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 VARGA,S.A. Operating Reactors Branch 1

SUBJECT: Responds to request for addl info re post-accident sampling
 sys per NUREG-0737, Item II.B.3, in response to 841109
 conference call.

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Director of Nuclear Reactor Regulation
Attention: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing
United States Nuclear Regulatory Commission
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23
RESPONSES TO QUESTIONS REGARDING THE POST
ACCIDENT SAMPLING SYSTEM (PASS)

Dear Mr. Varga:

SUMMARY

A November 9, 1984 conference call between members of my staff and NRC defined additional information required by NRC to complete their review of the H. B. Robinson Steam Electric Plant, Unit 2 (HBR2) PASS. This letter addresses those requests.

DETAILS

The following information is provided in response to the three issues raised by the reviewer, Mr. Paul Wu, during the November 9, 1984 conference call:

Item 1 Criterion 2 - Carolina Power & Light Company (CP&L) should adopt the Westinghouse Owners' Group (WOG) methodology regarding core damage estimating. This would consist of inserting HBR2 specific parameters into the generic methodology. This should be submitted to NRC.

Response CP&L will adopt the WOG methodology regarding core damage assessment. Copies of the procedures will be submitted when the incorporation is completed.

Item 2 Criterion 3 - CP&L needs to submit confirmation that appropriate PASS valves are environmentally qualified. Qualification was to have been accomplished during the SG replacement outage.

Response Environmental qualification of appropriate PASS valves was accomplished during the Steam Generator Replacement Outage.

Item 3 Criterion 10 - CP&L needs to address the requirements of the Standard Test Matrix for Item II.B.3 of NUREG-0737 and submit results. It is permissible to reference work performed by vendors.

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Mr. Steven A. Varga

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Response CP&L is currently evaluating a proposal from the Vendor, Combustion Engineering (CE), to address the requirements of the Standard Test Matrix. We will complete our evaluation and provide a date by January 31, 1985 for submittal of the required information.

In addition to the questions received from NRC, additional clarification to our August 12, 1983 submittal is provided below:

Response 1, Pg. 2: Our submittal stated "The counting room is on the same level and about 50 feet away from the Post Accident Sample Sink (PASS). The chemistry lab is about 20 feet from the PASS."

The distance described in the submittal is currently correct, but the counting room and the chemistry lab will be relocated to the new laboratory facility in the near future. At that time, the distance from the PASS will be 540 to 650 feet depending upon post-accident accessibility.

Response 6, Pg. 17: Our submittal stated that the total exposure at the control panel will be less than 200 mr/hr. This was based on the assumption that a lead curtain would be installed behind the sample sink. The lead curtain was deleted from the final design; therefore, the calculated dose at the control panel would be 251.92 mr/hr. Utilizing the existing shielding and minimizing stay time is sufficient for maintaining low exposures.

Questions your staff may have regarding this matter may be referred to Mr. Sherwood R. Zimmerman at 919-836-6242.

Yours very truly,



A. B. Cutter - Vice President
Nuclear Engineering & Licensing

JSK/pgp (870NLU)

cc: Mr. J. P. O'Reilly (NRC-RII)
Mr. G. Requa (NRC)
NRC Resident Inspector (RNP)