



NUREG-0800

U.S. NUCLEAR REGULATORY COMMISSION

STANDARD REVIEW PLAN

5.2.1.2— APPLICABLE CODE CASES

REVIEW RESPONSIBILITIES

Primary -- Organization responsible for ~~the review of~~ mechanical engineering ~~issues~~reviews

Secondary - Organization responsible for component ~~performance and testing~~integrity reviews

I. AREAS OF REVIEW

This Standard Review Plan (SRP) section is used to verify whether acceptable American Society of Mechanical Engineers (ASME) Code Cases are identified for component construction¹. Acceptable Code Cases for construction of nuclear power plants are identified in Regulatory Guide (RG) 1.84, "Design and Fabrication Code Case Acceptability, ASME Section III"; RG 1.147, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1"; and RG 1.192, "Operation and Maintenance Code Case Acceptability, ASME OM

¹ Constructed, as used herein, is an all inclusive term comprising material certification, design, fabrication, examination, testing, inspection, and certification required in the manufacture and installation of components.

~~Revision 3 — March 2007~~
Draft Revision 4 — August 2015

USNRC STANDARD REVIEW PLAN

This Standard Review Plan (SRP), NUREG-0800, has been prepared to establish criteria that the U.S. Nuclear Regulatory Commission (NRC) staff responsible for the review of applications to construct and operate nuclear power plants intends to use in evaluating whether an applicant/licensee meets the NRC regulations. The SRP is not a substitute for the NRC regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide an acceptable method of complying with the NRC regulations.

The SRP sections are numbered in accordance with corresponding sections in Regulatory Guide (RG) 1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)." Not all sections of RG 1.70 have a corresponding review plan section. The SRP sections applicable to a combined license application for a new light-water reactor (LWR) are based on RG 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)." These documents are made available to the public as part of the NRC policy to inform the nuclear industry and the general public of regulatory procedures and policies. Individual sections of NUREG-0800 will be revised periodically, as appropriate, to accommodate comments and to reflect new information and experience. Comments may be submitted electronically by email to NRO_SRP@nrc.gov.

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Code," which are incorporated by reference into Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a, "Codes and Standards." These Code Cases provide generally acceptable alternatives to the requirements of the ASME *Boiler and Pressure Vessel Code* (BPV Code) and Operation and Maintenance of Nuclear Power Plants (OM Code), as modified by conditions in the associated regulatory guides.

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The specific areas of review are as follows:

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1. ~~4.~~—ASME Code Cases contain alternatives to the requirements in the ASME BPV Code and ASME OM Code for the design, fabrication, manufacturing, construction, installation, testing, examination, and inspection of nuclear power plant components within the scope of these codes. ~~The primary organization determines~~ U.S Nuclear Regulatory Commission (NRC) staff review of an application under 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," or 10 CFR Part 52, "Licenses, Certifications, and Approvals for Nuclear Power Plants," will determine the acceptability of ~~American Society of Mechanical Engineers (ASME) and American National Standards Institute (ANSI) Code Case interpretations~~ ASME Code Cases specified in the ~~safety analysis report (SAR). These~~ applicant's technical submittal. The review will determine the acceptability of ASME Code Cases ~~must be approved before being applied to ASME~~ *Boiler and Pressure Vessel* BPV Code, Section III, Division 1, Subsection NB - Class 1 Components, Subsection- NC - Class 2 Components, and Subsection ND - Class 3 Components, ~~and Subsection NE - Class MC Components, as described in the Codes and Standards Rule, Section 50.55a(a) of 10 CFR Part 50. The review should include the approval.~~ The review will also determine the acceptability of Code Cases applied to ASME ~~Boiler and Pressure Vessel~~ BPV Code, Section III, Division 1, Subsection NE – Class MC Components, Subsection NF - Component Supports, and Subsection NG - Core Support Structures, and ASME ~~Boiler and Pressure Vessel~~ BPV Code, Section- III, Division- 2, Concrete Containment. ~~The~~ Further, the review ~~should also include~~ will determine the ~~approval~~ acceptability of Code Cases that may be applied to ASME BPV Code, Section XI, Division 1, Inservice Inspection, ~~and ASME Code for Operation and Maintenance for Nuclear Power Plant Components (OM). These Code Cases contain requirements, Code alternatives, or special rules which may be used for application in the construction of components for light water-cooled nuclear power plants. (ISI), and ASME OM Code Division 1, Inservice Testing (IST).~~
2. Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC). For design certification (DC) and combined license (COL) ~~reviews applications~~, the staff reviews the applicant's proposed ITAAC associated with the structures, systems, and components (SSCs) related to this SRP section in accordance with SRP Section 14.3, ~~"Inspections, Tests, Analyses, and Acceptance Criteria-."~~ The staff recognizes that the review of ITAAC cannot be completed until after ~~the rest of this portion of~~ the application has been reviewed against acceptance criteria contained in this SRP section. Furthermore, the staff reviews the ITAAC to ensure that all SSCs in this area of review are identified and addressed as appropriate in accordance with SRP Section 14.3.

