

50-261

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO:

Mr. Robert W. Reid

FROM:

Carolina Power & Light Company
Raleigh, North Carolina
E. E. Utley

DATE OF DOCUMENT

4/22/77

DATE RECEIVED

4/25/77

☒ LETTER☐ NOTORIZED

PROP

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1 SIGNED

DESCRIPTION

Ltr. consisting of information regarding
Reactor Vessel Overpressurization.....

ENCLOSURE

ACKNOWLEDGED

REACTOR VESSEL OVERPRESSURIZATION
DISTRIBUTION PER G. ZECH 10-21-76

DO NOT REMOVE

PLANT NAME: H.B. Robinson Unit 2

(1-P)

RJL

(40 cys rec'd)

SAFETY

FOR ACTION/INFORMATION

BRANCH CHIEF: (5)

LIC. ASST:

PROJECT MANAGER:

Reid
Zwetsch
Ingram

INTERNAL DISTRIBUTION

REG FILE

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LPR: Hartselle, S.C.

TIC:

RSIC:

ACRS 16 CYS. HOLDING SENT TO 1A AS CAT B

CONTROL NUMBER

771160175



Carolina Power & Light Company

April 22, 1977

FILE: NG 3514(R)

REGULATORY DOCKET FILE COPY

SERIAL: NG-77-478

Director of Nuclear Reactor Regulation
ATTN: Mr. Robert W. Reid, Chief
Operating Reactors Branch No. 4
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT UNIT NO. 2
DOCKET 50-261
FACILITY OPERATING LICENSE NO. DPR-23
REACTOR VESSEL OVERPRESSURIZATION



Dear Mr. Reid:

In your letter of April 1, 1977, it was requested that procedures be implemented to prevent Reactor Coolant Pump (RCP) starts (or "jogs") while in a water-solid condition when the steam generator temperature is greater than the RCS temperature. H. B. Robinson Unit No. 2 is currently operating at 100% power, and the concerns of your letter can best be satisfied by changes to the plant's administrative procedures which are performed in accordance with normal procedures. The required procedures change will be developed after a careful study of the systems involved and detailed review to ensure that all consequences are fully understood.

Until such time as these procedures are changed, no RCP starts will be conducted on a stagnant solid system. These procedure changes will remain in effect unless future overpressurization information renders them unnecessary and NRC approval has been obtained.

In reference to your request for a procedural review related to overpressurization, reviews have been performed on two previous occasions as required by your August 11, 1976 and January 10, 1977 letters. Further review at this time would not benefit the effort to prevent overpressurization. However, additional review of these procedures will be considered after the results of the overpressure analysis have been received and evaluated.

Yours very truly,

E. E. Utley
Senior Vice President
Power Supply

HSZ/CSB/gsm

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