



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

July 30, 1997

DOCKETED  
USNRC

'97 JUL 30 P1:17

OFFICE OF SECRETARY  
DOCKETING & SERVICE  
FBI/DOH

Mr. Patrick M. Sears  
Box 13  
Fairfield, Pennsylvania 17320

Dear Mr. Sears:

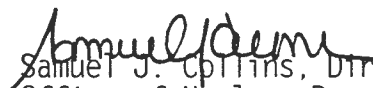
The NRC staff has completed its review of your Petition dated August 19, 1996, and as supplemented on April 14, 1997, in regard to the Maine Yankee Atomic Power Station (Maine Yankee), operated by the Maine Yankee Atomic Power Company (MYAPCO or the licensee). The Petition was evaluated pursuant to 10 CFR § 2.206.

For the reasons given in the enclosed Director's Decision (DD 97-17), your request to fine MYAPCO and Yankee Atomic Electric Company (YAEC) if records regarding use of the emergency core cooling system computer code known as RELAP5YA have not been kept in accordance with YAEC's computer code quality assurance procedures is granted in part. Your request that the NRC inspect all users of RELAP and fine those users not operating within required computer code verification procedures is also granted in part.

A copy of the Director's Decision will be filed with the Secretary of the Commission in accordance with 10 CFR § 2.206(c). As provided by Section 2.206, the Director's Decision will become the final action of the Commission 25 days after issuance unless the Commission, on its own motion, institutes review of the Decision.

I have also enclosed a copy of the notice of the issuance of DD-97-17, which is being filed with the Office of the Federal Register for publication. The notice includes the complete text of DD-97-17.

Sincerely,

  
Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Enclosures: 1. Director's Decision DD-97-17  
2. Notice

18416

cc w/encl:

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State Nuclear Safety Advisor  
State Planning Office  
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Mr. P. L. Anderson, Project Manager  
Yankee Atomic Electric Company  
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Mr. Michael B. Sellman, President  
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329 Bath Road  
Brunswick, ME 04011

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'97 JUL 30 P1:17

OFFICE OF SECRETARY  
DOCKETING & SERVICE  
BRANCH

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-309

MAINE YANKEE ATOMIC POWER COMPANY

MAINE YANKEE ATOMIC POWER STATION

ISSUANCE OF DIRECTOR'S DECISION UNDER 10 CFR 2.206

Notice is hereby given that the Director, Office of Nuclear Reactor Regulation, has taken action with regard to a Petition for action under 10 CFR 2.206 received from Mr. Patrick M. Sears (Petitioner), dated August 19, 1996, and revised on April 14, 1997, with regard to the Maine Yankee Atomic Power Station.

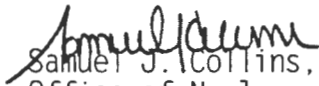
The Petitioner requested the NRC to (1) fine Maine Yankee Atomic Power Company (MYAPCO) and Yankee Atomic Electric Company (YAEC) if records regarding use of the computer code RELAP have not been kept in accordance with YAEC's computer code quality assurance procedures and (2) inspect all users of RELAP and fine those users not operating within required computer code verification procedures.

The Director of the Office of Nuclear Reactor Regulation has acknowledged parts (1) and (2) of the Petition. The reasons for this decision are explained in the "Director's Decision Pursuant to 10 CFR 2.206" (DD-97-17), the complete text of which follows this notice and is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room for the Maine Yankee Atomic Power Station located at the Wiscasset Public Library, High Street, P. O. Box 367, Wiscasset, Maine 04578.

A copy of this Decision will be filed with the Secretary of the Commission for the Commission's review in accordance with 10 CFR 2.206(c). As provided by this regulation, this Decision will constitute the final action of the Commission 25 days after the date of issuance unless the Commission, on its own motion, institutes review of the Decision within that time.

