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SUBJECT: Provides response to NRC 980323 RAI concerning reactor pressure vessel beltline region cross-sectional developed inner-surface areas of plate & welds for Oconee Unit 1.

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September 30, 1998

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269
Request for Additional Information Concerning Reactor Pressure Vessel Beltline Region Cross-Sectional Developed Inner-Surface Areas of Plate and Welds for Oconee Nuclear Station Unit 1

In a letter dated March 23, 1998, the NRC issued a request for information concerning the reactor pressure vessel beltline region cross-sectional developed inner-surface areas of plate and welds for Oconee Nuclear Station Unit 1. The information was needed to perform an overall pressurized thermal shock analysis and revise Regulatory Guide 1.154. Attachment 1 to this letter provides the additional information that was requested by the NRC.

A copy of the NRC's pressurized thermal shock summary report for Oconee Unit 1 was included as enclosure 2 to the Staff's March 23, 1998, letter. During the development of the response to the NRC's request for information, Duke noticed that the information in the NRC's pressurized thermal shock summary report for Oconee Unit 1 needed to be updated. Thus, Attachment 2 contains a marked up copy of the NRC's pressurized thermal shock summary report for Oconee Unit 1 that can be used to update the NRC's database for Oconee Unit 1. Duke anticipates that similar changes are necessary for the NRC's pressurized thermal shock summary reports for Oconee Units 2 and 3.

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A001

If there are any questions regarding this submittal, please contact Michael Bailey at (864) 885-4390.

Very truly yours,

W. R. McCollum, Jr.
Site Vice President
Oconee Nuclear Station

Attachments

cc: L. A. Reyes, Regional Administrator
Region II

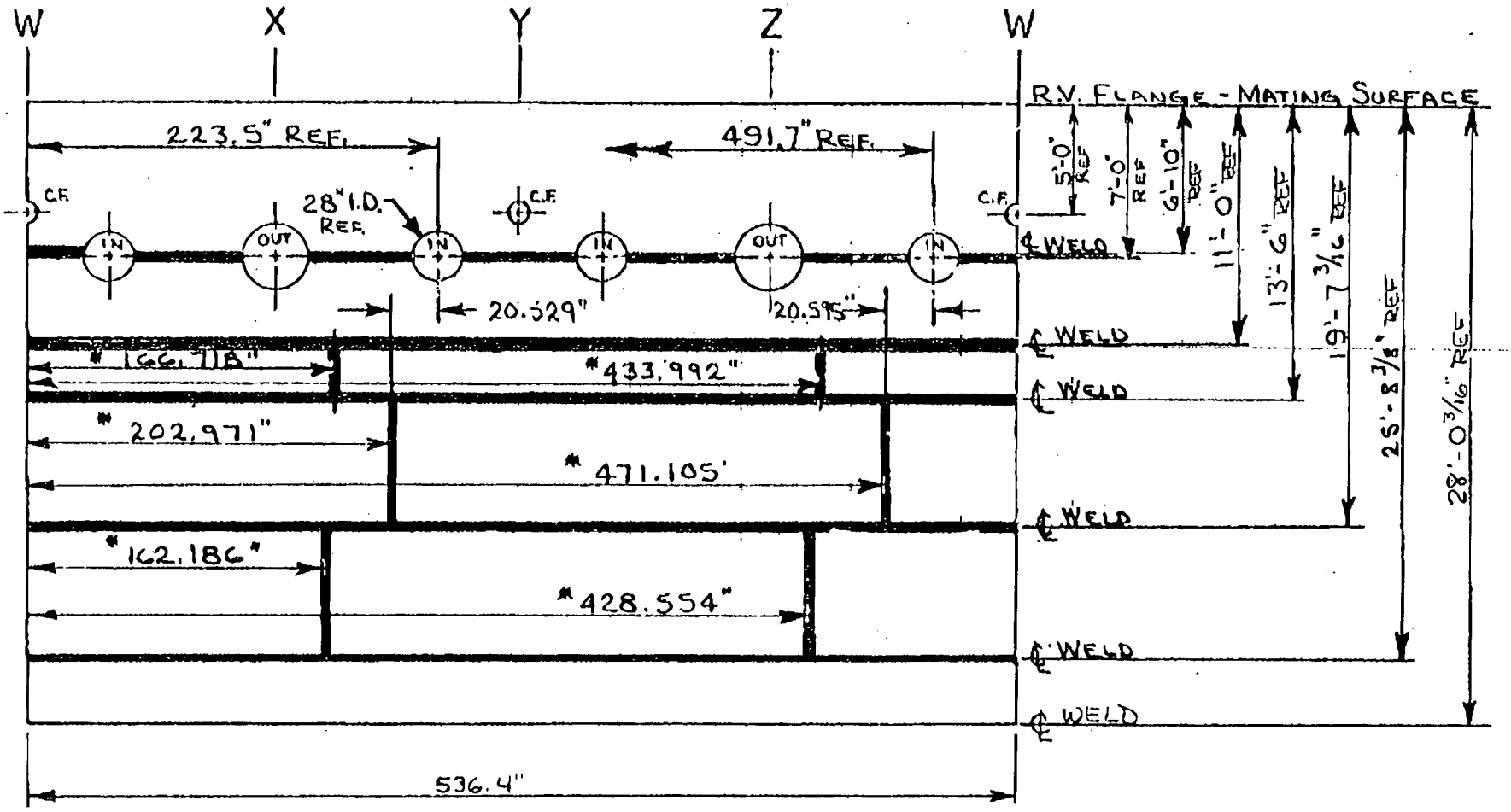
M. A. Scott, Senior Resident Inspector
Oconee Nuclear Site

