

NRC FORM 195 (2-76)		U.S. NUCLEAR REGULATORY COMMISSION		DOCKET NUMBER 50-261	
NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL				FILE NUMBER	
TO: MR R W REID		FROM: CAROLINA POWER & LIGHT CO RALEIGH, NC E E UTLEY		DATE OF DOCUMENT 6-24-76	
				DATE RECEIVED 6-25-76	
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PROP		INPUT FORM			
DESCRIPTION LTR NOTORIZED 6-24-76 REQUESTING FOR LICENSE AMDT - REVISION OF TECH SPEC....CORRECTING AN ERRONEOUS SURVEILLANCE REQUIREMENT IN THE IN- SERVICE INSPECTION PROGRAM.....				ENCLOSURE  ACKNOWLEDGED  DO NOT REMOVE  PLANT NAME: H.B. Robinson #2	
SAFETY		FOR ACTION/INFORMATION		ENVIRO 6-28-76 RKB	
ASSIGNED AD:		ASSIGNED AD:			
✓ BRANCH CHIEF: (b) REID		BRANCH CHIEF:			
PROJECT MANAGER:		PROJECT MANAGER:			
✓ LIC. ASST.: (b) R. Ingram		LIC. ASST.:			
INTERNAL DISTRIBUTION					
✓ REG FILE		SYSTEMS SAFETY		PLANT SYSTEMS	
✓ NRC PDR		HEINEMAN		TEDESCO	
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GOSSICK & STAFF		ENGINEERING		IPPOLITO	
MIPC		MACCARRY		KIRKWOOD	
CASE		KNIGHT			
HANAUER		SIHWEIL		OPERATING REACTORS	
HARLESS		PAWLICKI		STELLO	
				SITE SAFETY &	
PROJECT MANAGEMENT		REACTOR SAFETY		ENVIRO ANALYSIS	
BOYD		ROSS		DENTON & MULLER	
P. COLLINS		NOVAK			
HOUSTON		ROSZTOCZY		ENVIRO TECH.	
PETERSON		CHECK		ERNST	
MELTZ				BALLARD	
HELTEMES		AT & I		SPANGLER	
SKOVHOLT		SALTZMAN			
		RUTBERG		SITE TECH.	
				GAMMILL	
				STEPP	
				HULMAN	
				SITE ANALYSIS	
				VOLIMER	
				BUNCH	
				✓ J. COLLINS	
				KREGER	
EXTERNAL DISTRIBUTION				CONTROL NUMBER	
✓ LPDR: Hartung, Sc		NAT LAB:		BROOKHAVEN NAT LAB	
✓ TIC:		REG. VIE		ULRIKSON(ORNL)	
✓ NSIC:		LA PDR			
ASLB:		CONSULTANTS			
ACRS 16 CYS HOLDING SENT					
				6428	

CP&L

Carolina Power & Light Company

Regulatory Docket File

June 24, 1976

DOCKETED  
USNRC

JUN 25 1976

Nuclear Section  
Docket Clerk

FILE: NG-3514(R)

SERIAL: NG-76-878

Director of Nuclear Reactor Regulation  
Attention: Mr. Robert W. Reid, Chief  
Operating Reactors Branch No. 4  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555



H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-263

LICENSE NO. DPR-23

REQUEST FOR LICENSE AMENDMENT - REVISION OF TECHNICAL SPECIFICATIONS

Dear Mr. Reid:

In accordance with the Code of Federal Regulations, Title 10, Part 50.90 and Part 2.101, Carolina Power & Light Company (CP&L) hereby requests a revision to the Technical Specifications for the H. B. Robinson Steam Electric Plant, Unit No. 2. The requested revision corrects an erroneous surveillance requirement in the Inservice Inspection program.

Item 6.7 of the H. B. Robinson Unit No. 2 Inservice Inspection program deals with an inspection requirement for valve supports and hangers on the reactor coolant pressure boundary. The item requires a visual inspection of the valve supports and hangers such that 33% are completed in the five-year interval and 100% are completed in a ten-year interval.