Dated at Rockville, Maryland, this 30th day of July 1997.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

DOCKETED  
USNRC

DD-97-17

'97 JUL 30 P1:17

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF NUCLEAR REACTOR REGULATION

OFFICE OF SECRETARY  
DOCKETING & SERVICE  
BRANCH

Samuel J. Collins, Director

|                                     |   |                   |
|-------------------------------------|---|-------------------|
| In the Matter of                    | ) |                   |
|                                     | ) |                   |
| MAINE YANKEE ATOMIC POWER COMPANY,  | ) | Docket No. 50-309 |
| YANKEE ATOMIC ELECTRIC COMPANY      | ) | (10 CFR 2.206)    |
|                                     | ) |                   |
| (Maine Yankee Atomic Power Station) | ) |                   |

DIRECTOR'S DECISION UNDER 10 CFR 2.206

I. INTRODUCTION

On August 19, 1996, Patrick M. Sears (Petitioner) filed a Petition with the U.S. Nuclear Regulatory Commission (NRC) pursuant to Section 2.206 of Title 10 of the Code of Federal Regulations (10 CFR § 2.206). Petitioner requested the NRC to (1) fine Maine Yankee Atomic Power Company (MYAPCO) and Yankee Atomic Electric Company (YAEC) if records regarding use of the computer code RELAP5YA have not been kept in accordance with YAEC's computer code quality assurance (QA) procedures, and (2) inspect all users of RELAP and fine those users not operating within required computer code verification procedures.

As the basis for these requests, the Petition states that (1) the May 5, 1989, oral statement of Steve Nichols, then licensing supervisor of MYAPCO, to Petitioner, then NRC Project Manager for Maine Yankee Atomic Power Station (MYAPS), that RELAP5YA was "operable" and would be used for subsequent reloads was false; (2) no computer code inspections were performed by NRC before a 1992 inspection at YAEC by Mr. Sears, and not again until 1995; (3) when Mr. Sears was in the Vendor Inspection Branch, he was told not to do any more

computer code inspections; (4) RELAP is widely used; (5) RELAP has been shown to have serious deficiencies; and (6) the RELAP problem is not confined to the MYAPS but is endemic to the industry as a whole.

On September 24, 1996, Mr. William T. Russell, then Director of the Office of Nuclear Reactor Regulation, acknowledged receipt of the Petition. By letter dated April 14, 1997, Petitioner supplemented his Petition by correcting his characterization of Mr. Nichols' comment, substituting the word "operational" for "operable".

## II. BACKGROUND

As a result of concerns regarding small-break loss-of-coolant accident (SBLOCA) analyses of emergency core cooling systems (ECCS) raised by the 1979 accident at Three Mile Island Unit 2, and pursuant to 10 CFR § 50.54(f), the NRC required licensees to submit revised, documented SBLOCA analyses which were to meet the guidance provided in NRC's "Clarification of TMI Action Plan Requirements" (NUREG-0737 or TMI Action Plan), Item II.K.3.30. and II.K.3.31. In response to the guidance of Item II.K.3.30, on January 14, 1983, Maine Yankee submitted a report, YAEC-1300P, "RELAP5YA: A Computer Program for Light Water Reactor System Thermal-Hydraulic Analysis" to the NRC. In January 1989, the NRC approved RELAP5YA for use by Maine Yankee as a 10 CFR Part 50, Appendix K, evaluation model, acceptable to demonstrate compliance with the requirements of 10 CFR § 50.46, "Acceptance criteria for emergency core cooling systems for light water nuclear power reactors." RELAP5YA is a generic, non-plant-specific LOCA computer code for calculating ECCS performance over the small-break portion of the break spectrum.

Item II.K.3.31 of the TMI Action Plan states that licensees are to

submit plant-specific calculations using the SBLOCA evaluation model approved by the NRC pursuant to Item II.K.3.30. In response to TMI Action Plan Item II.K.3.31, YAEC prepared for Maine Yankee a plant-specific Appendix K, RELAP5YA SBLOCA evaluation model analysis and prepared a report in June 1993 identified as YAEC-1868: "Maine Yankee Small Break LOCA Analysis." The SBLOCA analysis described in YAEC-1868 was used to prepare Core Performance Analysis Reports (CPARs) which were submitted to the NRC as part of Maine Yankee's reload analyses for Cycle 14 and Cycle 15 operations, and was the SBLOCA analysis of record throughout Cycle 14 operations; it was not used during Cycle 15 operations because of the intervening January 3, 1996, "Confirmatory Order Suspending Authority for and Limiting Power Operation and Containment Pressure (Effective Immediately), and Demand for Information" (Order).<sup>1</sup> 61 *Fed. Reg.* 735 (January 10, 1996).