D. E. LaBarge, Project Manager
NRR

Attachment 1

Reactor Pressure Vessel Beltline Region Cross-Sectional Developed Inner-Surface Areas of Plate and Welds for Oconee Nuclear Station Unit 1

Figure 3.2-2. Reactor Vessel Weldment Locations in Oconee Unit 1



Dimensions based on inside travel

Attachment 2

Markup of Pressurized Thermal Shock Summary
Report for Oconee Nuclear Station Unit 1

NRC - Reactor Vessel Integrity Database

PTS Summary Report

OCONEE 1

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Docket No: 50-269
EOL Date: 02/06/2013

	Beltline Identification		RTpts @EOL	Neutron Fluence @EOL	RTndt(u)	RTndt(u) METHOD	ΔRTndt(u) @EOL	Fluence Factor @EOL	Chem Factor	Chemistry Factor Method	Margin	Margin Method	Cu %	Ni %	P %	S %
1	LOWER NOZZLE BELT		~~127.3~~ 117.9	~~0.118~~ 0.079	3.0	B&W GENERIC	~~53.8~~ 44.2	0.451	119.25	TABLE	70.7	POSITION 1.1 (NO S DATA)	0.160	0.650	0.008	0.010
	FORGING	AHR54 (ZV2861)														
2	INTERMEDIATE SHELL		~~162.4~~ 164.0	~~0.796~~ 0.838	1.0	B&W GENERIC	~~97.8~~ 99.3	0.936	104.50	TABLE	63.6	POSITION 1.1 (NO S DATA)	0.150	0.500	0.008	0.010
	PLATE	C2197-2														
3	LOWER SHELL		~~136.1~~ 137.6	~~0.888~~ 0.930	1.0	B&W GENERIC	~~71.5~~ 72.9	0.960	74.45	TABLE	63.6	POSITION 1.1 (NO S DATA)	0.110	0.630	0.012	0.017
	PLATE	C2800-1														
4	LOWER SHELL		~~136.1~~ 137.6	~~0.888~~ 0.930	1.0	B&W GENERIC	~~71.5~~ 72.9	0.960	74.45	TABLE	63.6	POSITION 1.1 (NO S DATA)	0.110	0.630	0.012	0.017
	PLATE	C2800-2														
5	UPPER SHELL		~~113.8~~ 128.4	~~0.904~~ 0.932	1.0	B&W GENERIC	~~50.4~~ 63.7	0.972	58.01	~~OVERRIDE~~	~~50.4~~ 63.6	POSITION 2.1 (S DATA)	0.100	0.500	0.015	0.015
	PLATE	C3265-1														
6	UPPER SHELL		~~145.3~~ 146.0	~~0.904~~ 0.932	1.0	B&W GENERIC	~~80.7~~ 81.4	0.972	83.00	TABLE	63.6	POSITION 1.1 (NO S DATA)	0.120	0.600	0.010	0.016
	PLATE	C3278-1														
7	INTERMEDIATE SHELL AXIAL WELDS SA-1073		~~211.9~~ 213.8	~~0.628~~ 0.655	-5.0	B&W GENERIC	~~148.4~~ 150.4	0.870	170.60	TABLE	68.5	POSITION 1.1 (NO S DATA)	0.210	0.640	0.025	0.017
	WELD	1P0962														
8	NOZZLE BELT/INT. SHELL CIRC WELD SA-1135		~~113.2~~ 121.8	~~0.118~~ 0.079	-5.0	B&W GENERIC	~~69.8~~ 58.3	0.451	154.87	~~OVERRIDE~~	~~48.3~~ 68.5	POSITION 2.1 (S DATA)	0.250	~~0.540~~ 0.52	0.011	0.013
