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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9206240263 DOC.DATE: 92/06/12 NOTARIZED: NO DOCKET #
 FACIL:50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261
 AUTH.NAME AUTHOR AFFILIATION
 CROOK,D. Carolina Power & Light Co.
 CHAMBERS,R.H. Carolina Power & Light Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 92-009-00:on 920521,discovered that hydraulic snubber not included in snubber insp program.Caused by inadequate snubber insp.Adverse condition rept initiated & field rev issued to install new restraint.W/920612 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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| INTERNAL: | ACNW | 2 2 | AEOD/DOA | 1 1 |
| | AEOD/DSP/TPAB | 1 1 | AEOD/ROAB/DSP | 2 2 |
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| | NRR/DLPQ/LPEB10 | 1 1 | NRR/DOEA/OEAB | 1 1 |
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| | NRR/DST/SRXB 8E | 1 1 | REG FILE 02 | 1 1 |
| | RES/DSIR/EIB | 1 1 | RGN2 FILE 01 | 1 1 |
| EXTERNAL: | EG&G BRYCE,J.H | 3 3 | L ST LOBBY WARD | 1 1 |
| | NRC PDR | 1 1 | NSIC MURPHY,G.A | 1 1 |
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Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
POST OFFICE BOX 790
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JUN 15 1992

Robinson File No: 13510C

Serial: RNP/92-1650
(10CFR50.73)

United States Nuclear Regulatory Commission
Attn: Document Control Desk
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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT NO. 92-009-00

Gentlemen:

The enclosed Licensee Event Report (LER), is submitted in accordance with
10 CFR 50.73 and NUREG 1022, Supplements No. 1 and 2.

Very truly yours,

R. H. Chambers
General Manager
H. B. Robinson S. E. Plant

RDC:lkq

Enclosure

cc: Mr. S. D. Ebnetter
Mr. L. W. Garner
INPO

9206240263 920612
PDR ADOCK 05000261
S PDR

LEP
11

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NUMBER (2)
0 5 0 0 0 2 6 1

PAGE (3)
1 OF 0 4

TITLE (4)
TECHNICAL SPECIFICATION VIOLATION DUE TO INADEQUATE SNUBBER INSPECTION

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---|------------------|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | DOCKET NUMBER(S) | | | | | |
| 0 | 5 | 2 | 1 | 9 | 2 | 9 | 2 | 0 | 0 | 9 | 0 | 0 | 0 | 0 | 0 | 0 |
| 0 | 5 | 2 | 1 | 9 | 2 | 9 | 2 | 0 | 0 | 9 | 0 | 0 | 0 | 0 | 0 | 0 |

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

| OPERATING MODE (9) | 20.402(b) | 20.405(c) | 50.73(a)(2)(iv) | 73.71(b) |
|--------------------|-------------------|------------------|----------------------|--|
| N | | | | |
| POWER LEVEL (10) | 20.405(a)(1)(i) | 50.36(c)(1) | 50.73(a)(2)(v) | 73.71(c) |
| 0 | 20.405(a)(1)(ii) | 50.36(c)(2) | 50.73(a)(2)(vii) | |
| | 20.405(a)(1)(iii) | X 50.73(a)(2)(i) | 50.73(a)(2)(viii)(A) | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
| | 20.405(a)(1)(iv) | 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | |
| | 20.405(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(ix) | |

LICENSEE CONTACT FOR THIS LER (12)

| NAME | TELEPHONE NUMBER |
|---|--------------------------------|
| David Crook, Sr. Specialist - Regulatory Compliance | AREA CODE: 8 0 3 3 8 3 1 1 7 9 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPDs | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPDs |
|-------|--------|-----------|--------------|--------------------|-------|--------|-----------|--------------|--------------------|
| | | | | | | | | | |
| | | | | | | | | | |
| | | | | | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

| YES (If yes, complete EXPECTED SUBMISSION DATE) | NO | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|----|-------------------------------|-------|-----|------|
| | X | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 21, 1992, with H. B. Robinson Unit No. 2 in cold shutdown for a scheduled refueling outage, a hydraulic snubber was found that was not included in the licensees' Snubber Inspection Program. The snubber was discovered during a planned inspection to assess the impact of a discrepancy identified by a valve manufacturer where valve weight and center of gravity had been improperly evaluated with respect to small bore piping systems. Since Plant Technical Specifications require that all safety related hydraulic snubbers be inspected on a specified schedule, this condition constitutes a violation of plant Technical Specifications. The cause of this condition is attributed to a programmatic inadequacy.