When the original H. B. Robinson Technical Specifications were issued, the applicability of item 6.7 was not fully understood and there was therefore no CP&L comment on its inclusion in the Technical Specifications. Subsequently, in the course of plant operation and the performance of the Inservice Inspection program, it has been determined that no valve supports or hangers exist on the H. B. Robinson Unit No. 2 reactor coolant system pressure boundary. Therefore, CP&L requests that the Technical Specifications be revised to indicate that item 6.7 of the Inservice Inspection program is not applicable to H. B. Robinson.

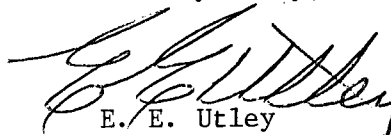
6428

June 24, 1976

The necessary replacement pages to incorporate this revision in the Technical Specifications are provided as an attachment to this letter. Appropriate changes are indicated by a vertical line in the margin of the affected page.

As required by Commission Regulations, this request is signed under oath by a duly authorized officer of the Company.

Yours very truly,



E. E. Utley  
Vice President  
Bulk Power Supply

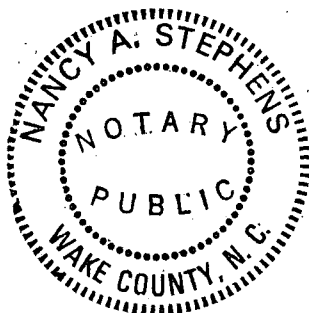
CSB/nja

Attachment

Sworn to and subscribed before me this 24th day of June, 1976.

  
Notary Public

My commission expires: June 29, 1976



H. B. ROBINSON UNIT NO. 2  
PROPOSED TECHNICAL SPECIFICATION CHANGE

Instructions

Delete

Page 4.2-14  
Page 4.2-24

Insert

Page 4.2-14  
Page 4.2-24

TABLE 4.2-1 (Cont.)

<u>Item No.</u>	<u>Examination Category</u>	<u>Components and Parts To Be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Extent of Examination (Percent in 5 Year Interval)</u>	<u>Remarks</u>
6.5	C-2	Pressure-retaining bolting	Visual	100%	33%	Exception is taken for valves which are not accessible.
6.6	X-1	Integrally-welded supports		Not Applicable	Not Applicable	
6.7	X-2	Supports and Hangers	Visual	Not Applicable	Not Applicable	
7.1		Primary pump Flywheel	V & UT	See Remarks	See Remarks	The flywheels shall be visually examined at the first refueling. At the fourth refueling the outside surface shall be examined by ultrasonic methods.
7.2		Irradiation Specimen Schedule	Tensile and Charpy V Notch (Wedge Open Loading)	See Remarks	See Remarks	Capsule 1 shall be removed and examined at the first region replacement. Capsule 2 shall be removed and examined at the fourth region replacement. Capsule 3 shall be removed and examined after twenty years of plant operation. Capsule 4 shall be removed and examined after thirty years of plant operation. Capsule 5 shall be removed and examined after forty years of plant operation.

Item 6.3 (Category F) - Valve-to-Safe End Welds

There are no valve-to-safe end welds in the piping boundary subject to this examination.

Item 6.4 (Category G-1) - Pressure-Retaining Bolting

There is no pressure-retaining bolting greater than two (2) inches in the valves subject to this examination.

Item 6.5 (Category G-2) - Pressure-Retaining Bolting

The bolting subject to this examination will be the bonnet bolting in valves three (3) inches in size or greater. This bolting will be inspected in accordance with Section XI of the code as shown in Table 4.2-1.

Item 6.6 (Category K-1) - Integrally-Welded Supports

There are no integrally-welded supports on the valves subject to this examination.

Item 6.7 (Category K-2) - Supports and Hangers

There are no supports and hangers on the valves subject to this examination.

G. Miscellaneous Inspections

Item 7.1 - Primary Pump Flywheels

The flywheels shall be visually examined at the first refueling. At the fourth refueling the outside surfaces shall be examined by ultrasonic methods. These examinations scheduled are shown in Table 4.2-1.

Item 7.2 - Materials Irradiation Surveillance Specimens

The reactor vessel surveillance program includes eight specimen capsules to evaluate radiation damage based on pre-irradiation and post-irradiation testing of specimens. The specimens are located about