

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-261

REC: OREILLY J P
NRC

ORG: BANKS H R
CAROLINA PWR & LIGHT

DOCDATE: 01/20/78
DATE RCVD: 01/24/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 0 ENCL 1

LICENSEE EVENT REPT. NO. 50-261/77-33 CONCERNING THE AUXILIARY FEEDWATER
SYSTEM FAILING TO OPEN AS COMMANDED.

PLANT NAME: H B ROBINSON - UNIT 2

REVIEWER INITIAL: XRL
DISTRIBUTOR INITIAL:

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BRANCH CHIEF REID**W/4 ENCL

INTERNAL: REG FILE**W/ENCL
I & E**W/2 ENCL
SCHROEDER/IPPOLITO**W/ENCL
NOVAK/CHECK**W/ENCL
KNIGHT**W/ENCL
HANAUER**W/ENCL
EISENHUT**W/ENCL
SHAO**W/ENCL
KREGER/J. COLLINS**W/ENCL
L. CROCKER**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
HOUSTON**W/ENCL
GRIMES**W/ENCL
BUTLER**W/ENCL
TEDESCO**W/ENCL
BAER**W/ENCL
VOLLMER/BUNCH**W/ENCL
ROSA**W/ENCL

EXTERNAL: LPDR'S
HARTSVILLE, SC**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+2P

CONTROL NBR: 780300166

***** THE END *****


004

CP&L

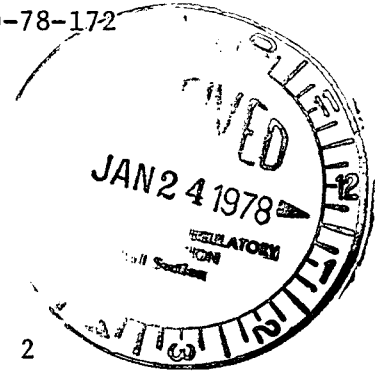
Carolina Power & Light Company
January 20, 1978

REGULATORY DOCKET FILE COPY
KID 312
B03034

FILE: NG-3516 (R)

SERIAL: GD-78-172

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Region II, Suite 1217
230 Peachtree Street, N.W.
Atlanta, Georgia 30303



H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 77-33

Dear Mr. O'Reilly:

In accordance with Section 6.9.2.b of the Technical Specifications for the H. B. Robinson Steam Electric Plant, Unit 2, the attached Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in Regulatory Guide 1.16, Revision 4.

Yours very truly,

H. R. Banks
Manager
Nuclear Generation

DCS:as

Attachment

cc: Messrs. W. G. McDonald
E. Volgenau

780300166

A002/s
0/1

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME						LICENSE NUMBER						LICENSE TYPE				EVENT TYPE											
01	S	C	H	B	R	2	0	0	-	0	0	0	0	-	0	0	4	1	1	1	0	0	3				
7	8	9				14	15										25	26				30	31	32			
CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER				EVENT DATE				REPORT DATE															
01	CONT		L					0	5	0	-	0	2	6	1	1	2	2	1	7	7	0	1	2	0	7	8
7	8			57	58	59	60	61							68	69						74	75				80

EVENT DESCRIPTION

02	While performing PT 22.1C the motor driven Auxiliary Feed Pump discharge valve to "A"																									80
03	Steam Generator, V2-16A failed to open as required. The Steam Driven Auxiliary Feed																									80
04	Pump and valves were test operated satisfactorily. The valve appeared to be binding																									80
05	on the seat. It was manually broken off the seat and operated electrically several																									80
06	times with satisfactory results. (HBR2 RO 77-33)																									80

SYSTEM CODE	CAUSE CODE	COMPONENT CODE				PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION			
07	CH	E	V	A	L	V	O	P	N	L	2	0	0	N
7	8	9	10	11	12			17	43				47	48

CAUSE DESCRIPTION

08	Valve V2-16A apparently failed to open due to excessive seating pressure caused by the																									80
09	valve being heated by backleakage from the downstream check valve. This check valve																									80
10	will be inspected and refurbished during the refueling outage in February 1978.																									80

FACILITY STATUS	% POWER	OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION								
11	E	1	0	0	N/A		A			N/A				
7	8	9	10	12	13			44	45	46				80
FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY				LOCATION OF RELEASE								
12	Z	Z				N/A								
7	8	9	10	11				44	45					80

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION												
13	000	Z	N/A											
7	8	9	11	12	13									80

PERSONNEL INJURIES

NUMBER	DESCRIPTION													
14	000	N/A												
7	8	9	11	12										80

OFFSITE CONSEQUENCES

15	N/A																									80		
7	8	9																										80

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION																										
16	Z	N/A																									
7	8	9	10																								80

PUBLICITY

17	N/A																									80		
7	8	9																										80

ADDITIONAL FACTORS

18																										80		
7	8	9																										80

19																										80		
7	8	9																										80

NAME: R. B. Starkey, Jr., *[Signature]* PHONE: (803) 332-1351

Supplemental Information
For
Reportable Occurrence 77-33

1. Report No: 50-261/77-33

2a. Report Date: January 13, 1978

2b. Occurrence Date: December 21, 1977

3. Facility: H. B. Robinson Unit No. 2
Hartsville, South Carolina 29550

4. Identification of Occurrence:

At 2025 hours on December 21, 1977, while performing PT 22.1C, Valve V2-16A in the Auxiliary Feedwater System failed to open as commanded. This constitutes a reportable occurrence in accordance with Technical Specification paragraph 6.9.2.b.(2).

5. Conditions Prior to Occurrence:

The plant was operating at 100% reactor power.

6. Description of the Occurrence:

At 2025 hours on December 21, 1977, while performing PT 22.1C, Valve V2-16A failed to open as commanded. The valve apparently was binding on the seat because of heat from backleakage of a downstream check valve. The valve was isolated and then broken off the seat manually. Once the valve was off the seat, it was cycled electrically twice with satisfactory results. The isolation valve was then opened and V2-16A tested again. As the valve was being cycled, an audible indication of check valve backleakage was noted. The next day Maintenance requested that V2-16A be operated again for their observation. The valve was cycled at that time and was found to operate satisfactorily.

7. Designation of Apparent Cause of Occurrence:

The cause of the occurrence is suspected to be backleakage of a check valve located between the feedwater line to "A" Steam Generator and the Valve V2-16A.

8. Analysis of Occurrence:

The check valve not seating completely allowed V2-16A to close with cold water going through it during the previous valve test and then to heat up from the backleakage of feedwater. This increase in temperature apparently caused the valve to become more tightly seated and thus resulted in greater resistance when commanded to open. When V2-16A was isolated with a manual valve and manually backed off the seat, it operated satisfactorily. PT 22.1C was successfully completed and valve V2-16A was declared operable at 0905 hours on December 22, 1977.

9. Corrective Action:

The check valve which had indications of backleakage will be inspected during the refueling outage starting in February 1978. The valve seat will be refurbished to reduce the probability of further backleakage.

10. Failure Data:

Similar events to this occurred on August 17, 1976 and on August 17, 1977. (Reportable Occurrence 76-15 and 77-19).

RECEIVED DOCUMENT
CONTROL DESK

1215 JAN 24 AM 11 39

U.S.M.C.
DISTRIBUTION SERVICES
BRANCH