

SEP 05 2014

LES-14-00102-NRC

ATTN: Document Control
Director,
Office of Nuclear Material Safety and Safeguards
U.S Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Louisiana Energy Services, LLC
NRC Docket Number: 70-3103

Subject: Technical Support for SBM1005 Operations

Reference: Letter from J. Laughlin (UUSA) to T. Gody (NRC), LES-14-00099-NRC,
Request for Operation Readiness Review of SBM1005

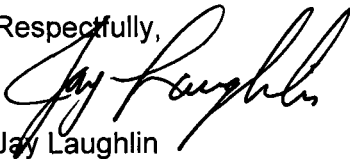
To Whom it May Concern:

On August 29, 2014 UUSA submitted a request for an NRC Operational Readiness Review (ORR) of separations building module (SBM) 1005 (reference). In support of this request, this submittal is to provide technical information regarding the commencement of operations in SBM1005 under UUSA's current license basis. The supporting technical information is included as enclosure 1 of this letter. This letter does not constitute a request for licensing action from the NRC.

Upon initial licensing of UUSA, the facility description included three SBMs. As requested, enclosure 2 includes the 10 CFR 70.72(c) evaluation that was completed to return SBM1005 to the UUSA's LBDs. This evaluation determined that prior NRC approval was not required for the change.

Should there be any questions concerning this submittal, please contact Timothy Knowles, UUSA Licensing and Performance Assessment Manager, at 575.394.6212.

Respectfully,



Jay Laughlin
Chief Nuclear Officer and Head of Operations

Enclosures: 1. License Basis for SBM1005
2. 10 CFR 70.72(c) Evaluation 2012-0477

NM5501

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Enclosure 1

License Basis for SBM1005

License Basis for SBM1005

Upon initial NRC licensing of UUSA, the facility description included three separation building modules (SBM). All initial License Basis Documents (LBD) , NUREG-1790 – the facility Environmental Impact Statement (EIS), and NUREG-1827 – the initial facility Safety Evaluation Report (SER) described this layout. Therefore, a complete NEPA environmental review with a conclusion of no significant impacts for the construction and operation of three SBMs was documented in an EIS and publically vetted by the NRC.

In September 2009, CC-EG-2008-0212 was initiated to bring UUSA's LBDs into alignment with the current plant design (approved by UUSA's design agency). The design of SBM 1003 and 1005 structures and components had not been completed yet and were subject to change. This resulted in the removal of SBM1003 and 1005 from the LBDs until the design was finalized and could be accurately put back into the LBDs. Thus, the intent was not to remove SBMs based on regulatory requirements / alignment, but rather to ensure that the LBDs described accurate design information. The transition to SBM1003 operation was successfully completed after subsequent LBD updates; as described below.

In February 2010, CC-LS-2010-0005 was initiated to add the design of SBM1001 extension structures and systems to the LBDs.

In March 2010, CC-EG-2010-0114 was initiated to add SBM1003 back to the LBDs to allow the start of construction. This change did not include the design of SBM1003 UF6 systems.

In December 2011, License Amendment Request (LAR) 11-02 was approved by the NRC. The SER associated with this LAR stated that the UUSA analysis of operations at 9.9 SWU (out to and including SBM 1009) were bounding and provided adequate assurance of meeting 10 CFR 70.61 performance requirements.

In May 2012, CC-EG-2010-0333 was initiated to add the design of SBM1003 UF6 systems to the LBDs.

In August 2012, CC-LS-2012-0009 was initiated to add SBM1005 back into the LBDs to bound the buildings construction in the UUSA license basis.

In summary, both the construction and the operation of SBM1005 were analyzed for environmental considerations and safety concerns during initial licensing of the facility. The conclusion of these analyses were that the proposed action presented no significant environmental impacts and that the safety bases were adequate to meet performance requirements. These conclusions were documented in the EIS and the SER completed at the time of initial licensing. Prior to the NRC's Operational Readiness Review and prior to putting SBM1005 into operation, UUSA will complete changes, under its own authority, to incorporate the operation of SBM1005 into the current license basis. Assay 1005 decommissioning funding will be in place at least 30 days prior to the introducing of UF6 into SBM1005. Finally, to bring SBM1005 into operation an Operational Readiness Review must be completed by NRC Region II, Division of Fuel-Cycle Facility Inspections.

