver's measurement results.

SG-3.1.4 With respect to Transitional Facility Attachment Code 4:

The licensee shall use the material composition codes documented in the DIQ dated October 14, 1985, and as modified by Concise Notes. Further, notwithstanding any other requirements for advance notification and/or reporting, the licensee may add or delete composition codes for nuclear material routinely processed and on inventory at Columbia Fuel Fabrication Facility immediately upon telephone notification to the Office of Nuclear Material Safety and Safeguards. Follow-up documentation, in the form of a Concise Note accompanied by appropriate changes to Table 1 of Attachment 34.8 to the DIQ shall be submitted within three regular workdays of the telephone notification.

SG-3.1.5 With respect to Transitional Facility Attachment Code 4.1:

Measured discards should be reported as a shipment to non-safeguards facility when shipped offsite to an authorized burial ground. (The IAEA system will not process measured discards as loss/disposal when they are shipped offsite.)

NRC FORM 374A	U.S. NUCLEAR REGULATORY COMMISSION	Page 8 of 8
<p align="center">MATERIALS LICENSE SUPPLEMENTARY SHEET</p>		License Number SNM-1107
		Docket or Reference Number 70-1151
		Amendment 18

SG-3.1.6 With respect to Transitional Facility Attachment Code 5.1.1:

For inventory changes, time of recording, upon means no later than the next regular workday (Monday through Friday).

For those occasions where natural or depleted uranium is inadvertently enriched above 0.711 percent through commingling with residual enriched uranium in process equipment, the resultant product shall be considered as being produced through a blending operation and the material category change shall be recorded upon obtaining measurement confirmation that a material category change has occurred.

SG-3.1.7 With respect to Transitional Facility Attachment Code 6.2.2:

For Concise Notes describing the anticipated operational programme, anticipated operational programme means: Anticipated physical inventory schedule.

