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50-261

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INCIDENT REPORT

TO: Mr. Norman C. Moseley

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DESCRIPTION

1 page

PLANT NAME: H. B. ROBINSON UNIT # 2

jcm

ENCLOSURE

Licensee Event Report (R0-50-261/
77-8) on 04-24-77 concerning Unit # 2 power
was being reduced due to a RCP seal problem
when at 19% power a reactor trip occurred causing
control for N-9 not to fall into the core....

3 pages

ACKNOWLEDGED
DO NOT REMOVE
ACKNOWLEDGEDNOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

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EXTERNAL DISTRIBUTION

LPDR: Hartsville, SC

TIC:

NSIC:

CONTROL NUMBER

771470245



Carolina Power & Light Company

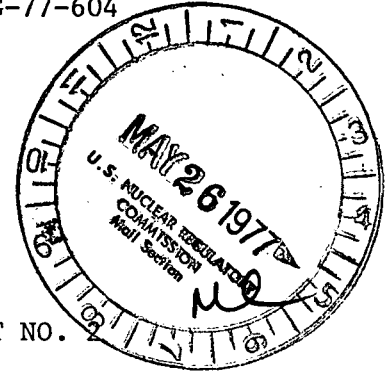
May 24, 1977

FILE: NG-3513 (R)

SERIAL: NG-77-604

Mr. Norman C. Moseley, Director
U. S. Nuclear Regulatory Commission
Region II, Suite 818
230 Peachtree Street, N.W.
Atlanta, Georgia 30303

Regulatory Docket File



H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO.
DOCKET 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 77-8

Dear Mr. Moseley:

In accordance with Section 6.9.2.b of the Technical Specifications for the H. B. Robinson Steam Electric Plant, Unit 2, the attached Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in Regulatory Guide 1.16, Revision 4.

Yours very truly,

H. R. Banks
Manager
Nuclear Generation

WH:lv

Attachment

cc: Messrs. W. G. McDonald
E. Volgenau

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE	
01	S	C	H	B	R	2	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	0	0	3		
7	8	9				14	15									25	26					30	31	32		

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER					EVENT DATE					REPORT DATE									
01	CONT	L	L	0	5	0	-	0	2	6	1	0	4	2	4	7	7	0	5	2	4	7	7
7	8	57	58	59	60	61					68	69					74	75					80

EVENT DESCRIPTION

02 At 1939 hours on April 24, 1977, Unit No. 2 power was being reduced due to a RCP

03 seal problem when at 19% power a reactor trip occurred. Control rod N-9 did not

04 fall into the core, which is a violation of paragraph - 3.10.6.1 of the Tech. Specs.

05 and constitutes a reportable occurrence in accordance with paragraph 6.9.2.b.2

06 of Tech. Specs. (HBR2 RO 77-8)

SYSTEM CODE	CAUSE CODE	COMPONENT CODE					PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION		
R	B	E	C	O	N	R	O	D	N	W	1	2	0	N
7	8	9	10	11	12			17	43	44			47	48

CAUSE DESCRIPTION

08 N/A

09

10

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION						
I	019	Reducing Power to	A	Operator Surveillance						
7	8	9	10	12	13	44	45	46		80
secure "C" RCP										

FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE					
Z	Z	N/A	N/A					
7	8	9	10	11	44	45		80

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION					
000	Z	N/A					
7	8	9	11	12	13		80

PERSONNEL INJURIES

NUMBER	DESCRIPTION					
000	N/A					
7	8	9	11	12		80

OFFSITE CONSEQUENCES

15 N/A

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION				
	N/A				
7	8	9	10		80

PUBLICITY

17 N/A

ADDITIONAL FACTORS

18 N/A

19

NAME: J. B. McGirt

PHONE: (803) 332-1351

Supplementary Information for

Reportable Occurrence 77-8

1. Report No.: 50-261/77-8
- 2a. Report Date: May 16, 1977
- 2b. Occurrence Date: April 24, 1977
3. Facility: H. B. Robinson Unit No. 2
Hartsville, South Carolina 29550
4. Identification of Occurrence:

On April 24, 1977, while in the process of unit load reduction, a reactor trip occurred at 19% power with "A" Shutdown Bank rod N-9 failing to drop into core. This is a violation of paragraph 3.10.6.1 of the Technical Specifications and constitutes a reportable occurrence in accordance with Technical Specifications paragraph 6.9.2.b.2.

5. Conditions Prior to Occurrence:

The plant was at 19% power and power reduction was in progress due to "C" Reactor Coolant Pumps (RCP) seal leakoff problems. No other unusual conditions prevailed prior to the occurrence.

6. Description of Occurrence:

On April 24, 1977, the "C" Reactor Coolant Pump seal leakoff became erratic. The unit was being retired to initiate repairs to the pump seals. At 19% power and 1939 hours, a reactor trip was received due to a high water level on "C" Steam Generator. All rods dropped into the core except N-9 on "A" Shutdown Bank, which did not move. The lift coil for N-9 was disconnected and the Shutdown Banks withdrawn. After the lift coil for N-9 was reconnected, operability of N-9 with its Bank was established, and at 2037 hours the Shutdown Banks were tripped with all rods dropping into the core, including N-9.

The plant was brought to cold shutdown conditions to perform maintenance on "C" RCP seals. On April 29, 1977, rod drop tests for N-9 under full flow, cold shutdown conditions were performed. Also on April 30, 1977, rod drop tests for N-9 under full flow, hot shutdown conditions were performed. All rod drop tests indicated that control rod N-9 was well within Technical Specification requirements of 1.8 seconds drop time. Subsequent operation of "A" Shutdown Bank indicated shutdown Rod N-9 to function properly and, therefore, was considered operable.

7. Designation of Apparent Cause of Occurrence:

The cause of the malfunction of control rod N-9 was either the failure of it's drive mechanism to release the rod or other restriction in either the control rod cluster or its drive shaft. The exact cause is not clearly known, but subsequent testing proved the control rod operable. Further examination and testing will be performed during the scheduled November refueling outage.

8. Analysis of Occurrence:

The malfunction of "A" Shutdown Bank control rod N-9 did not reduce the shutdown margin below the Technical Specification limits. No plant parameters or limitations were exceeded other than rod misalignment and rod drop requirements. There were no hazards imposed on the plant or public health and safety due to this equipment malfunction.

9. Corrective Action:

After the Shutdown Rod N-9 failed to drop into the core following the Reactor trip, the "A" Shutdown Bank was withdrawn and realigned with N-9. Operability of the Shutdown Bank as a unit, including N-9, was then demonstrated to be satisfactory.

Rod drop test were performed satisfactorily on Shutdown Rod N-9 in the cold and hot, full flow, shutdown conditions. Corrective action taken was considered adequate to demonstrate the operability of this Shutdown rod and to meet Technical Specifications requirements for rod drop time. Further examination and testing of N-9 control rod and rod drive mechanism is scheduled for the November 1977 refueling outage.

10. Failure Data:

There have been no prior component failures of this type.

RECEIVED DOCUMENT
PROCESSING UNIT

1977 MAY 26 PM 3 24