Enclosure 2

10 CFR 70.72(c) Evaluation 2012-0477

Procedure Title: 10 CFR 70.72 (c) Evaluations for Proposed Changes

LS-3-1000-04-F-1
10 CFR 70.72(c) Screen/Evaluation

LS-3-1000-04
Rev. 10
Level 3 - Information Use
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70.72(c) Tracking Number: 2012-0477**Rev. Number:** 0**Facility Change/Document/Program Number:** CC-LS-2012-0009**Rev. Number:** 0**Part I – Applicability Determination****Document/Activity Information****Brief Description of the Activity (what is being changed and why):**

The proposed configuration change is a revision to the license basis documents (Emergency Plan, Fundamental Nuclear Material Control Plan, Integrated Safety Analysis Summary and the Safety Analysis Report) to describe the SBM-1005 facility, including the associated Utilities Service Module (USM). The revision includes changes to relevant figures, functional description and general site descriptions. The reason for the change is to facilitate partial facility expansion.

Address the questions below for all aspects of the change. If the answer is YES for any portion of the change, then the identified Governing Process(es) applies to that portion of the change rather than 10 CFR 70.72(c).

Note: It is not unusual to have more than one process apply to a change.

I. Does the proposed activity involve a change to the following:	Governing Process	YES	NO
A. Quality Assurance Program Description (QAPD)? [see Item (xix) in SER on QAPD]	QA-3-2000-06	<input type="checkbox"/>	<input checked="" type="checkbox"/>
B. Environmental Report (ER) or implementing procedure? [see 10 CFR 51.60(b)(2)]	EN-3-1000-01	<input type="checkbox"/>	<input checked="" type="checkbox"/>
C. Emergency Plan (EP) or implementing procedure? [see 10 CFR 70.32(i) & 10 CFR 40.35(f)]	EP-3-1000-02	<input checked="" type="checkbox"/>	<input type="checkbox"/>
D. Fundamental Nuclear Material Control Plan (FNMCP) or implementing procedure? [see 10 CFR 70.32(c)(1)(iii)]	MC-3-1000-01	<input checked="" type="checkbox"/>	<input type="checkbox"/>
E. Security Plan (SP) or implementing procedure? [see 10 CFR 70.32(d) and (e)]	SY-3-1000-10	<input type="checkbox"/>	<input checked="" type="checkbox"/>
F. SPPP for Protection of Classified Matter or implementing procedure? [see 10 CFR 95.19]	SY-3-1000-11	<input type="checkbox"/>	<input checked="" type="checkbox"/>
G. SPPP for Protection of Classified Matter at the ET-US location of the National Enrichment Facility or implementing procedure? [see 10 CFR 95.19]	SY-3-1000-11	<input type="checkbox"/>	<input checked="" type="checkbox"/>
H. Safety Analysis Report (SAR)? [See Materials License Condition 30]	LS-3-1000-06	<input checked="" type="checkbox"/>	<input type="checkbox"/>

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Facility Change/Document/Program Number: <u>CC-LS-2012-0009</u>	Rev. Number: <u>0</u>

Part I – Applicability Determination

II. Does the proposed activity involve:	Governing Process	YES	NO
A. Maintenance that restores SSCs to their as-designed condition?	Maint Procedures	<input type="checkbox"/>	<input checked="" type="checkbox"/>

III. Does the proposed activity involve a change to a:	Governing Process	YES	NO
A. Administrative or managerial procedures that do not affect management measures or process safety information?	AD-3-1000-01	<input type="checkbox"/>	<input checked="" type="checkbox"/>

IV. Does the proposed activity involve:	Governing Process	YES	NO
A. Previous 70.72(c) Screen, 70.72(c) Evaluation or NRC approval?	If YES, reference the previous 70.72(c) Tracking Number or NRC approval below:	<input type="checkbox"/>	<input checked="" type="checkbox"/>
Previous 70.72(c) Tracking Number or NRC approval: - - - Not Applicable - - -			

NOTE

"ALL aspects of the change" indicates that all appropriate licensing and design bases documents that are considered to be applicable to the current change have been reviewed.
If applicable, the evaluator and reviewer have reviewed the previous 70.72(c) Screen or 70.72(c) Evaluation quality record to ensure it meets current program expectations.

