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 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power and Light 05000261
 AUTH. NAME: UTLEY, E. E. AUTHOR AFFILIATION: Carolina Power & Light Co.
 RECIP. NAME: SCHWENCER, A. RECIPIENT AFFILIATION: Operating Reactors Branch 1

SUBJECT: Requests revision to Tech Specs for License DPR-23 re steam generator water level instrumentation. Forwards revised page for Tech Specs, Westinghouse 790806 ltr to util concurring w/change & Class III amend fee.

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Notes - w/ check \$4,000.00

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APPL



Carolina Power & Light Company

September 27, 1979

File: NG-3514(R)

SERIAL: GD-79-2175

Office of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Operating Reactors Branch No. 1
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET 50-261
LICENSE NO. DPR-23
REQUEST FOR LICENSE AMENDMENT - STEAM GENERATOR
WATER LEVEL INSTRUMENTATION

Dear Mr. Schwencer:

In accordance with the Code of Federal Regulations, Title 10, Part 50.90 and Part 2.101, Carolina Power & Light Company (CP&L) hereby requests a revision to the Technical Specifications for its H. B. Robinson Steam Electric Plant, Unit No. 2. This request reflects changes necessary as a result of the information made available by Westinghouse Electric Corporation concerning the effects on Steam Generator Water Level Instrumentation as a result of a high energy line break (feedwater line rupture) inside containment and the ensuing high ambient temperature. This effect, it was determined, could cause as much as a 10% (of level span) error resulting in a delay of the Lo-Lo level reactor trip and auxiliary feedwater initiation. Although the Plant Operating Manual (POM) requires this setpoint to be 15% of level span which is adequate conservatism for the Safety Analyses and this temperature induced error, the Technical Specifications require only a 5% of level span setpoint. This request will increase this value to 14%. Included with this request is a copy of the Westinghouse letter concurring with this corrective action.

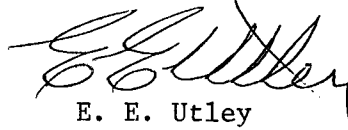
Enclosed is a replacement page for the H. B. Robinson Steam Electric Plant, Unit No. 2 Technical Specifications which reflects this revision. Appropriate changes are indicated by a vertical line in the right hand margin of the affected page.

In accordance with 10 CFR 170.12(c) we have determined that this revision constitutes one Class III Amendment. Accordingly, our check for \$4,000 is enclosed.

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w/ check
\$4,000.00
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If you have any questions on this issue, please contact our staff.

Yours very truly,



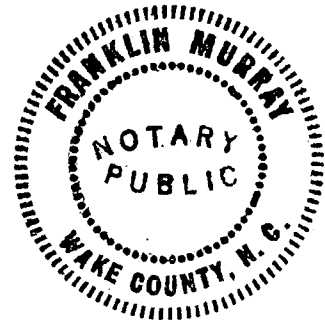
E. E. Utley
Executive Vice President
Power Supply and Customer Service

EEU/t1
Enclosure

Sworn to and subscribed before me this 27th Day of September, 1979.

Franklin Murray
Notary Public

My Commission Expires: October 4, 1981.



Westinghouse
Electric Corporation

Box 4808
1299 Northside Drive NW
Atlanta Georgia 30302

August 6, 1979

CPL-79-550

Mr. H. R. Banks, Manager
Nuclear Generation
Carolina Power & Light Company
P. O. Box 1551
Raleigh, North Carolina 27602

Dear Mr. Banks:

CAROLINA POWER & LIGHT COMPANY
H. B. ROBINSON UNIT 2
STEAM GENERATOR WATER LEVEL SETPOINT

Reference: Westinghouse letter CPL-79-541 dated July 5, 1979

Westinghouse concurs with Carolina Power & Light's logic on changes to the Technical Specification on Low-Low Steam Generator Level Setpoint from present 5% to 14% total, with actual plant setpoint being 15%. This new value would account for the original 5% correction for instrument error plus the new correction for reference leg heatup effects due to post accident containment temperature of 280°F maximum before reactor trip.

The change in Technical Specification to 14% with setting of actual setpoint of 15% would agree with the H. B. Robinson Unit 2 Precautions, Limitations and Setpoint Document (WCAP-7694) value as shown on page 16.

Very truly yours,

R. S. Longdon

R. S. Longdon, Manager
Southern Service Region
Nuclear Service Division

RSL/R. J. Muth

cc: B. J. Furr
R. M. Coats
E. G. Hollowell
B. H. Webster
R. B. Starkey, Jr.
J. F. Halifax
R. S. McGirt

e. Overpower ΔT

$$\leq \Delta T_o \left[K_4 - K_5 \frac{dT}{dt} - K_6 (T - T') - f(\Delta I) \right]$$

where:

ΔT_o = Indicated ΔT at rated power, °F

T = Average temperature, °F

T' = Indicated average temperature at nominal conditions and rated power, °F

K_4 = 1.07

K_5 = $\begin{cases} 0 & \text{for decreasing average temperature} \\ 0.2 \text{ seconds per } ^\circ\text{F} & \text{for increasing average temperature} \end{cases}$

K_6 = 0.002235 for $T \geq T'$; $K_6 = 0$ for $T < T'$

$f(\Delta I)$ = as defined in d. above.

f. Low reactor coolant loop flow $\geq 90\%$ of normal indicated flow

g. Low reactor coolant pump frequency ≥ 57.5 Hz

h. Under voltage $\geq 70\%$ of normal voltage.

2.3.1.3 Other Reactor Trips

a. High pressurizer water level $\leq 92\%$ of span

b. Low-low steam generator water level $\geq 14\%$ of narrow range instrument span.

2.3.2 Protective instrumentation settings for reactor trip interlocks shall be as follows:

2.3.2.1 The low pressurizer pressure trip, high pressurizer level trip, and the low reactor coolant flow trip (for two or more loops) may be bypassed below 10% of rated power.

2.3.2.2 The single-loop-loss-of-flow trip may be bypassed below 45% of rated power.