

VoglecolRAIsPEm Resource

From: Jaffe, David
Sent: Wednesday, August 13, 2014 11:10 AM
To: VoglecolRAIsPEm Resource
Subject: VOG-42-RAI-LTR-03.docx
Attachments: VOG-42-RAI-LTR-03.docx

Hearing Identifier: Vogtle_COL_eRAIs
Email Number: 104

Mail Envelope Properties (BBC4D3C29CD0E64E9FD6CE1AF26D84D501697A6D2973)

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Sent Date: 8/13/2014 11:10:06 AM
Received Date: 8/13/2014 11:10:14 AM
From: Jaffe, David

Created By: David.Jaffe@nrc.gov

Recipients:
"VogtlecolRAIsPEm Resource" <VogtlecolRAIsPEm.Resource@nrc.gov>
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August 13, 2014

Mr. B. L. Ivey, Vice President
Regulatory Affairs
Southern Nuclear Operating Company, Inc.
40 Inverness Center Parkway, B022
Birmingham, AL 35242

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 03 RELATED TO
FUKUSHIMA NEAR-TERM TASK FORCE RECOMMENDATION 4.2, "MITIGATION
STRATEGIES FOR BEYOND-DESIGN-BASIS EXTERNAL EVENTS"
FOR THE VOGTLE ELECTRIC GENERATING PLANT UNITS 3 AND 4
COMBINED LICENSES

Dear Mr. Ivey:

On March 12, 2012, the NRC staff issued Order EA-12-049, "Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events." Southern Nuclear Operating Company (SNC) responded to Order EA-12-049 by letter dated August 22, 2013 (ADAMS Accession No. ML13235A228). As part of the response, the licensee submitted a non-proprietary Westinghouse report, APP-GW-GLR-171, "AP1000 FLEX Integrated Plan". A proprietary version of APP-GW-GLR-171 was also submitted as an attachment to the August 22, 2013 letter and designated as APP-GW-GLR-170 (ADAMS Accession No. ML13235A229.)

In the course of reviewing the August 22, 2013 response to Order EA-12-049 the NRC staff has identified the need for additional information. The request for additional information (RAI) is enclosed. Please respond to this RAI within 30 days of receipt of this letter.

If you have any questions or comments concerning this matter, you may contact me at 301-415-1439 or david.jaffe@nrc.gov.

Sincerely,

/RA/

David H. Jaffe, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-025
52-026
eRAI Tracking Nos. 7463 and 7469

Enclosure:
Request for Additional Information 03

CC: see next page

If you have any questions or comments concerning this matter, you may contact me at 301-415-1439 or david.jaffe@nrc.gov.

Sincerely,

/RA/

David H. Jaffe, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-025
52-026
eRAI Tracking Nos. 7463 and 7469

Enclosure:
Request for Additional Information 03

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*Approval captured electronically in the electronic RAI system.

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Request for Additional Information 03
FUKUSHIMA NEAR-TERM TASK FORCE RECOMMENDATION 4.2
MITIGATION STRATEGIES FOR BEYOND-DESIGN-BASIS EXTERNAL EVENTS
Southern Nuclear Operating Co. (SNC)
Dockets 52-0025 and 52-0026

QUESTIONS

1. Following-up to Southern Nuclear Operating Company's (SNC's) response to RAI 02-9, dated June 19, 2014, in which SNC defined the ultimate water source as the Savannah River, based on NRC and industry concerns for potential debris in fluid systems at nuclear power plants, the NRC staff requests that the SNC address its plans to satisfy item 1.a.ix in Section 11.2 of NEI 12-06 as follows:

Potential clogging of strainers, pumps, valves or hoses from debris or ice when using rivers, lakes, ocean or cooling tower basins as a water supply.

The response should not be limited to the FLEX equipment, but should consider the entire flow path credited as part of the mitigating strategy to maintain or restore core, containment, and spent fuel pool cooling.

2. During a telephone conference on July 3, 2014 with the NRC staff, SNC indicated its plan to prepare plant-specific documentation and controls for the implementation of the AP1000 FLEX Plan similar to Vogtle Units 1 and 2, rather than preparing a revision to its FSAR to address the plant-specific FLEX plan. The NRC staff requests that SNC specify when its plant-specific documentation and controls for the implementation of the AP1000 FLEX Plan will be available for NRC review. The NRC staff also requests that SNC address its plans regarding an audit of the Regional Response Center to review maintenance activities and implementation of EPRI Technical Report 3002000623.