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3. COL Action Items and Certification Requirements and Restrictions. For a DC application, the review will ~~also~~ address COL action items and requirements and restrictions (e.g., interface requirements and site parameters).

~~For a COL application referencing a DC, a COL applicant must address COL action items (referred to as COL licensee or information in certain DCs) items included in the referenced DC. Additionally, a COL applicant must address requirements and restrictions (e.g., interface requirements and site parameters) included in the referenced DC.~~

Review Interfaces

Other SRP sections interface with this section as follows:

1. ~~The materials engineering staff will evaluate ASME Code Cases pertaining to nondestructive testing are evaluated by the organizations responsible for the review of mechanical engineering issues and for the review of materials engineering issues related to flaw evaluation and welding~~SI under SRP ~~Sections 3.12 and 3.13.~~

~~Section 5.2. Code Cases pertaining to ASME Section III, Division 2, are evaluated by the organization.~~

2. ~~The technical branch~~ responsible for structural analysis reviews will evaluate ASME Code Cases pertaining to ASME BPV Code, Section III, Division 2, under SRP Sections 3.8.1, 3.8.3, and 3.8.5.

3. ~~Technical staff responsible for IST programs will evaluate ASME Code Cases pertaining to the OM Code are evaluated by the organization responsible for component performance and testing~~ under SRP Section 3.9.6.

- ~~3. All 4.~~ Appropriate technical branches will evaluate other areas covered by ASME Code Cases ~~are evaluated with assistance from other appropriate organizations~~ as necessary.

~~The specific acceptance criteria and review procedures are contained in the referenced SRP sections.~~

II. ACCEPTANCE CRITERIA

Requirements

Acceptance criteria are based on meeting the relevant requirements of the following Commission regulations:

1. ~~4.~~ 10 CFR Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants," General Design Criterion ~~1,~~(GDC) 1, "Quality Standards and Records," as it relates to the requirement that SSCs important to safety shall be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety function to be performed.

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2. 2.—10 CFR 50.55a, as it relates to the rule that establishes minimum quality standards for the design, fabrication, erection, construction, testing, and inspection of certain components of boiling and pressurized water reactor nuclear power plants by requiring conformance with appropriate editions of specified ~~published~~ industry codes and standards ~~incorporated by reference in 10 CFR 50.55a~~.
3. 3.—10 CFR 52.47(b)(1), which requires that a DC application contain the proposed ~~inspections, tests, analyses, and acceptance criteria (ITAAC)~~ that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, a ~~plant~~ facility that incorporates the design certification ~~is built~~ has been constructed and will ~~operate~~ be operated in ~~accordance~~ conformity with the design certification, the provisions of the Atomic Energy Act, (AEA), and the ~~NRC's~~ NRC's regulations.
4. 4.—10 CFR 52.79(a)(11), which requires a COL applicant to provide, in its safety analysis report, a description of the programs and their implementation necessary to ensure that the systems and components meet the requirements of the ASME BPV Code and ASME OM Code in accordance with 10 CFR 50.55a at a level sufficient to enable the NRC to reach a final conclusion on all safety matters that must be resolved before COL issuance.
5. 10 CFR 52.80(a), which requires that a COL application contain the proposed inspections, tests, and analyses, including those applicable to emergency planning, that the licensee shall perform, and the acceptance criteria that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, the facility has been constructed and will operate in conformity with the combined license, the provisions of the ~~Atomic Energy Act~~ AEA, and the ~~NRC's~~ NRC's regulations.