- ☐ One or more processes above control **ALL** aspects of the activity; thus a 70.72(c) Review is **NOT** required.
- ☒ **IF ANY PORTION** of the activity IS NOT controlled by one or more of the processes above, **THEN** a 70.72(c) Review IS required and should be initiated by completing the 70.72(c) Screen.

10CFR70.72(c)
Evaluator: T. D. Foster / T.D. Foster **Date:** September 17, 2012

10CFR70.72(c)
Reviewer: A. Blackshear / [Signature] **Date:** September 17, 2012
Printed Name **Signature**

NOTE:

A copy of the completed Applicability Determination must be transmitted to Licensing.

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Part II – 10 CFR 70.72(c) Screen

N/A

I. Description of the Change:

II. Applicability Determination

List other applicable processes and any relevant basis for identifying the process(es) used during completion of the Applicability Determination:

III. 70.72(c) Screening Questions (Check correct response and provide basis for each answer)

- A. Does the proposed change involve a change to the site, structures, processes, systems, equipment, components, computer programs, procedures or instructions that could impact worker or public health and safety? ☐ Yes ☐ No
Basis for answer:
- B. Does the proposed change involve a change to the site, structures, processes, systems, equipment, components, computer programs, procedures or instructions that could impact the control of licensed material? ☐ Yes ☐ No
Basis for answer:
- C. Does the proposed change involve a change to the ISA, ISA Summary, IROFS, items that affect the function of IROFS, or IROFS boundary definition? ☐ Yes ☐ No
Basis for answer:
- D. Does the proposed change involve a change to the Process Safety information provided in the Safety Program? ☐ Yes ☐ No
Basis for answer:
- E. Does the proposed change involve a change to the Management Measures provided in the Safety Program? ☐ Yes ☐ No
Basis for answer:
- F. Does the proposed change involve a change that is prohibited by a License Condition or an Order? ☐ Yes ☐ No
Basis for answer:

This section was skipped as allowed by licensing procedure LS-3-1000-04,
10 CFR 70.72(c) Evaluations for Proposed Changes Step 5.1.3
TDF TD3 September 17, 2012

N/A

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Part III – 10 CFR 70.72(c) Evaluation

I. Description of Change:

The proposed configuration change is a revision to the license basis documents (Emergency Plan, Fundamental Nuclear Material Control Plan, Integrated Safety Analysis Summary and the Safety Analysis Report) to describe the SBM-1005 facility (including the associated USM). The revision includes changes to relevant figures, functional description and general site descriptions. The reason for the change is to facilitate partial facility expansion.

II. 10 CFR 70.72(c) Evaluation:

NOTES: Provide a written response providing the basis for the answer to each question below.

- (c)(1)(i) Does the proposed change create new types of accident sequences that, unless mitigated or prevented, would exceed the performance requirements of 10 CFR 70.61 and that have not previously been described in the ISA Summary? ☐ YES ☒ NO

Basis for answer:

The proposed change describes the basic SBM-1005 facility (including the USM). It does not describe any new processes, material or activities. This change is to allow initial construction of the SBM-1005 and USM structure. The actual construction will be pursuant to the Operate While Construct program.

The proposed change does not describe any new features, hazards, scenarios, capabilities or any new function that could introduce a new hazard, or increase the probability or consequence of an existing hazard. Therefore, it will not create any new types of accident sequences that unless mitigated or prevented would exceed the performance requirements of 10 CFR 70.61 and that have not previously been described in the ISA Summary

- (c)(1)(ii) Does the proposed change use new processes, technologies or control systems for which the licensee has no prior experience? ☐ YES ☒ NO

Basis for answer:

The proposed change describes the basic SBM-1005 facility (including the USM) to enable initial construction. No processes, technologies, or control systems for SBM-1005/USM are described with this change; therefore, this change does not use new processes, technologies or control systems for which the LES has no prior experience.

