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 AUTH. NAME: UTLEY, E. E. AUTHOR AFFILIATION: Carolina Power & Light Co.  
 RECIP. NAME: VARGA, S. A. RECIPIENT AFFILIATION: Operating Reactors Branch 1

SUBJECT: Responds to NRC 810821 ltr requesting plant-specific info re thermal shock to reactor pressure vessels. Response to Questions (3) & (4) of 60-day request is more appropriately included w/ submittal of 150-day response.

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Carolina Power & Light Company

September 21, 1981

File: NG-3514(R)

Serial No.: NO-81-1540

Office of Nuclear Reactor Regulation  
Attention: Mr. Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DPR-23  
THERMAL SHOCK TO REACTOR PRESSURE VESSELS



Dear Mr. Varga:

Summary:

Mr. D. G. Eisenhower's letter of August 21, 1981 requested certain plant specific information concerning Thermal Shock to Reactor Pressure Vessels be provided within 60 days and additional information be provided within 150 days. After reviewing the requested information, we have determined that we can answer questions (1), (2) and (5) within the requested 60 days; however, it would be more appropriate to defer answering questions (3) and (4) of the 60 day request until submittal of the remaining information with the 150 day response.

Background:

As described in the Westinghouse Owners' Group (WOG) letter OG-58, all Westinghouse plants, including HBR, meet vessel integrity criteria to beyond January, 1983 as a minimum. The RT<sub>NDT</sub> limit for HBR (or equivalently the specific date vessel integrity criteria may not be satisfied - without taking remedial actions) will be developed as part of the current WOG program as detailed in OG-58. To attempt to provide this information at an earlier date than already committed would impact the current WOG program in its abilities to provide the information for all Westinghouse operating plants. In addition, the RT<sub>NDT</sub> limit requested is based in part on generic WOG analysis which is planned to be completed consistent with the 150-day submittal date.

The basis of the RT<sub>NDT</sub> limit has already been provided in letter OG-58 (i.e., basis of calculation of time in life that vessel integrity criteria may not be met without remedial actions), however, additional information will be provided concurrently with the RT<sub>NDT</sub> limit as requested in question (3).

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411 Fayetteville Street • P. O. Box 1551 • Raleigh, N. C. 27602

Based on these considerations, we anticipate being able to provide the requested RT<sup>NDT</sup> value considered appropriate for continued operation at the time of our 150-day response.

If you have any questions on this item, please contact our staff.

Yours very truly,



E. E. Utley  
Executive Vice President  
Power Supply and  
Engineering & Construction

JJS/lr (3842)

cc: D. G. Eisenhower (NRC)  
W. Ross (NRC)