



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

August 27, 2014

LICENSEE: INDIANA MICHIGAN POWER COMPANY

FACILITIES: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2

SUBJECT: SUMMARY OF THE AUGUST 8, 2014, PRE-APPLICATION MEETING TO DISCUSS PENDING LICENSE AMENDMENT REQUEST ASSOCIATED WITH THE IMPLEMENTATION OF ALTERNATIVE SOURCE TERM AND TECHNICAL SPECIFICATION TASK FORCE TRAVELER (TSTF)-490 (TAC NOS. MF4483 AND MF4484)

On August 8, 2014, a Category 1 public meeting was held via teleconference between the U.S. Nuclear Regulatory Commission (NRC) staff and representatives of Indiana Michigan Power Company (I&M, the licensee). The purpose of the meeting was to discuss the licensee's pending license amendment request (LAR) to implement alternative source term (AST) analyses and TSTF-490, "Deletion of E Bar Definition and Revision of RCS Specific Activity Tech Spec."

A notice for this meeting was issued on July 28, 2014, and is available in the Agencywide Documents Access and Management System (ADAMS) under Accession No. ML14209A195. The licensee's meeting slide presentation can be found in ADAMS under Accession No. ML14231A198.

During the meeting, the licensee discussed its proposed submittal as outlined in its meeting slide presentation. The NRC staff queried the licensee and provided feedback throughout the presentation. A brief summary of the items discussed at the public meeting is provided below.

1. The NRC staff questioned the licensee as to what accidents were being analyzed in support of the AST LAR. The licensee stated that the following accidents were to be analyzed in association with the AST LAR:
 - Fuel Handling Accident
 - Main Steam Line Break
 - Steam Generator Tube Rupture
 - Locked Rotor
 - Control Rod Ejection
 - Waste Gas Decay Tank Rupture
 - Volume Control Tank Rupture
 - Loss-of-Coolant Accident (LOCA)
2. The NRC staff questioned if there were any specific differences (e.g., vendor, fuel, etc.) between Unit 1 and Unit 2 that would impact the analyses or calculations. The licensee stated that there are minor differences between the units, but all the analyses and calculations will be bounding for both units.

3. The NRC staff questioned if there would be any affect on the containment sump pH analysis. The licensee stated that it anticipates the AST LAR will not have an impact on the containment sump pH analysis.
4. The licensee stated that the new Title 10 of the *Code of Federal Regulations* Section 50.67 accident source term for the control room will be re-addressed in the AST LAR submittal. The submittal will include a full-scope analysis.
5. The NRC staff questioned the licensee about the large break LOCA (LBLOCA) peak cladding temperature (PCT) analysis methodology. The licensee stated that the LBLOCA PCT analysis for both units utilizes the NRC-approved ASTRUM methodology.
6. The NRC staff questioned the licensee about the nuclear fuel vendor for each unit. The licensee stated that both units utilize Westinghouse fuel.
7. The NRC staff questioned the licensee of the impact of the ice condensers on the dose analysis. The licensee stated that the ice condensers were not credited with iodine removal in the dose analysis.
8. The NRC staff asked if there will be any affect on the containment sump pH analysis. The licensee stated that there will be some impact.
9. The NRC staff asked if the technical support center (TSC) was considered part of the control room envelope and if there were any changes to the TSC dose assessment. The licensee stated that the TSC was not considered part of the control room enveloped, nor were there any anticipated changes to the TSC dose assessment.
10. The licensee stated that environmental qualification (EQ) calculations would be outside the scope of the submittals.
11. In previous AST license amendment request reviews, the NRC staff noted that a summary table for each analyzed accident has been a very useful tool. As such, the staff recommended that I&M provide a summary table for each accident being analyzed. The summary table would simplify the comparison of specific plant parameter value changes for AST. If there is a change in value, the NRC staff recommended a brief explanation be provided as to where in the license amendment request the justification for change could be found.
12. The licensee indicated that a formal license amendment request submittal is currently planned for late September 2014, with a requested NRC approval date of October 2015.

