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 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287

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SUBJECT: Application for amends to licenses DPR-38, DPR-47 & DPR-55
 to TS for Oconee Nuclear Station.

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January 15, 1998

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287
Request for Technical Specification Amendment
for Test and Calibration Surveillances

Pursuant to 10 CFR 50.90, Duke Energy Corporation (Duke) hereby requests an amendment to the Technical Specifications for Oconee Nuclear Station. The amendment consists of a proposed one-time extension to the instrument channel test frequency for the wide range hot leg level, reactor vessel head level, and subcooling monitors. In addition, the amendment consists of a proposed one-time extension to the instrument calibration frequency for the wide range hot leg level, reactor vessel head level, core exit thermocouples, and subcooling monitors. Finally, a one-time extension of the engineered safeguards (ES) channels 5 and 6 test frequency and the Reactor Building Cooling system test frequency is requested in the proposed amendment.

The revised Technical Specification pages are included in Attachment 1. Attachment 2 contains the markup of the current Technical Specification pages. The Technical Justification for the amendment request is included in Attachment 3. Attachments 4 and 5 contain the No Significant Hazards Consideration Evaluation and the Environmental Impact Analysis, respectively.

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In preparation for the upcoming Oconee Unit 2 refueling outage, the testing and calibration requirements were reviewed to ensure compliance with the Technical Specifications. This review indicated that three instrument channel tests and four instrument calibrations were required to be completed prior to the upcoming Oconee Unit 2 refueling outage. In addition, the review indicated that the ES channels 5 and 6 test and the Reactor Building

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Cooling system test must be performed prior to the upcoming Oconee Unit 2 refueling outage.

The three instrument channel tests which must be performed are for the wide range hot leg level, reactor vessel head level, and subcooling monitors. The four instrument calibrations which must be performed are for the wide range hot leg level, reactor vessel head level, core exit thermocouples, and subcooling monitors. The instrument channel test and calibration are performed simultaneously by the same procedure and have a refueling outage frequency.

In Technical Specification Section 4.0, a refueling outage frequency is listed as having a maximum interval of 22 months and 15 days. The maximum interval will be exceeded on February 23, 1998 for Train A of the four instruments. Forced outages on Oconee Unit 2 have delayed the start of the refueling outage which will result in the channel test and calibration frequency being exceeded for the instruments, which are listed above, prior to the start of the refueling outage. The core exit thermocouples are inaccessible during plant operations. The wide range hot leg level instrument, reactor vessel head level instrument, and subcooling monitors cannot be calibrated without major procedure revisions and numerous entries into limited conditions of operation for Oconee Unit 2. The calibration of the wide range hot leg level instrument, reactor vessel head level instrument, and subcooling monitors during plant operations would provide minimal benefits while placing Oconee Unit 2 into limited conditions of operation. Therefore, Oconee Unit 2 needs to be placed in a cold shutdown condition to perform the channel test and to calibrate the instruments which are discussed in the amendment request.

In order to support the operation of Oconee Unit 2 to the scheduled refueling outage date of March 13, 1998, a one-time extension of the instrument channel test and calibration frequency is necessary. The test data from past channel tests and calibrations for the instruments were reviewed. The review indicated that a one-time extension of the channel test and calibration frequency to a maximum of 24 months will not result in any instrument drift outside of the allowed tolerance. The one-time extension results in the addition of 1 month and 15 days to the maximum allowed Technical Specification surveillance frequency of 22 months and 15 days.

In addition, the ES channels 5 and 6 test and the Reactor Building Cooling system test are performed simultaneously by the same procedure and have a refueling outage frequency. In Technical Specification Section 4.0, a refueling outage frequency is listed as having a maximum interval of 22 months and 15 days. The maximum interval will be exceeded on March 12, 1998 for the ES channels 5 and 6 test and the Reactor Building Cooling system test. Forced outages on Oconee Unit 2 have delayed the start of the refueling outage which will result in the test frequency being exceeded for the ES channels 5 and 6 test and the Reactor Building Cooling system test prior to the start of the refueling outage. The ES channels 5 and 6 test and the Reactor Building Cooling system test require the movement of plant equipment to the ES position which cannot be performed during plant operations. Therefore, Oconee Unit 2 needs to be placed in a cold shutdown condition to perform the ES channels 5 and 6 test and the Reactor Building Cooling system test.

In order to support the operation of Oconee Unit 2 to the scheduled refueling outage date of March 13, 1998, a one-time extension of the ES channels 5 and 6 test and the Reactor Building Cooling system test is necessary. The test data from past tests were reviewed and indicated that a one-time extension of the test frequency to a maximum of 23 months will not have any adverse affects on the operation of Oconee Unit 2. The one-time extension results in the addition of 15 days to the maximum allowed Technical Specification surveillance frequency of 22 months and 15 days.

This proposed change to the Technical Specifications has been reviewed and approved by the Plant Operations Review Committee and Nuclear Safety Review Board. The implementation of these changes does not result in an undue risk to the health and safety of the public. In addition, the Oconee Updated Final Safety Analysis Report has been reviewed and no changes are necessary to support this amendment request.

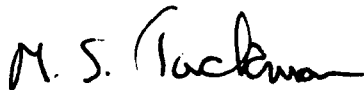
Pursuant to 10 CFR 50.91, a copy of this proposed amendment is being sent to the South Carolina Department of Health and Environmental Control for review, and as deemed necessary and appropriate, subsequent consultation with the NRC staff.

This submittal will impact the current Technical Specification markup and discussion of changes in the Oconee

Improved Technical Specification (ITS) submittal. A supplement to the ITS submittal will be provided at a later date or the one-time surveillance extensions will be removed by a another Oconee Technical Specification amendment prior to the approval of the ITS submittal.

If there are any questions regarding this submittal, please contact Michael Bailey at (864) 885-4390.

Very truly yours,



M. S. Tuckman,
Executive Vice President
Nuclear Generation Department

MEB

Attachments (5)

cc: L. A. Reyes, Regional Administrator
Region II

M. A. Scott, Senior Resident Inspector
Oconee Nuclear Site

D. E. LaBarge, Project Manager
NRR

M. Batavia,
DHEC

M. S. Tuckman, being duly sworn, states that he is Executive Vice President of Duke Energy Corporation, that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this revision to the Oconee Nuclear Station License Nos. DPR-38, DPR-47, and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.

M. S. Tuckman

M. S. Tuckman, Executive Vice President

Subscribed and sworn to before me this 15th day of January, 1998.

Mary P. Delms

Notary Public

My Commission Expires:

JAN 22, 2001