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SRP Acceptance Criteria

Specific SRP acceptance criteria acceptable to meet the relevant requirements of the ~~NRC's~~ NRC's regulations identified above are as follows for the review described in this SRP section. The SRP is not a substitute for the ~~NRC's~~ NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide acceptable methods of compliance with the NRC regulations.

~~To meet the requirements of General Design Criterion 1 and 10 CFR 50.55a, the following regulatory guides are used:~~

1. ~~Regulatory Guide~~ 10 CFR Part 50, Appendix A, GDC 1 requires that SSCs be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety function to be performed. For those SSCs defined as safety-related or risk-significant, the NRC regulations specify special treatment requirements to provide reasonable assurance of the capability of those SSCs to perform their safety-related functions. One special treatment requirement is that applicable components meet the requirements in the ASME BPV Code and ASME OM Code as incorporated by reference

in 10 CFR 50.55a. ASME Code Cases provide alternatives to specific requirements in the ASME BPV Code and ASME OM Code. The NRC staff considers ASME Code Cases to satisfy the requirements in 10 CFR Part 50, Appendix A, GDC 1 to design, fabricate, erect, and test components to quality standards commensurate with the importance of the safety function to be performed, where those Code Cases have been accepted by the NRC staff. Where risk insights will be applied in the review of an application, the applicable technical branch will evaluate whether the provisions in the applicant's submitted documentation are acceptable to provide reasonable assurance in the capability of SSCs to perform their intended functions.

2. Regulations in 10 CFR 50.55a require that components of the reactor coolant pressure boundary be designed, fabricated, erected, and tested in accordance with the requirements for Class 1 components of Section III of the ASME BPV Code and OM Code as incorporated by reference in 10 CFR 50.55a. This regulation also requires that pressure-retaining components of other fluid systems designated as Quality Group B or Quality Group C meet ASME BPV Code requirements for Class 2 or Class 3 components, respectively. Components within the scope of ASME BPV Code, Section XI, and ASME OM Code are subject to ISI and IST in accordance with ASME BPV Code and OM Code, respectively, as incorporated by reference in 10 CFR 50.55a. NRC regulations in 10 CFR 50.55a incorporate by reference specific revisions of the following NRC RGs for the acceptability of ASME Code Cases:

4-a. ~~RG 1.84, "Design and Fabrication Code Case Acceptability, ASME Section III, Division 1." This guide lists those ASME BPV Code, Section III, Division 1, ASME Code Cases oriented to design, fabrication, materials, and testing, which that are acceptable to the staff for implementation in the licensing of nuclear power plants.~~

4-b. ~~Regulatory Guide RG 1.147, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1." This guide lists those ASME BPV Code, Section XI-ASME, Code Cases which that are acceptable to the staff for use in the inservice inspection ISI of components and their supports, as described in the first numbered paragraph of subsection I, of this SRP.~~

4-c. ~~Regulatory Guide RG 1.192, "Operation and Maintenance Code Case Acceptability, ASME OM Code." This guide lists ASME OM Code Cases oriented to operation and maintenance for nuclear power plant components which that are acceptable to the staff for implementation in the licensing of nuclear power plants.~~

For Code Cases pertaining to ASME BPV Code, Section III, Division 2, as well as ASME Code Cases ~~alternatives to Regulatory Guides not covered in RGs 1.84, 1.147, or 1.192, or for those not covered in Regulatory Guides 1.84, 1.147, or 1.192~~ the applicant may ~~be acceptable in either of the following cases:~~

4- submit a request for NRC authorization to use an alternative to the ASME Code requirements in accordance with 10 CFR 50.55a(z). The NRC staff will review the proposed ~~Code Cases provide an acceptable level of quality and safety; or alternative for authorization based on the considerations indicated in 10 CFR 50.55a.~~