On December 4, 1995, the NRC received allegations that, among other things, YAEC, acting as agent for the licensee, knowingly performed inadequate analyses of the emergency core cooling system (ECCS) to support two license amendment applications to increase the rated thermal power at which MYAPS operates to 2630 Mwt, and then to 2700 Mwt. It was further alleged that YAEC management knew that the ECCS for Maine Yankee, if evaluated in accordance with 10 CFR § 50.46, using the RELAP5YA SBLOCA evaluation model, did not meet licensing requirements.

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<sup>1</sup> Among other things, the Order limited operation of MYAPS to 2440 Mwt, pending NRC review and approval of a plant-specific SBLOCA analysis which conforms to TMI Action Plan Items II.K.3.30 and II.K.3.31 and which meets the requirements of 10 C.F.R. § 50.46.

In response to the allegations, NRC dispatched an Assessment Team to YAEC headquarters between December 11 and 14, 1995, to examine, among other things, SBLOCA analyses, especially the SBLOCA analysis which supported the licensee's operating Cycle 15 reload application. Based on the Assessment Team review, and a meeting held with the licensee on December 18, 1995, the NRC staff issued its January 3, 1996, Order. The Order concluded, *inter alia*, that the licensee had not demonstrated that computer code RELAP5YA would reliably calculate the peak cladding temperature for all break sizes in the small-break LOCA spectrum for Maine Yankee and that, for a variety of reasons, the plant-specific application of RELAP5YA did not conform to the requirements of 10 CFR § 50.46 and thus was not acceptable for use by the licensee. The Order required the licensee to submit a SBLOCA analysis specific to Maine Yankee for operation at power levels up to 2700 Mwt, which must meet the requirements of 10 CFR § 50.46, and which must conform to the guidance of NUREG-0737, Items II.K.3.30 and 31, "SBLOCA Methods" and "Plant-specific Analysis," respectively, and NUREG-0737, Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps During LOCA". The Order suspended authority to operate Maine Yankee at 2700 Mwt maximum power and limited power to 2440 Mwt, pending NRC review and approval of the required SBLOCA analysis. MYAPCO submitted the required SBLOCA analysis specific to Maine Yankee on April 25, 1996, and the NRC staff is currently reviewing it.

The NRC also initiated an investigation by the NRC Office of Investigations (OI) to examine possible wrongdoing. The NRC staff is currently reviewing the results of that investigation.



### III. DISCUSSION

- A. Do MYAPCo and other NRC licensees who use RELAP operate within required computer code verification procedures?

Petitioner requests that the NRC inspect all users of RELAP and fine those users not operating within required computer code verification procedures. The staff presumes that the phrase "required computer code verification procedures," as used by Petitioner, means the conditions, if any, of the NRC's approval of the computer code, as well as the licensee or vendor quality assurance (QA) procedures pursuant to 10 CFR Part 50, Appendix B.

There are many vintages of RELAP, which was developed by Idaho National Engineering Laboratory, such as RELAP4, RELAP5/MOD1, RELAP5/MOD2, and RELAP5/MOD3 (higher suffix numbers indicate more current vintages). Major improvements were made in each new vintage, including the use of more sophisticated modeling of two-phase flow. For example, RELAP5/MOD1 has a "mixture" model with five governing equations, whereas RELAP5/MOD2 has a full two-fluid treatment with six equations.

Each vintage of RELAP has many versions, representing primarily modifications in supporting models on constitutive relationships and corrections of errors. Idaho National Engineering Laboratory maintains a reporting system for problems discovered by users of the code, which are prioritized and referred to the code development staff for resolution. Therefore, it cannot be assumed that a problem with a particular RELAP vintage or version also exists in other RELAP vintages or versions.

Vendors or licensees who use RELAP codes to support license applications normally take a specific vintage or version of RELAP and create their own

variations by making modifications and adding certain features, such as those required by 10 CFR Part 50, Appendix K. The RELAP codes used by different vendors and licensees are not necessarily developed from the same versions or vintages of RELAP. For example, the RELAP5YA code used by YAEC for Maine Yankee SBLOCA analysis was derived from RELAP5/MOD1, while most other RELAP codes used for the ECCS analyses of NRC-licensed nuclear plants were derived from different vintages, namely, RELAP4 or RELAP5/MOD2.

Before a vendor-modified or licensee-modified RELAP code is used for licensing applications, it must be reviewed and approved by the staff. The staff's review and approval will require, among other things, benchmark comparison of the code's predictions against experimental test data. In many cases, the staff's approval of a licensing RELAP code imposes conditions or restrictions for application of the code to ensure that licensing calculations are acceptably conservative, in accordance with the requirements of 10 CFR § 50.46 and Appendix K to 10 CFR Part 50. The implementation by a licensee or vendor of an approved emergency core cooling system (ECCS) code is controlled by the licensee or vendor's own quality assurance programs in accordance with Appendix B to 10 CFR Part 50.