An evaluation was conducted on the lines in question, concluding that the lines were functional without the snubber, and that the inoperable snubber did not adversely affect any other component. However, due to the high stresses that could be seen on the affected line, additional supports were installed.

This report is submitted pursuant to 10 CFR 50.73(a)(2)(i)(B).

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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| H. B. ROBINSON, UNIT NO. 2 | 0 5 0 0 0 2 6 1 | 9 2 | — | 0 0 9 | — | 0 0 |
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TEXT (If more space is required, use additional NRC Form 364A's) (17)

I. DESCRIPTION OF EVENT

On May 21, 1992, H. B. Robinson Unit No. 2¹ was in cold shutdown condition for a scheduled refueling outage. During an equipment stress analysis walkdown in the containment building, a hydraulic snubber not previously known to exist was discovered. This snubber is attached between the operator of valve SI-850D and the Polar Crane wall. Valve SI-850D, which is a cold leg safety injection test valve, is attached to line 3/4-SI-097, which is connected to lines 10-SI-048 and 3/4-SI-105. This snubber had not previously been included in the licensee's Snubber Inspection Program². The valve and support are located in the containment building below the 251 foot platform elevation.

The snubber was removed from the containment building and stroke tested. The test indicated that the snubber would allow movement, but would not lock up as required under simulated seismic conditions. In addition, the stress analysis for line 3/4-SI-097 determined that the lateral support at valve SI-850D required additional restraint in the vertical direction.

II. CAUSE OF EVENT

The cause of this condition is at this time attributed to a programmatic inadequacy in that the snubber had not been previously included in the inspection process. The snubber was discovered during inspections of small bore piping valves manufactured by Copes-Vulcan. This was a planned inspection to assure increased valve weights identified by Copes-Vulcan did not adversely affect the small bore piping. Field walkdowns were required since no previous pipe configuration verification inspections had been performed since original construction. This piping was exempt from the requirements of IEB 79-14 due to size and analysis method.

¹ H. B. Robinson Steam Electric Plant, Unit No. 2 is a Westinghouse Pressurized Water Reactor in commercial operation since March, 1971.

²EST-032, Visual Inspection of Hydraulic and Mechanical Shock Suppressors (Refueling and As Required Interval).

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 364A's) (17)

III. ANALYSIS OF EVENT

The lines where the inoperable snubber was located were determined by evaluation to be in an operable condition from original plant construction until the present time. Piping met acceptable criteria to assure pipe integrity but did not meet all original design criteria. Technical Specification 4.13 applies to safety related shock suppressors, and requires that all hydraulic snubbers be inspected on a specified schedule to ensure continued operability. Because the snubber discovered at SI-850D had not been included in the inspection program, had never been inspected, and was found to be inoperable when it was discovered, this condition is considered to be an operation prohibited by Technical Specifications. Therefore, this report is submitted pursuant to 10 CFR 50.73(a)(2)(i)(B).

IV. CORRECTIVE ACTIONS

An Adverse Condition Report³ was initiated to identify this condition and to determine cause and corrective actions necessary.

An evaluation was conducted on lines 3/4-SI-105, 3/4-SI-91, 3/4-SI-97A and 3/4-SI-97 to determine if the lines were functional without the snubber. An as-built configuration of the lines and their supports was performed and was used to develop a mathematical model for a piping and hanger analysis. A stress evaluation was then conducted to evaluate the lines for short term operability. The resulting calculations concluded that the lines are considered functional without the snubber, that the snubber did not restrict the line during thermal expansion, and the inoperable snubber did not impact any other component. However, the short term operability calculations identified areas where stresses were beyond code allowables. Based on these high stresses, a recommendation to install an additional support on line 3/4-SI-97 was made. On June 1, 1992, a Field Revision to Modification M-1104⁴ was issued to remove the existing support, install a new restraint at valve SI-850D, and to modify two existing supports. This Field Revision also installed a new mechanical snubber for the valve. These actions ensure that the line exceeds the short term operability criteria.

³ ACR 92-159, May 21, 1992.

⁴ Modification M-1104, RNP Piping System Upgrades, Field Revision 12.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-2164

EXPIRES: 8/31/88

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| FACILITY NAME (1) H. B. ROBINSON, UNIT NO. 2 | DOCKET NUMBER (2) 0 5 0 0 0 2 6 1 | LER NUMBER (8) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | 9 2 | — 0 0 9 | — 0 0 | 0 4 | OF | 0 4 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

V. ADDITIONAL INFORMATION

A. Previous Similar Events

None

B. Component Failures

None