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- ~~1. If compliance with the specified requirements of 10 CFR 50.55a would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.~~
3. Regulations in 10 CFR 52.47(b)(1) require that a DC application contain proposed ITAAC necessary and sufficient to assure the plant is built and will operate in accordance with the design certification. Regulations in 10 CFR 52.80(a) requires that the COL identify the ITAAC necessary and sufficient to assure that the facility has been constructed and will be operated in conformity with the license. SRP Section 14.3 provides guidance for reviewing ITAAC. The requirements of 10 CFR 52.47(b)(1) and 10 CFR 52.80(a), respectively, will be met, in part, by identifying ITAAC for the top-level design features related to applicable Code cases referenced in DC and COL applications.
4. Regulations in 10 CFR 52.79(a)(11) require that a COL applicant provide, in its safety analysis report, a description of the programs and their implementation necessary to ensure that the systems and components meet the requirements of the ASME BPV Code and ASME OM Code in accordance with 10 CFR 50.55a at a level sufficient to enable the NRC to reach a final conclusion on all safety matters that must be resolved before COL issuance. RG 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)," provides guidance for the content of COL applications for a description of the ISI and IST programs to meet 10 CFR 50.55a. The NRC staff reviews the descriptions of these programs and documents its review in the applicable safety evaluation report (SER) sections.

Technical Rationale

The technical rationale for application of these acceptance criteria to the areas of review addressed by this ~~SRP~~ section is discussed in the following paragraphs:

1. ~~4. Compliance with General Design Criterion GDC 1 requires that SSCs be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety functions to be performed.~~
 - a. SRP Section 5.2.1.2 cites ~~Regulatory Guides RGs 1.84, 1.147, and 1.192 to provide a list of for Code Cases for applicable to ASME BPV Code, Section III, Division 1, components and materials, and Section XI, Division 1, tests and inspections, and ASME OM Code for operation and maintenance of nuclear power plant components, that are acceptable to the staff. Code Cases for Section III, Division 2, and other Code Cases that may be proposed are reviewed on a case-by-case basis. Under these Code Cases, components of the reactor coolant pressure boundary and other specified components must meet specific criteria that have been reviewed and accepted by the staff. The ASME Boiler and Pressure Vessel Committee adopts Code Cases that provide additions, revisions, and clarifications of existing Code requirements, or when urgently needed, provides rules for construction not covered by existing requirements.~~

~~The staff endorses the application of Code Cases that provide adequate assurance that these SSCs will perform acceptably, commensurate with the importance of their safety function.~~

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~~2. Section 50.55a of 10 CFR requires that SSCs be designed, fabricated, erected, constructed, tested, and inspected. Applicants may submit requests for authorization to quality standards commensurate with the importance of the safety functions to be performed and that suitable optional Code Cases may be applied to such SSCs.~~

~~b. SRP Section 5.2.1.2 cites Regulatory Guides 1.84, 1.147, and 1.192 (also cited in 10 CFR 50.55a(b)) to provide a list of use Code Cases for ASME Section III, Division 1, components and materials and Section XI, Division 1, tests and inspections and OM Code for operation and maintenance of nuclear plant components, that are acceptable to the staff. Code Cases for BPV Code, Section III, Division 2, and other Code Cases that may be proposed are reviewed on a case-by-case basis. Under these Code Cases, components of the reactor coolant pressure boundary and other specified components must meet specific criteria that have been reviewed and ASME Code Cases not accepted by the staff. The ASME Boiler and Pressure Vessel Committee adopts Code Cases that provide additions, revisions, and clarifications of existing in RGs 1.84, 1.147, and 1.192 as alternatives to ASME Code requirements, or when urgently needed, provides rules for construction not covered by existing in accordance with 10 CFR 50.55a(z). The NRC staff will prepare a safety evaluation describing its review of proposed alternatives to the ASME Code requirements.~~

~~c. The staff endorses considers the application of Code Cases that provide accepted in RGs 1.84, 1.147, and 1.192, or accepted in specific safety evaluations, to satisfy GDC 1 in providing adequate assurance that these plant SSCs will perform acceptably, commensurate with the importance of their safety function.~~

2. Regulations in 10 CFR 52.47(b)(1) require that a DC application contain proposed ITAAC necessary and sufficient to assure the plant is built and will operate in accordance with the design certification. Regulations in 10 CFR 52.80(a) require that the COL identify the ITAAC necessary and sufficient to assure that the facility has been constructed and will be operated in conformity with the license. SRP Section 14.3 provides guidance for reviewing ITAAC. These requirements in 10 CFR 52.47(b)(1) and 10 CFR 52.80(a) provide confidence in an applicant's compliance with 10 CFR 50.55a where accepted ASME Code Cases are applied.