	WELD	61782														
9	INT./LOWER SHELL CIRC WELD SA-1229		~~233.4~~ 225.6	~~0.796~~ 0.844	~~5.0~~ 10.0	B&W GENERIC	~~170.0~~ 159.6	0.936	181.60	~~OVERRIDE~~	~~68.5~~ 56.0	POSITION 1.1 (NO S DATA)	0.260	~~0.610~~ 0.59	0.021	0.012
	WELD	71249														
10	LS/DUTCHMAN CIRC. WELD WF-9		59.4	0.010	-5.0	B&W GENERIC	~~48.1~~	0.110	146.09	~~OVERRIDE~~	48.3	POSITION 2.1 (S DATA)	0.210	0.590	0.000	0.000
	WELD	72445														
11	UPPER/LOWER SHELL CIRC WELD SA-1585		~~183.5~~ 216.8	~~0.888~~ 0.899	-5.0	B&W GENERIC	~~140.2~~ 153.3	0.960	146.00	~~OVERRIDE~~	~~48.3~~ 68.5	POSITION 2.1 (S DATA)	~~0.210~~ 0.22	~~0.590~~ 0.54	0.016	0.016
	WELD	72445														
12	LOWER SHELL AXIAL WELDS SA-1426		~~202.2~~ 201.6	~~0.729~~ 0.767	-5.0	B&W GENERIC	~~130.7~~ 138.1	0.911	~~152.25~~ 149.3	TABLE	68.5	POSITION 1.1 (NO S DATA)	~~0.200~~ 0.19	0.550	0.017	0.013
	WELD	8T1762														
13	LOWER SHELL AXIAL WELDS SA-1430		~~202.2~~ 201.6	~~0.729~~ 0.767	-5.0	B&W GENERIC	~~130.7~~ 138.1	0.911	~~152.25~~ 149.3	TABLE	68.5	POSITION 1.1 (NO S DATA)	~~0.200~~ 0.19	0.550	0.017	0.015
	WELD	8T1762														
14	UPPER SHELL AXIAL WELDS SA-1493		~~201.8~~ 203.1	~~0.723~~ 0.794	-5.0	B&W GENERIC	~~138.4~~ 139.6	0.909	~~152.25~~ 149.3	TABLE	68.5	POSITION 1.1 (NO S DATA)	~~0.200~~ 0.19	0.550	0.017	0.010
	WELD	8T1762														

Plant References and Beltline Material Notes

~~Fluence, chemical composition, and IRTndts are from BAW-2166.~~
↳ FTI document 86-1266231-01; PTS FTI document 32-5000884-00

~~Chemistry Factor for SA-1585 was calculated from Point Beach and Crystal River surveillance data that was reported in BAW-1803, Rev. 1. The surveillance welds were fabricated using the same weld wire heat number as SA-1585.~~

Chemistry Factor for SA-1135 was calculated from Ginna and Davis-Besse surveillance data that was reported in BAW-1803, Rev. 1. The surveillance welds were fabricated with the same weld wire heat number as SA-1135.

The UUSE data are from BAW-2222 (June 1994).

Surveillance data for heat 71249 was determined not credible, Table was used to calculate chemistry factor.

Enclosure 2