In view of the above, it cannot be presumed that all other vintages of RELAP codes used by the industry have the same deficiencies as those experienced by Maine Yankee with its particular vintage of RELAP, that is RELAP5/MOD1. Two NRC licensees other than Maine Yankee, however, used the RELAP5/MOD1 vintage, that is, Yankee Rowe Nuclear Power Station and Vermont Yankee Nuclear Power Station. Yankee Rowe Nuclear Power Station has been permanently shut down for decommissioning since October 1, 1991. In May 1996, the NRC staff conducted an ECCS code and analysis inspection, and in

June 1996, a special inspection of Vermont Yankee. As a result, the NRC issued a Notice of Violation and Proposed Imposition of Civil Penalty--\$50,000 (EA 96-210) on August 23, 1996, for the licensee's failure to assume a specific failure scenario in the LOCA analysis. In that enforcement action, the NRC staff also concluded that Vermont Yankee's corrective actions were prompt and comprehensive. With respect to Maine Yankee, the NRC staff has examined MYAPCO's use of RELAP5YA through the Assessment Team review and the OI investigation. The staff's evaluation of Maine Yankee's use of RELAP5YA is ongoing with regard to any violations of NRC requirements, including 10 CFR § 50.46. The staff will keep Petitioner informed by providing Petitioner with copies of publicly available inspection reports and enforcement actions.

Petitioner, nonetheless, correctly points out that the NRC staff should conduct ECCS code and analysis inspections more frequently. In February 1997, the staff's Maine Yankee Lessons Learned Task Group provided its report to the Commission, "Report of the Maine Yankee Lessons Learned Task Group" (December 5, 1996), Attachment 1 to SECY-97-042, "Response to OIG Event Inquiry Regarding Maine Yankee" (February 18, 1997). The Task Group identified a need to place additional emphasis on (1) audits and inspections of implementation by licensees and vendors of their ECCS codes and methodologies, not limited to the various RELAP codes, and (2) verification of the conformance by licensees and vendors with the conditions specified in the NRC staff's Safety Evaluation Reports as a basis for determining whether codes and methodologies conform with NRC requirements. The Task Group also addressed inspections pursuant to the Core Performance Action Plan, performed to assess the impact of reload core design activities on plant safety. Licensees or vendors found to be in violation of NRC regulations will be subject to enforcement actions.

As explained above, there is no basis to conclude that the problems identified with the RELAP5/MOD1 vintage ECCS code used by Maine Yankee are or may be present in the different RELAP code vintages at other NRC-licensed plants. Additionally, the two other users of the RELAP5/MOD1 code vintage have either been inspected (Vermont Yankee) or are permanently shut down (Yankee Rowe). Nevertheless, the NRC will conduct computer code inspections of selected NRC licensees and vendors, not limited to users of RELAP, as explained above.

In view of the above, Petitioner's request to inspect all users of RELAP and to fine those users not operating within required computer code verification procedures is granted in part, since some users of RELAP will be included in forthcoming computer code inspections and since Maine Yankee and Vermont Yankee have already been inspected.

- B. Have MYAPCo and YAEC kept records of the use of the RELAP ECCS computer code in accordance with YAEC's computer code quality assurance procedures?

Petitioner requests that the NRC fine MYAPCO and YAEC if records regarding use of the computer code RELAP5YA have not been kept in accordance with YAEC's computer code quality assurance (QA) procedures. The NRC staff's review of the application of RELAP5YA for Maine Yankee between December 11 and 14, 1995, focused on the adequacy of the RELAP5YA SBLOCA analysis to support operation of Maine Yankee during Cycle 15. In particular, the staff evaluated conformance of the code to SER conditions and compliance of the ECCS evaluation model with regulatory requirements. Although the staff's review did not focus on record keeping requirements, the staff did not identify instances in which the appropriate records had not been kept. The staff is

continuing its evaluation of RELAP5YA for compliance with other NRC requirements.

