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SECRETARY

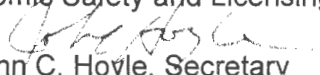
DOCKET NUMBER
PROD. & UTIL. FAC. 50-295/304-LA
UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001
June 10, 1998

DOCKETED
USNRC

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OFFICE
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MEMORANDUM TO: B. Paul Cotter, Jr.
Chief Administrative Judge
Atomic Safety and Licensing Board Panel

FROM: 
John C. Hoyle, Secretary

SUBJECT: PETITION FOR LEAVE TO INTERVENE SUBMITTED
BY EDWIN D. DIENETHAL

Attached is a petition to intervene dated June 4, 1998, and submitted by Edwin D. Dienethal. The petition was filed in response to a notice of a proposed determination by the staff that the issuance of a license amendment to the Commonwealth Edison Company for the Zion Nuclear Power Station (Docket Nos. 50-295/304) would involve no significant hazards considerations. The amendment would make several technical specification changes, reinstate License Conditions that were deleted by a previous amendment and modify staffing requirements and management titles to reflect a shutdown status. The notice was published in the Federal Register at 63 Fed. Reg. 25101, 25105 (May 6, 1998) (copy attached).

The petition to intervene is being referred to you for appropriate action in accordance with 10 C.F.R. Sec. 2.772(j).

Attachment: as stated

cc: Commission Legal Assistants
OGC
CAA
OPA
EDO
NRR
Michael I. Miller, Esquire
Attorney for Licensee
Edwin D. Dienethal

19193

Edwin D. Dienethal

8354 47th Court
Kenosha, WI 53142

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USNRC

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OFFICE OF THE
SECRETARY
ADJUTANT GENERAL

June 4, 1998

Secretary of the Commission
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

**Re: *Applications and Amendments to Facility Operating Licenses Involving No
Significant Hazards Considerations:
Docket Nos. 50-295 and 50-304***

To whom it may concern:

Pursuant to *10 C.F.R. 2.714 I*, Edwin D. Dienethal, do hereby petition for leave to intervene in Commonwealth Edison Company's application to amend its Facility Operating Licence for Zion Station, Unit 1 and 2, Lake County, Illinois. Notice of the application was printed at *63 Fed. Reg. 25, 101 (1998)*.

I have direct information concerning the threat to public health and safety posed by Zion Nuclear Station including, but not limited to, intentional violations of safety-related procedures. See, e.g., testimony placed on public record in the matter of *Robarge v. Commonwealth Edison*, 98-ERA-2, hearing dates May 18-22, 1998.

The specific reasons for my petition are outlined below pursuant to the factors listed in *10 C.F.R. § 2.714(d)(1)*:

- (1) The nature of the petitioner's right under the Act to be made a party to the proceeding.

I have standing to make this petition as a resident of Kenosha, Wisconsin who resides within fifty miles of Zion Nuclear Station and for other reasons which will be stated in my amended petition.

- (2) The nature and extent of the petitioner's property, financial, or other interest in the proceeding.

The operations at Zion Nuclear Station directly impact the health and safety of myself and my family, as well as the health and safety of the community. My financial interests in this proceeding stem from my ownership of both real and personal property in close proximity to Zion Nuclear Station. These and other interests which will be stated in my amended petition constitute the nature and extent of my interests in this proceeding.

- (3) The possible effect of any order which may be entered in the proceeding on the petitioner's interest.

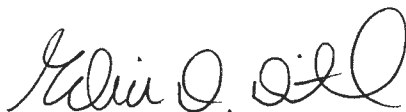
An order entered on the petitioner's interest will protect the health and safety of myself and my community as we reside in close proximity to the plant. In addition, an order which decreases the safety risk posed by Plant Zion will serve to maintain the property value of my house and the economic viability of the surrounding area. These and other effects which will be stated in my amended petition represent the possible effect of any order which may be entered in my interest.

