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TUCKMAN,M.S. Duke Power Co.
RECIP.NAME RECIPIENT AFFILIATION
Records Management Branch (Document Control Desk)

SUBJECT: Forwards summary of clarifications discussed in 980817
telcon re licensee 980410 & 0610 submittals for request for
relief from bolt torque or tension requirements of ASME
Boiler & Pressure Vessel Code, Section XI.

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August 18, 1998

U.S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

ATTENTION: Document Control Desk

SUBJECT: Duke Energy Corporation

Oconee Nuclear Station - Units 1, 2, & 3
Docket Nos. 50-269, 50-270, and 50-287

McGuire Nuclear Station - Units 1 & 2
Docket Nos. 50-369 and 50-370

Catawba Nuclear Station - Units 1 & 2
Docket Nos. 50-413 and 50-414

TAC Nos. MA1504, MA1505, MA1509, MA1510, MA1513,
MA1514, and MA1515

Request for Relief from Requirements of The ASME
Boiler and Pressure Vessel Code, Section XI in
Accordance with 10 CFR 50.55a (a) (3) (ii)
Duke Energy Corporation Serial Number 98-GO-002

In letters dated April 6, 1998 and June 10, 1998, Duke Energy Corporation submitted a request for relief from the bolt torque or tension requirements of the ASME Boiler and Pressure Vessel Code, Section XI, Subsection IWE, 1992 Edition with the 1992 Addenda, and submitted additional information for evaluating this request.

A telephone conversation between NRC officials and Duke representatives was held on August 17, 1998 and provided clarification of two issues necessary for the NRC to complete their review of this request. Attached is a summary of these clarifications.

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Please contact J. S. Warren at (704) 382-4986 if there are any questions or if additional information is needed on this request for relief.

Very truly yours,

A handwritten signature in black ink, appearing to read "M. S. Tuckman". The signature is fluid and cursive, with the first letters of the first and last names being capitalized and prominent.

M. S. Tuckman

MST/JSW

Attachment

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xc w/att:

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Attachment

DUKE ENERGY CORPORATION

Oconee Nuclear Station Units 1, 2, and 3
McGuire Nuclear Station Units 1 and 2
Catawba Nuclear Station Units 1 and 2

The following clarifications were discussed in a telephone conversation between NRC officials and Duke representatives on August 17, 1998. The issues are provided below, and clarifications are provided in responses to paraphrased questions.

Issue #1 - Scope of Request for Relief

Duke Energy Corporation's original submittal dated April 6, 1998 indicated that Relief was requested for the following Systems/Components:

"Pressure retaining bolting of Class MC pressure retaining components and metallic shell and penetration liners of Class CC components. The function of this bolting is to maintain containment leak-tight integrity."

In response to an NRC letter dated May 29, 1998 requesting additional information, Duke Energy Corporation submitted a letter dated June 10, 1998 which requested that the NRC limit the scope of this request to those pressure unseating bolted connections listed in the letter and all pressure seating bolted connections (not listed).

Question 1:

Would Duke Energy Corporation agree to amend the scope of this request to include all pressure retaining bolting of Class MC pressure retaining components and metallic shell and penetration liners of Class CC components, as originally stated in Duke Energy Corporation's letter dated April 6, 1998?

Duke Response to Question 1:

Yes. Duke Energy Corporation requests that the scope of this request for relief be amended to include all pressure retaining bolting of Class MC pressure retaining components

and metallic shell and penetration liners of Class CC components, as originally stated in Duke Energy Corporation's letter dated April 6, 1998.

Issue #2 - Clarification of Alternative Examination(s)

Duke Energy Corporation's original submittal dated April 6, 1998 indicated that the following alternative examination(s) are proposed in lieu of the bolt torque or tension tests required by The ASME Boiler and Pressure Vessel Code, Section XI, Division 1, 1992 Edition with the 1992 Addenda, Table IWE-2500-1, Examination Category E-G, Item E8.20:

V. Alternative Examination(s):

The adequacy of containment pressure retaining bolted connections shall be verified by the following:

- (1) Bolted connections shall receive a visual, VT-1 examination in accordance with requirements of Table IWE-2500-1, Examination Category E-G, Pressure Retaining Bolting, Item No. E8.10, and
- (2) A local leak rate test shall be performed on all containment penetrations, airlocks, and other pressure retaining bolted connections in accordance with 10 CFR 50, Appendix J. Current Technical Specifications utilize Option A of 10 CFR 50, Appendix J for Type B tests.

Please note that visual, VT-1 examinations at Oconee, McGuire, and Catawba Nuclear Stations include requirements to inspect for evidence of loose bolting.

Question 2:

Do the leak rate tests referenced in paragraph (2) above satisfy the requirements of The ASME Boiler and Pressure Vessel Code, Section XI, Division 1, 1992 Edition with the 1992 Addenda, Table IWE-2500-1, Examination Category E-P, Items E9.30 and E9.40?

Duke Response to Question 2:

Yes.