Siemens Power Corporation (SPC) has prepared a plant-specific SBLOCA ECCS evaluation model for Maine Yankee, which has been submitted by Maine Yankee in response to the January 3, 1996, Order. The evaluation model is based on SPC's ANF-RELAP SBLOCA methodology, which was originally approved by the NRC in 1989, with further modifications approved by the NRC in 1994. Between February 10, 1997 and April 4, 1997, the staff conducted a four-week QA inspection of SPC. The inspection included a comprehensive review of documentation associated with SPC's LBLOCA and SBLOCA ECCS evaluation models, including the approved ANF-RELAP SBLOCA methodology. The staff's findings associated with ANF-RELAP will be documented in the inspection report, which will be issued by the NRC in the near future. A copy of the inspection report will be provided to Petitioner when it is publicly available. In addition, the NRC staff is currently performing a detailed technical review of the plant-specific ANF-RELAP ECCS evaluation model prepared by SPC for Maine Yankee, and submitted by Maine Yankee. The staff's evaluation of the plant-specific evaluation model will be documented in a Safety Evaluation Report (SER) when completed. The staff concludes that these activities respond directly to the issues raised by Petitioner.

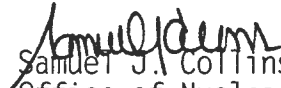
In view of the above, the Petitioner's request for a QA inspection of Maine Yankee's and YAEC's use of RELAP is granted in part, by virtue of the staff's previous and current inspection and review activities. Additionally, the staff will keep Petitioner informed by providing Petitioner with publicly available inspection reports, enforcement actions, and other documents as appropriate.

IV. CONCLUSION

As explained above, Petitioner's request to inspect all users of RELAP and fine those users not operating within required computer code verification procedures is granted in part. Petitioner's request to fine MYAPCO and YAEC if records regarding use of the computer code RELAP have not been kept in accordance with YAEC's computer code quality assurance procedures is also granted in part.

A copy of this Director's Decision will be filed with the Secretary of the Commission for Commission review in accordance with 10 CFR 2.206(c) of the Commission's regulations. As provided by this regulation, this Director's Decision will constitute the final action of the Commission 25 days after issuance unless the Commission, on its own motion, institutes review of the Decision within that time.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Dated at Rockville, Maryland,  
this 30th day of July 1997

DOCKETED  
USNRC

Patrick M. Sears

Box 13 113

Fairfield, PA 17320 '97 JUL 30 P1:17

OFFICE OF SECRETARY  
DOCKETED  
SERVICE  
FBI/DOJ

Congressman William F. Goodling  
2263 Rayburn Bldg.  
Washington, DC 20515

August 19, 1996

Dear Congressman Goodling,

I worked at the Nuclear Regulatory Commission from April 1980 to January 1993 when I retired. During most of that time I worked as a Project Manager (in general, a person who handles NRC regulatory issues concerning a particular nuclear power plant). From approximately Spring 1985 to Summer 1989 I was assigned Maine Yankee Atomic Power Station. In late 1988 and early 1989 I handled the NRC approval of RELAP5YA which is a large thermohydraulic computer code. RELAP5YA is a computer code modified from RELAP, a computer code developed for NRC by Idaho National Engineering Laboratory. The modifications were to have made RELAP adaptable to Maine Yankee Atomic Power Station. RELAP5YA was reviewed by Idaho National Engineering Laboratory and a favorable Technical Evaluation Report was forwarded to NRC in late 1988. That Technical Evaluation Report contained 12 conditions. I, in my capacity as NRC's Project Manager for Maine Yankee Atomic Power Plant issues, with full concurrence of my management, forwarded a favorable Safety Evaluation to Maine Yankee advising them that RELAP5YA had been reviewed and, contingent upon the 12 conditions, was in conformance with NRC regulations. On May 5, 1989 I received a phone call from Mr. S. Nichols of Maine Yankee telling me in my capacity as an NRC official that RELAP5YA was "operable." On May 8, 1989 I wrote to Mr. Nichols a letter confirming our phone conversation. A copy of that letter is attached.

In November or December 1995 an anonymous person sent a letter to the Union of Concerned Scientists. A copy of that letter is also attached. The allegations therein concerning me are false.

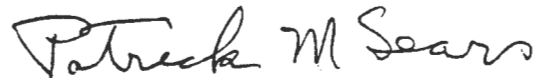
An investigation by the NRC Inspector General's Office has shown that Mr. Nichols lied to me in our May 5, 1989 phone conversation. The NRC's Inspector General's Office generated a report, (copy attached) which, to say the least, is not very complimentary to me.