I, therefore, petition for leave to intervene in each and every aspect of Commonwealth Edison's proposed amendments, including, but not limited to, Commonwealth Edison's application to amend its operating license to change management titles and responsibilities. Some of my other specific contentions include the following:

- (A) Radiation Protection (RP) is still required during plant decommission because of the continued presence of radioactive systems. RP is required to ensure the health and safety of the maintenance worker and the health and safety of the public in the event that a system is opened.
- (B) Shift Control Room Engineer (SCRE) is still needed to evaluate and take appropriate actions in the operations of spent fuel and its safety systems. Without an SCRE affirmative actions may not be taken in the case of a Loss of Coolant Accident (LCA).
- (C) Under-staffing of the plant during the decommissioning process is a risk to the health and safety of the public.

I am in the process of locating counsel for representation in this matter and reserve the right to amend this petition at a later date.

Sincerely,



Edwin D. Dienethal

cc: Office of the General Counsel, U.S. NRC
Michael I. Miller, Esq.

Safeguards Policy and Procedures Letter 1-50, Revision 1, is not warranted.

Alternatives to the Proposed Action

The proposed action is to amend NRC Source Material License SUA-648, for reclamation of the Heap Leach Area, as requested by Umetco. Therefore, the principal alternatives available to NRC are to:

1. Approve the license amendment request as submitted; or
2. Amend the license with such additional conditions as are considered necessary or appropriate to protect public health and safety and the environment; or
3. Deny the amendment request.

Based on its review, the NRC staff has concluded that the environmental impacts associated with the proposed action do not warrant either the limiting of Umetco's future operations or the denial of the license amendment. Additionally, in the TER prepared for this action, the staff has reviewed the licensee's proposed action with respect to the criteria for reclamation, specified in 10 CFR Part 40, Appendix A, and has no basis for denial of the proposed action. Therefore, the staff considers that Alternative 1 is the appropriate alternative for selection.

Finding of No Significant Impact

The NRC staff has prepared an EA for the proposed renewal of NRC Source Material License SUA-648. On the basis of this assessment, the NRC staff has concluded that the environmental impacts that may result from the proposed action would not be significant, and therefore, preparation of an Environmental Impact Statement is not warranted.

The EA and other documents related to this proposed action are available for public inspection and copying at the NRC Public Document Room, in the Gelman Building, 2120 L Street N.W., Washington, DC 20555.

Notice of Opportunity for Hearing

The Commission hereby provides notice that this is a proceeding on an application for a licensing action falling within the scope of Subpart L, "Informal Hearing Procedures for Adjudications in Materials and Operators Licensing Proceedings," of the Commission's Rules of Practice for Domestic Licensing Proceedings and Issuance of Orders in 10 CFR Part 2 (54 FR 8269). Pursuant to § 2.1205(a), any person whose interest may be affected by this proceeding may file a request for a hearing. In accordance with § 2.1205(c), a request for a hearing must be filed within thirty (30) days from the date of publication of

this Federal Register notice. The request for a hearing must be filed with the Office of the Secretary either:

(1) By delivery to the Rulemakings and Adjudications Staff of the Office of the Secretary at One White Flint North, 11555 Rockville Pike, Rockville, MD 20852; or

(2) By mail or telegram addressed to the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Rulemakings and Adjudications Staff.

Each request for a hearing must also be served, by delivering it personally or by mail to:

(1) The applicant, Umetco Mineral Corporation, P.O. 1029, Grand Junction, CO 81502;

(2) The NRC staff, by delivery to the Executive Director of Operations, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852, or

(3) By mail addressed to the Executive Director for Operations, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

In addition to meeting other applicable requirements of 10 CFR Part 2 of the Commission's regulations, a request for a hearing filed by a person other than an applicant must describe in detail:

(1) The interest of the requestor in the proceeding;

(2) How that interest may be affected by the results of the proceeding, including the reasons why the requestor should be permitted a hearing, with particular reference to the factors set out in § 2.1205(g);

(3) The requestor's areas of concern about the licensing activity that is the subject matter of the proceeding; and

(4) The circumstances establishing that the request for a hearing is timely in accordance with § 2.1205(c).

Any hearing that is requested and granted will be held in accordance with the Commission's "Informal Hearing Procedures for Adjudications in Materials and Operator Licensing Proceedings" in 10 CFR Part 2, Subpart L.

Dated at Rockville, Maryland, this 30th day of April 1998.

For the Nuclear Regulatory Commission.