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- (c)(2) Does the proposed change involve removal, without at least an equivalent replacement of the safety function, of an IROFS that is listed in the ISA Summary and is necessary for compliance with the performance requirements of § 70.61? ☐ YES ☒ NO

Basis for answer:

The proposed change describes the basic SBM-1005/USM and does not involve the operation of SBM-1005/USM. No IROFS specific to the SBM-1005 facility or USM are introduced or discussed. The IROFS applicable to buildings containing uranic material (IROFS27e, IROFS35, IROFS36a, etc.,) are already identified in the ISA Summary. The proposed change will not, in and of itself, involve removal, without at least an equivalent replacement of the safety function, of an IROFS that is listed in the ISA Summary (table 3.8-1, Items Relied On For Safety) and is necessary for compliance with the performance requirements of 10 CFR 70.61.

- (c)(3) Does the proposed change alter any IROFS, listed in the ISA Summary, that is the sole item preventing or mitigating an accident sequence that exceeds the performance requirements of 10 CFR 70.61? ☐ YES ☒ NO

Basis for answer:

The proposed change is only intended to allow initial construction to commence on the SBM-1005 and USM facility. This proposed change is not involved with, nor does it discuss, the operation of SBM-1005 or the USM. No sole IROFS specific to these facilities are introduced or discussed. The IROFS applicable to buildings containing uranic material (IROFS27e, IROFS35, IROFS36a, etc.,) are already identified in the ISA Summary. Therefore, the proposed change will not, in and of itself, alter any IROFS listed in the ISA Summary (table 3.8-2) that is the sole item preventing or mitigating an accident sequence that exceeds the performance requirements of 10 CFR 70.61.

- (c)(4) Is the proposed change otherwise prohibited by 10 CFR 70.72, license condition, or order? ☐ YES ☒ NO

Basis for answer:

The proposed change has been evaluated in accordance with approved LES procedures established to ensure compliance with 10 CFR 70.72. US NRC Materials License, SNM-2010, contains conditions related to the types, quantities, control, and records of Source and Special Nuclear Material, introduction of UF₆ to site modules, the decommissioning plan, boundaries and control circuitry of IROFS, storage of DUF₆, specific Quality Assurance terminology associated with 10 CFR 21.3, "Reporting of Defects and Noncompliance", security requirements for Controlled Access Area and Physical Security Plan, restrictions on changes to the Safety Analysis Report (SAR), and designation of areas for dissemination of classified information. SAR changes are evaluated in accordance with approved LES procedures established to ensure compliance Materials License Condition 30. The proposed change does not challenge any of these issues; and therefore, is not prohibited by any license condition. NRC Orders EA-06-193, EA-06-230, EA-06-264, and EA-07-086 issued for LES concern Additional Security Measures, Protection of Safeguards Information, and Fingerprinting and Criminal Background Checks. The proposed change is not related to any security issue; therefore, the proposed change is not prohibited by any NRC orders.

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Part III – 10 CFR 70.72(c) Evaluation

III. Conclusion of 10 CFR 70.72(c) Evaluation:

Based upon the results of this Evaluation: (Select one of the following)

- ☒ IF the answers to all the questions in Section II are "NO", THEN a License Amendment is not required. Proceed with implementation using applicable processes.
- ☐ IF the answer to any question in Section II is "Yes", THEN STOP, and either revise the proposed activity and perform another 70.72(c) review **OR** contact Licensing to initiate a License Amendment Request as NRC approval of the change is required prior to implementation.

IV. References used to perform the evaluation (if not provided in the response to each question).

Integrated Safety Analysis Summary
 Safety Analysis Report
 US NRC Materials License, SNM-2010
 NRC Orders EA-06-193, EA-06-230, EA-06-264, and EA-07-086

V. Review and Approval:

10 CFR 70.72(c) Evaluator:	<u>T. D. Foster</u> (Printed Name)	<u>T. D. Foster</u> (Signature)	Date: <u>September 17, 2012</u>
10 CFR 70.72(c) Reviewer:	<u>A. Blackshear</u> (Printed Name)	<u>[Signature]</u> (Signature)	Date: <u>September 17, 2012</u>
10 CFR 70.72(c) Approver:	<u>W. Padgett</u> (Printed Name)	<u>[Signature]</u> (Signature)	Date: <u>9/17/12</u>

NOTE:

A copy of the completed 10 CFR 70.72(c) Evaluation must be transmitted to Licensing.