Before the meeting adjourned, all meeting participants were given the opportunity to comment on any aspects of the meeting.

No regulatory decisions or commitments were made during the meeting. The public was invited to observe the meeting and was given the opportunity to communicate with the NRC staff during the public meeting and before adjourning. The NRC staff received no public comments and no meeting feedback forms.

Please direct any inquiries to Mahesh Chawla at 301-415-8371 or via e-mail at Mahesh.Chawla@nrc.gov.

A handwritten signature in black ink, appearing to read "Terry A. Beltz", with a long horizontal line extending to the right.

Terry A. Beltz, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosure:
List of Participants

cc w/encl: Distribution via ListServ

LIST OF PARTICIPANTS

PUBLIC MEETING BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION (NRC)

AND INDIANA MICHIGAN POWER COMPANY (I&M)

TO DISCUSS A FUTURE LICENSE AMENDMENT REQUEST REGARDING

ALTERNATIVE SOURCE TERM AND TSTF-490

FOR THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2

<u>NAME</u>	<u>ORGANIZATION</u>
Terry Beltz	NRC/NRR/DORL/LPL3-1 ¹
Mahesh Chawla	NRC/NRR/DORL/LPL3-1 ¹
Terry Curtiss	I&M
Elijah Dickson	NRC/NRR/DRA/ARCB ²
Helen Etheridge	I&M
Fred Forsaty	NRC/NRR/DSS/SRXB ³
Matthew Hamm	NRC/NRR/DSS/STSB ⁴
Greg Hill	I&M
Alan Huynh	NRC/NRR/DE/ESGB ⁵
David Nold	NRC/NRR/DSS/SCVB ⁶
John Parillo	NRC/NRR/DRA/ARCB ²
Sheila Ray	NRC/NRR/DE/EEEB ⁷
Michael Scarpello	I&M
Jason Wright	I&M
Matt Yoder	NRC/NRR/DE/ESGB ⁵

¹ Office of Nuclear Reactor Regulation, Division of Operating Reactor Licensing, Plant Licensing Branch III-1

² Office of Nuclear Reactor Regulation, Division of Risk Assessment, Radiation Protection & Consequence Branch

³ Office of Nuclear Reactor Regulation, Division of Safety Systems, Reactor Systems Branch

⁴ Office of Nuclear Reactor Regulation, Division of Safety Systems, Technical Specifications Branch

⁵ Office of Nuclear Reactor Regulation, Division of Engineering, Steam Generator Tube Integrity and Chemical Engineering Branch

⁶ Office of Nuclear Reactor Regulation, Division of Safety Systems, Containment and Ventilation Branch

⁷ Office of Nuclear Reactor Regulation, Division of Engineering, Electrical Engineering Branch

Enclosure

Before the meeting adjourned, all meeting participants were given the opportunity to comment on any aspects of the meeting.

There were no members of the general public in attendance or participating in the teleconference.

No additional feedback regarding the meeting was provided.

Please direct any inquiries to Mahesh Chawla at 301-415-8371 or via e-mail at Mahesh.Chawla@nrc.gov.

Terry A. Beltz, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
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TBowers, EDO R-III

ADAMS Accession Numbers:

Package: ML14231A228

Meeting Notice: ML14209A195

Meeting Summary: ML14231A075

Licensee's Presentation: ML14231A198

OFFICE	LPL3-1/PM	LPL3-1/LA	LPL3-1/BC	LPL3-1/PM
NAME	TBeltz <i>TAB</i>	MHenderson <i>MAH</i>	DPelton <i>DP</i>	TBeltz <i>TAB</i>
DATE	08/25/14	08/25/14	08/27/14	08/27/14

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Please direct any inquiries to Mahesh Chawla at 301-415-8371 or via e-mail at Mahesh.Chawla@nrc.gov.

/RA/

Terry A. Beltz, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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