Joseph J. Holonich,

Chief, Uranium Recovery Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 98-11980 Filed 5-5-98; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Biweekly Notice; Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Considerations

I. Background

Pursuant to Public Law 97-415, the U.S. Nuclear Regulatory Commission (the Commission or NRC staff) is publishing this regular biweekly notice. Pub. L. 97-415 revised section 189 of the Atomic Energy Act of 1954, as amended (the Act), to require the Commission to publish notice of any amendments issued, or proposed to be issued, under a new provision of section 189 of the Act. This provision grants the Commission the authority to issue and make immediately effective any amendment to an operating license upon a determination by the Commission that such amendment involves no significant hazards consideration, notwithstanding the pendency before the Commission of a request for a hearing from any person.

This biweekly notice includes all notices of amendments issued, or proposed to be issued from April 10 through April 24, 1998. The last biweekly notice was published on April 22, 1998 (63 FR 19964).

Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The Commission has made a proposed determination that the following amendment requests involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The basis for this proposed determination for each amendment request is shown below.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period.

However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received before action is taken. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administration Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC. The filing of requests for a hearing and petitions for leave to intervene is discussed below.

By June 5, 1998, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC and at the local public document room for the particular facility involved. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or

petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of a hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room for the particular facility involved.

equipment necessary to achieve and maintain post-fire safe shutdown in areas requiring alternate capabilities does not involve a significant reduction in margin.

The licensee has evaluated the use of the mobile SSD battery cart to provide the power source for the Automatic Depressurization System (ADS) valves under certain scenarios where the valves are needed to achieve cold shutdown and determined that it does not involve a significant hazards consideration for the reasons discussed below.

(1) No significant increase in the probability or consequences of an accident previously evaluated is involved.

The accident previously evaluated is the postulated fire requiring alternate shutdown capability. The probability of a previously evaluated fire is not increased significantly because the mobile SSD batteries do not create significant new ignition sources or any other fire initiators. The consequences of a previously evaluated fire are not increased significantly because the mobile SSD batteries do not significantly increase the fire loading in the plant, do not interfere with the plant's ability to extinguish a fire, and are fully capable of fulfilling the designed safety function.

The associated systems related to this proposed change are not affected in a way that could impact the initiation of any accident sequence for the Quad Cities Station. No modes of operation are introduced by the proposed change such that adverse consequences result.

The probability of an accident involving the use of the mobile SSD batteries would not be increased significantly by this proposed change because the use is not significantly different from the alternative manual attachment of a power source to the ADS valves.

The consequences of an accident involving the use of the mobile SSD batteries are not increased because the only significant consequences would be a delay in achieving cold shutdown and that would have no different consequences than would a delay due to an accident related to the currently used manual power source.

(2) The possibility of a new or different kind of accident from any accident previously evaluated is not created.

The proposed change for the Quad Cities Station does not create the possibility of a new or different kind of accident from that previously evaluated. Because the mobile SSD batteries simply provide a different form of manually connecting a source of power to the ADS valves, the use of the mobile SSD batteries does not present new or different kinds of accidents related to such manual actions. Finally, because no new modes of operation are introduced by the proposed change, the change does not create the possibility of a new or different kind of accident that could be related to new modes of operation.

(3) No significant reduction in the margin of safety is involved.

The analytic framework for determining the extent to which a proposed change affects

the margin of safety has been discussed above and, so will not be repeated here. In this case, a review of the proposed changes shows that they will not have an adverse impact on the ability to achieve and maintain safe shutdown. Several features associated with the use of the mobile SSD batteries show, as discussed above, that it provides an effective method for achieving and maintaining safe shutdown following a fire. In particular, use of the mobile SSD batteries reduces the overall complexity of the cold shutdown repairs required to supply power to the ADS valves and is familiar to plant personnel from their training on its use for other purposes.

Design calculations regarding capabilities of the mobile SSD batteries show they will be capable in fulfilling their intended safety function for their design basis Appendix R scenario. Reliability of the mobile SSD batteries will be maintained by augmented quality standards. This will entail the conduct of appropriate maintenance and surveillance which is designed to ensure that the mobile batteries will function as intended. Reliability of this power source is further enhanced by the circumstance that there are two mobile SSD batteries, thus permitting one to act as a backup to the other.