3. Regulations in 10 CFR 52.79(a)(11) require that a COL applicant provide, in its safety analysis report, a description of the programs and their implementation necessary to ensure that the systems and components meet the requirements of the ASME BPV Code and ASME OM Code in accordance with 10 CFR 50.55a at a level sufficient to enable the NRC to reach a final conclusion on all safety matters that must be resolved before COL issuance. RG 1.206 provides guidance for the content of COL applications for a description of the ISI and IST programs to meet 10 CFR 50.55a. The requirements in 10 CFR 52.79(a)(11) and the guidance in RG 1.206 provide confidence in an applicant's compliance with 10 CFR 50.55a where accepted ASME Code Cases are applied.

III.- REVIEW PROCEDURES

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The reviewer will select material from the procedures described below, as may be appropriate for a particular case.

These review procedures are based on the identified SRP acceptance criteria. For deviations from these acceptance criteria, the staff should review the ~~applicant's~~ applicant's evaluation of how the proposed alternatives provide an acceptable method of complying with the relevant NRC requirements identified in Subsection II.

~~1. The table provided by the applicant identifying those ASME Code Cases applied to Section III, Division 1 and Division 2 components is checked for compliance with the list of acceptable Code Cases identified in Regulatory Guides 1.84, 1.147, and 1.192.~~

~~2. ASME Section III, Division 2, Code Cases oriented to Concrete Containments are reviewed by the staff on a case-by-case basis.~~

~~In the event an applicant should propose to use a Code Case not previously approved by the staff, upon request, a review of the Code Case is performed by the primary organization with assistance from the other organizations as appropriate.~~

1. In accordance with 10 CFR 52.47(a)(8), 10 CFR 52.47(a)(21), and 10 CFR 52.47(a)(22), and 10 CFR 52.79(a)(17) and 10 CFR 52.79(a)(20), for new reactor license applications submitted under Part 52, the applicant is required to (1) address the proposed technical resolution of unresolved safety issues and medium and high-priority generic safety issues which are identified in the version of NUREG-0933 current on the date up to 6 months before the docket date of the application and which are technically relevant to the design; (2) demonstrate how the operating experience insights have been incorporated into the plant design; and, (3) provide information necessary to demonstrate compliance with any technically relevant portions of the Three Mile Island requirements set forth in 10 CFR 50.34(f), except paragraphs 10 CFR 50.34(f)(1)(xii), 10 CFR 50.34(f)(2)(ix), and 10 CFR 50.34(f)(3)(v). These cross-cutting review areas should be addressed by the reviewer for each technical subsection and relevant conclusions documented in the corresponding SER section.
2. For reviews of DC and COL applications under 10 CFR Part 52, the NRC staff verifies that Section 5.2.1.2 of the applicant's submitted documentation specifies the use of ASME Code Cases that are included in revisions to RGs 1.84, 1.147, and 1.192 incorporated by reference in 10 CFR 50.55a. The staff also checks the table provided by the DC applicant identifying the applied ASME Code Cases for compliance with RGs 1.84, 1.147, and 1.192.
3. For ASME Code Cases pertaining to ASME BPV Code, Section III, Division 2, as well as for other Code Cases not previously accepted in RG 1.84, 1.147, or 1.192, the NRC staff reviews the request by the applicant to apply those Code Cases as an alternative to the ASME Code requirements in accordance with 10 CFR 50.55a. The NRC staff will review ASME Code Cases proposed by the applicant as alternatives to the ASME Code requirements for authorization based on the considerations specified in 10 CFR 50.55a(z).