I am sending a letter to NRC, copy attached, outlining my concerns about the Maine Yankee affair and computer code usage Quality Assurance/Quality Control in the nuclear

industry as a whole. In that letter, I am suggesting that more oversight of computer usage be exercised by NRC.

I am not an intervener nor do I wish to associate myself with organizations such as the Union of Concerned Scientists. I wish only to enjoy my retirement peacefully and not be made a scapegoat for the Maine Yankee event.

Yours truly,

A handwritten signature in cursive script that reads "Patrick M. Sears". The signature is written in dark ink and is positioned above the printed name.

Patrick M. Sears



Mr. Patrick M. Sears  
Box 13  
Fairfield, Pennsylvania 17320

Dear Mr. Sears:

This letter is to acknowledge receipt of your Petition dated August 19, 1996, addressed to the Executive Director for Operations of the U. S. Nuclear Regulatory Commission (NRC). The Petition requests that the NRC take action with regard to the Maine Yankee Nuclear Power Station and all users of the RELAP computer code for Emergency Core Cooling Systems (ECCS). The Petition requests that the NRC fine Maine Yankee Atomic Power Company (MYAPCo) and Yankee Atomic Electric Company (YAEC) if computer runs have not been kept in accordance with YAEC's computer code Quality Assurance procedures, and that the NRC inspect all users of RELAP and fine those users not operating within required computer code verification procedures.

As the basis for this request, the Petition states that: the May 5, 1989, statement of Steve Nichols of Maine Yankee, that RELAP5YA was "operable" and would be used for subsequent reloads was false; no computer code inspections were performed by NRC before a 1992 inspection at YAEC by Petitioner when he was an employee in the NRC Vendor Inspection Branch and not again until 1995; Mr. Sears was told not to do any more computer code inspections; RELAP is widely used; RELAP has been shown to have serious deficiencies; and the RELAP problem is not confined to the Maine Yankee Atomic Power Plant but is endemic to the industry as a whole.

The Petition has been referred to me pursuant to 10 C.F.R. § 2.206 of the Commission's regulations. As provided by Section 2.206, action will be taken on your request within a reasonable time.

I have enclosed for your information a copy of the notice that is being filed with the Office of the Federal Register for publication.

Sincerely,

William. T. Russell, Director  
Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/encl: Maine Yankee Atomic Power Company  
Yankee Atomic Electric Company

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U.S. NUCLEAR REGULATORY COMMISSION

Maine Yankee Atomic Power Company  
Yankee Atomic Electric Company

RECEIPT OF PETITION FOR DIRECTOR'S DECISION  
UNDER 10 C.F.R. § 2.206

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Notice is hereby give that by Petition dated August 19, 1996, Patrick M. Sears (Petitioner) has requested that the U. S. Nuclear Regulatory Commission (NRC) take action with regard to the Maine Yankee Nuclear Power Station and all users of the RELAP computer code for Emergency Core Cooling Systems (ECCS). Petitioner requests that the NRC fine Maine Yankee Atomic Power Company (MYAPCo) and Yankee Atomic Electric company (YAEC) if computer runs have not been kept in accordance with YAEC's computer code Quality Assurance procedures, and that the NRC inspect all users of RELAP and fine those users not operating within required computer code verification procedures.

As the basis for his request, Petitioner states that: the May 5, 1989, statement of Steve Nichols of Maine Yankee, that RELAP5YA was "operable" and would be used for subsequent reloads was false; no computer code inspections were performed by NRC before a 1992 inspection at YAEC by Mr. Sears when he was an employee in the NRC Vendor Inspection Branch and not again until 1995; Mr. Sears was told not to do any more computer code inspections; RELAP is widely used; RELAP has been shown to have serious deficiencies; and the RELAP problem is not confined to the Maine Yankee Atomic Power Plant but is endemic to the industry as a whole.

The request is being treated pursuant to 10 C.F.R. § 2.206 of the Commission's regulations. The request has been referred to the Director of the Office of Nuclear Reactor

Regulation. A copy of the Petition is available for public inspection at the Commission's Public Document Room at 2120 L Street, N.W., Washington, D.C. 20555.

FOR THE NUCLEAR REGULATORY COMMISSION

William T. Russell, Director  
Office of Nuclear Reactor Regulation

Dated at Rockville, Maryland

This \_\_\_\_ day of \_\_\_\_\_ 1996.