Under these circumstances, the margin of safety for achieving cold shutdown using the ADS valves is not reduced significantly, if at all, by the use of non-safety related mobile SSD batteries to power the ADS valves. Although safety-related station batteries had previously been used in this function, the method for attaching those batteries was more prone to human error than the method which has been developed for the mobile SSD batteries. Moreover, substantial steps have been taken to provide a high level of reliability for the mobile SSD batteries. Overall, therefore, the ability to achieve and maintain safe shutdown in the event of a fire has not been reduced by this change in the source of power to the ADS valves.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92 are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

Local Public Document Room location:

Dixon Public Library, 221 Hennepin Avenue, Dixon, Illinois 61021
Attorney for licensee: Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60603

NRC Project Director: Stuart A. Richards

Commonwealth Edison Company,
Docket Nos. 50-295 and 50-304, Zion Nuclear Power Station, Units 1 and 2, Lake County, Illinois

Date of amendment request: March 30, 1998.

Description of amendment request:
The proposed amendments would restore the Zion Custom Technical

Specifications (CTS) that had been replaced with Improved Technical Specification by a previous amendment and would reinstate License Conditions that were deleted by that previous amendment. The proposed amendment would also modify the CTS to allow the use of Certified Fuel Handlers to satisfy shift staffing requirements and would change management titles and responsibilities to reflect the permanently shutdown organization.

Basis for proposed no significant hazards consideration determination:
As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Does the change involve a significant increase in the probability or consequences of an accident previously evaluated?

With a plant permanently shutdown and defueled the spectrum of accidents and events that remain credible is significantly reduced. As discussed below the proposed changes do not affect the probability or consequences of any accidents that do remain credible.

The restoration of the CTS which were replaced with the ITS by Amendments 178/165 cannot increase the probability or consequences of any event or accident because the amendment was never implemented. The CTS have been maintained as the legally binding Technical Specifications in effect at Zion Station. The reinstatement of the five License Conditions deleted by Amendments 178/165 is an administrative change in that the requirements contained in the License Conditions had been relocated elsewhere and are now being restored exactly as they were before the amendment was issued. Since the actual requirements have not changed there can be no change in the probability or consequences of any accident or event.

The changes in management titles and responsibilities will not increase the probability or consequences of any accident or event because these changes are administrative and will not result in any decrease in the quality of management applied to Zion Station. The changes are commensurate with the significant reduction in site activities, site staffing, and risk to public health and safety that occurs when an operational nuclear power plant transitions to a permanently shutdown and defueled plant. Responsible individuals will have the authority to commit the personnel and resources necessary to fulfill their obligations for safe storage and handling of nuclear fuel. The change of position designations will have no effect on the frequency of occurrence of accident or event initiators, or on their consequences.

The changes to allow use of Certified Fuel Handlers in lieu of personnel licensed in accordance with 10 CFR part 55 will not increase the probability or consequences of an accident or event because the Certified Fuel Handler Training and Retraining program (which will be approved by the

NRC) has been developed using a Systems Approach to Training as defined in 10 CFR 55.4. This approach provides assurance that the Certified Fuel Handlers have the knowledge, skills, and abilities that are commensurate with the tasks to be performed (i.e., the proper monitoring, handling, storage, and cooling of nuclear fuel). Therefore the frequency of occurrence of accident or event initiators is not increased and the consequences of the accidents or events are unaffected.

The changes in shift staffing numbers and crew composition will not increase the probability or consequences of an accident or event. These staffing changes are commensurate with the quantity, complexity, and hazard level of the activities required for storage and handling of nuclear fuel. The elimination of the Shift Control Room Engineer does not affect any accident or event initiator or consequence since the previous specification would not have required that the position be manned with both units shut down. The elimination of the requirement for a Radiation Protection Person on shift will have no effect on the frequency of occurrence of accidents or events, nor on the consequences of the accident or event.

The changes in verbiage to eliminate any implication that units are operational will not increase the probability or consequences of an accident or event because they are largely editorial changes and do not increase the frequency of occurrence of [or] event initiators, nor do they increase the consequences.

Therefore this proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Does the change create the possibility of a new or different kind of accident from any accident previously evaluated?