For review of a DC application, the reviewer should follow the above procedures to verify that the design, including requirements and restrictions (e.g., interface requirements and site parameters), set forth in the ~~final safety analysis report (FSAR)~~ technical submittal meets the acceptance criteria. DCs have referred to the ~~FSAR~~ technical submittal as the design control document (DCD) or final safety analysis report (FSAR). The reviewer should also consider the appropriateness of identified COL action items. The reviewer may identify additional COL action items; ~~however, to ensure these COL action items are addressed during a COL application, they that~~ should be added to the DC applicant's DCD or FSAR.

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For review of a COL application, the scope of the review is dependent on whether the COL applicant references a DC, an early site permit (~~ESP~~) or other NRC approvals (e.g., manufacturing license, site suitability report or topical report).

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For review of both DC and COL applications, SRP Section 14.3 should be followed for the review of ITAAC. The review of ITAAC cannot be completed until after the completion of this section.

IV.- EVALUATION FINDINGS

The reviewer verifies that the applicant has provided sufficient information and that the review and calculations (if applicable) support conclusions of the following type to be included in the staff's ~~safety evaluation report~~ SER. The reviewer also states the bases for those conclusions.

4.——The ~~specified~~ NRC staff concludes that system components are in compliance with 10 CFR 50.55a and meet the requirements of GDC 1. This conclusion is based on the following findings:

The NRC staff finds that the ASME ~~and ANSI~~ Code Cases ~~whose requirements will~~ specified in the application to be applied in the design, fabrication, manufacturing, construction ~~of ASME Section III, Division 1, Class 1, Class 2, Class 3,~~ installation, testing, examination, and ~~Class MC~~ inspection of components within the scope of the ASME BPV Code and ASME OM Code are acceptable for use in accordance with the ~~rules~~ requirements of 10- CFR– 50.55a, ~~General Design Criterion~~ and 10 CFR Part 50, Appendix A, GDC 1; and the guidance provided in ~~Regulatory Guides~~ RGs 1.84, 1.147, and 1.192. ~~We conclude that compliance with the requirements of~~ For ASME Code Cases not accepted in RGs 1.84, 1.147, and 1.192, the NRC staff has found that the applicant's request to apply those Code Cases specified in its technical submittal as alternatives to the Code requirements is authorized in accordance with 10 CFR 50.55a(z). The staff concludes that compliance with these Code Cases will result in a component quality level commensurate with the importance of ~~the~~ their safety function ~~of these components~~ and constitutes an acceptable basis for satisfying the requirements of 10- CFR 50.55a and ~~General Design Criterion~~ 10 CFR Part 50, Appendix A, GDC 1, ~~and is therefore acceptable.~~

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For DC and COL reviews, the findings will also summarize the ~~staff's~~ staff's evaluation of requirements and restrictions (e.g., interface requirements and site parameters) and COL action or information items relevant to this SRP section.

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In addition, to the extent that the review is not discussed in other SER sections, the findings will summarize the staff's evaluation of the ITAAC, including design acceptance criteria, as applicable.

V. IMPLEMENTATION

The staff will use this SRP section in performing safety evaluations of DC applications and license applications submitted by applicants pursuant to 10 CFR Part 50 or 10 CFR Part 52. Except when the applicant proposes an acceptable alternative method for complying with specified portions of the ~~Commission's~~Commission's regulations, the staff will use the method described herein to evaluate conformance with Commission regulations.

The provisions of this SRP section apply to reviews of applications ~~submitted six~~docketed 6 months or more after the date of issuance of this SRP section, unless superseded by a later revision.

VI. REFERENCES

1. ~~10 CFR Part~~American Society of Mechanical Engineers, *ASME BPV Code*, "Code Cases: Nuclear Components," New York, NY.
2. American Society of Mechanical Engineers, *ASME BPV Code*, "Nuclear Power Plant Components," New York, NY.
3. American Society of Mechanical Engineers, *ASME BPV Code*, "Code Cases: Nuclear Components," New York, NY.
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PAPERWORK REDUCTION ACT STATEMENT

The information collections contained in the Standard Review Plan are covered by the requirements of 10 CFR Part 50 and 10 CFR Part 52, and were approved by the Office of Management and Budget, approval number 3150-0011 and 3150-0151.

PUBLIC PROTECTION NOTIFICATION

Revision 3 — March 2007

5.2.1.2-11

Draft Revision 4 - August 2015

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