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SUBJECT: Responds to Generic Ltr 89-21 re request for info on status of implementation of USI requirements.

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ATTENTION: Document Control Desk  
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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261/LICENSE NO. DPR-23

RESPONSE TO NRC GENERIC LETTER 89-21: REQUEST FOR INFORMATION CONCERNING  
STATUS OF IMPLEMENTATION OF UNRESOLVED SAFETY ISSUE (USI) REQUIREMENTS

Gentlemen:

The information attached is provided in response to your Generic Letter 89-21 dated October 19, 1989 regarding the status of implementation of Unresolved Safety Issue (USI) requirements. Please note that the time provided to determine the status of implementation of all USIs as required by your request has been short. Therefore, the attached notations regarding the status of individual items are necessarily brief and cannot address all of the details of the more complex issues. The attached status has been compiled based upon a review of correspondence between CP&L and the NRC such as Safety Evaluation Reports (SER) and Inspection Reports, as well as other plant-specific documents.

If you have any questions concerning this information, please contact Mr. R. W. Prunty at (919)546-7318.

Yours very truly,

L. I. Loflin  
Manager

Nuclear Licensing Section

LIL/LSR/lbf (537CRS)  
Attachment

cc: Mr. S. D. Ebnetter  
Mr. L. Garner (NRC - HBR)  
Mr. R. Lo

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P PDC

ENCLOSURE 1UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	NC	Note 1
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	NC	
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	NC	Note 2
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	NA	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	NA	
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA	

\* C - COMPLETE  
NC - NO CHANGES NECESSARY  
NA - NOT APPLICABLE  
I - INCOMPLETE  
E - EVALUATING ACTIONS REQUIRED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	NA	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	NA	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	C(2/89)	Note 3
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhower dated 11/13/80 GL 81-11	BWR	NA	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	NC	
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWP	NC	Note 4
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/72 NUREG-1174, NUREG- 1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/ CR-4470, GL 89-18 (No requirements)	All	NC	Note 5
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C (3/85)	Note 6

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	C (4/78)	Note 7
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	NA	
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C (6/85)	Note 8
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	NA	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All		See A-46
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	NA	

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.8? (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	A11	NC	Note 9
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	A11	I (see note)	Note 10
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	A11	I (8/92)	Note 11
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	A11	I/E (see note)	Note 12
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG- 1218 GL 89-19	A11	E (3/90)	Note 13
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	A11, except PWRs with large dry containments	NA	
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C (6/89)	Note 14

## NOTES

1. Changes to steam generators circa 1984 incorporate J tubes.
2. Response to Generic Letter 85-02 submitted 6/17/85.
3. AMSAC installed in Refueling Outage 12 ending 2/25/89.
4. Resolution only applied to plants with CP issued after October 1983.
5. Resolution did not require any licensee actions.
6. NRC SER dated 3/19/85.
7. Overpressure protection system installed 2-4/78 documented in SER for TS dated 9/14/79. Response to GL 88-11 submitted 12/22/88.
8. Phase I SER dated 5/29/84; Phase II closed by GL 85-11 dated 6/28/85.
9. Revised guidance resulting from the resolution does not apply to HBR2 per GL 89-21.
10. CP&L letter dated 3/3/89 proposed HBR2 actions to meet rule. Awaiting NRC action.
11. Subsumed into IPE. Currently scheduled for submittal to NRC 8/92.
12. CP&L is part of SQUG. Awaiting NRC approval of Rev. 1 of GIP. However, see CP&L letter dated 10/6/88.
13. Response to GL 89-14 to be provided by 3/19/90.
14. NRC SER on 10 CFR 50.61 dated 6/19/89. Response to GL 88-11 submitted dated 12/22/88. See A-26.

JSK/lbf (541CRS)