The changes proposed by this amendment do not involve new structures, systems, or components, or the use of existing structures, systems, or components in a new manner. Consequently no new failure mechanisms are introduced. The design and operation of structures, systems, or components is unaffected by:

The restoration of CTS,

The reinstatement of the five License Conditions deleted by Amendments 178/165,

The changes in management titles and responsibilities,

The changes to allow use of Certified Fuel Handlers in lieu of 10 CFR [Part] 55 licensed personnel,

The changes in shift staffing numbers and crew composition, or

The changes in verbiage to eliminate any implication that units are operational.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

Does the change involve a significant reduction in a margin of safety?

One of the License Conditions that would be reinstated by this amendment establishes limits that help ensure that the assumptions of the fuel handling accident analysis remain valid. License Condition 2.C.(7).b limits the

weight of loads carried over fuel stored in the spent fuel pool to the weight of a single fuel assembly plus the tool for moving that assembly. This weight limit ensures that the number of fuel rods broken in a fuel handling accident does not exceed the maximum number of fuel rods assumed to break in the accident analysis. Consequently, this change continues to provide assurance that the margin of safety involving the number of fuel rods broken in the accident will not be reduced.

Therefore, these changes do not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

Local Public Document Room location:

Waukegan Public Library, 128 N. County Street, Waukegan, Illinois 60085

Attorney for licensee: Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60603

NRC Project Director: Stuart A. Richards

Duke Energy Corporation (DEC), et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

Date of amendment request: May 27, 1997, as supplemented by a letter dated April 20, 1998.

Description of amendment request:

The proposed amendments would revise the Technical Specifications (TS) of each unit to conform with NUREG-1431, Revision 1, "Standard Technical Specifications—Westinghouse Plants." The Commission had previously issued a Notice of Consideration of Issuance of Amendments in the *Federal Register* on July 14, 1997 (62 FR 37628) covering all the proposed changes that were indeed within the scope of NUREG-1431. In DEC's May 27, 1997, submittal, there are proposed changes that are beyond the scope of NUREG-1431, which were thus not covered by the staff's July 14, 1997, notice. The following descriptions and no significant hazard analyses cover only those beyond-scope changes. Associated with each change are administrative/editorial changes such that the new or revised requirements would fit into the format of NUREG-1431.

1. This proposed change affects the surveillance requirement currently contained in Sections 4.6.6.1 and 4.6.6.2, regarding the containment valve injection water system. The requirement to assure adequate capacity to maintain system pressure for at least 30 days

would be deleted, the required system pressure of 16.2 pounds per square inch gauge (psig) would be replaced with a surge tank pressure of 36.4 psig, and the system would be tested at lower pressures and more restrictive leak rates.

2. Section 3.9.2.1, regarding the boron dilution mitigating system, currently requires both trains to be operable in Mode 6 (refueling). DEC proposed to add a note stating that the system may be blocked during core reloading until two assemblies are loaded into the core. Adequate shutdown margin will continue to be controlled and verified by other specifications. This blocking would prevent inadvertent actuation of the system, which could distract the operating personnel, but would not diminish the monitoring function of the system.

3. DEC proposed to change the definition of 'dose equivalent iodine-131.' Subsequently, this proposed change was withdrawn by letter dated April 20, 1998.

4. DEC proposed to change Section 3.3.3.6 regarding accident monitoring instrumentation. Specifically, the change would (a) increase the time allowed to return the required number of channels to operable; and (b) permit continued operation if one channel is inoperable given certain conditions are met, instead of requiring shutdown.

5. DEC proposed to change Section 4.6.4.1 regarding surveillance requirements for the hydrogen monitors (combustible gas control). Specifically, this would eliminate the channel operational test, and extend the channel check frequency from once per 12 hours to once per 31 days.

6. DEC proposed to change Section 3.4.6.1 regarding reactor coolant leakage detection systems; a system comprising diverse instruments such as gaseous radioactivity monitoring, containment floor and equipment sump monitoring, etc. In addition to the instruments specified by this section, the plant has other installed instruments such as monitors for humidity, temperature, etc., which can provide indication for reactor coolant leakage. Currently, this specification allows operation up to 30 days if the containment floor and equipment sump monitoring system is inoperable. The change would impose a requirement to perform a precision water balance of the reactor coolant system every 24 hours during this period. The change would also reduce the number of monitors required operable provided compensatory measures are performed or diverse instruments continue to be available.