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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8904260179 DOC.DATE: 89/04/18 NOTARIZED: NO DOCKET #
 FACIL:50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261
 AUTH.NAME AUTHOR AFFILIATION
 CROOK,D. Carolina Power & Light Co.
 MORGAN,R.E. Carolina Power & Light Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 89-005-00:on 890322,reactor trip due to inadvertent closure of MSIV.

DISTRIBUTION CODE: IE22D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 4 W/8 ltr.
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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AEOD/DSP/TPAB	1 1	AEOD/ROAB/DSP	2 2
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NRR/DEST/ADE 8H	1 1	NRR/DEST/ADS 7E	1 0
NRR/DEST/CEB 8H	1 1	NRR/DEST/ESB 8D	1 1
NRR/DEST/ICSB 7	1 1	NRR/DEST/MEB 9H	1 1
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NRR/DEST/RSB 8E	1 1	NRR/DEST/SGB 8D	1 1
NRR/DLPQ/HFB 10	1 1	NRR/DLPQ/QAB 10	1 1
NRR/DOEA/EAB 11	1 1	NRR/DREP/RPB 10	2 2
NRR/DRIS/SIB 9A	1 1	NUDOCS-ABSTRACT	1 1
REG FILE 02	1 1	RES/DSIR/EIB	1 1
RES/DSR/PRAB	1 1	RGN2 FILE 01	1 1
EXTERNAL: EG&G WILLIAMS,S	4 4	FORD BLDG HOY,A	1 1
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NRC Form 386
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NUMBER (2)

0 5 0 0 0 2 6 1 1 OF 0 3

PAGE (3)

TITLE (4)

REACTOR TRIP DUE TO INADVERTENT CLOSURE OF MAIN STEAM ISOLATION VALVE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0	3	2	8	9	0	0	5	0	0	0
0	3	2	8	9	0	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)								
N		20.402(b) <input type="checkbox"/> 20.405(c) <input checked="" type="checkbox"/> 50.73(a)(2)(iv) <input type="checkbox"/> 73.71(b) <input type="checkbox"/>								
POWER LEVEL (10)		20.405(a)(1)(i) <input type="checkbox"/> 50.36(c)(1) <input type="checkbox"/> 50.73(a)(2)(v) <input type="checkbox"/> 73.71(c) <input type="checkbox"/>								
1		20.405(a)(1)(ii) <input type="checkbox"/> 50.36(c)(2) <input type="checkbox"/> 50.73(a)(2)(vii) <input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A) <input type="checkbox"/>								
		20.405(a)(1)(iii) <input type="checkbox"/> 50.73(a)(2)(i) <input type="checkbox"/> 50.73(a)(2)(viii)(A) <input type="checkbox"/>								
		20.405(a)(1)(iv) <input type="checkbox"/> 50.73(a)(2)(ii) <input type="checkbox"/> 50.73(a)(2)(viii)(B) <input type="checkbox"/>								
		20.405(a)(1)(v) <input type="checkbox"/> 50.73(a)(2)(iii) <input type="checkbox"/> 50.73(a)(2)(x) <input type="checkbox"/>								

LICENSEE CONTACT FOR THIS LER (12)

NAME

DAVID CROOK - SENIOR SPECIALIST, REGULATORY COMPLIANCE

TELEPHONE NUMBER

AREA CODE

810 3 318 31-1117 19

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 22, 1989, with Unit No. 2 operating at one hundred percent (%) power, a reactor trip occurred due to the inadvertent closure of a Main Steam Isolation Valve (MSIV), which isolated flow from "A" Steam Generator. All plant systems performed as designed throughout the event, and the plant was stabilized using emergency operating procedures. The NRC Operations Center was notified of the event in accordance with 10CFR50.72 at 0256 hours via the ENS. The event is attributed to personnel error on the part of the licensed Control Operator who manipulated the incorrect switch during the performance of a surveillance test. The Control Operator has been disciplined for inattention to detail. Additionally, the human factors aspects associated with the event are being reviewed. This LER is submitted pursuant to 10CFR50.73(a)(2)(iv).

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PDR ADOCK 05000261
S PDC

IE22 1/1

NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
H. B. ROBINSON STEAM ELECTRIC PLANT,		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
UNIT NO. 2	0 5 0 0 0 2 6 1 8 9	-	0 0 5	-	0 0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. Description Of Event

On March 22, 1989, Unit No. 2 was operating at one hundred percent (%) power.¹ Licensee Operations personnel were concurrently performing scheduled surveillance tests OST-202, "Steam Driven Auxiliary Feedwater System Components Test", and EST-013, "Auxiliary Feedwater Pump Bearing Temperature Test". OST-202 required closing valve MS-VI-8C, "C" Steam Generator supply to the Steam Driven AFW Pump, to stop the pump. The licensed Control Operator performing the tests mistakenly manipulated the switch for Main Steam Isolation Valve (MSIV) VI-3A instead, isolating main steam flow from "A" Steam Generator. Upon receipt of steam generator alarms, the Control Operator immediately attempted to re-open the MSIV to prevent a reactor trip. The MSIV did not re-open, and the reactor received a trip signal at 0220 hours from a low-low level on "A" Steam Generator. Emergency operating procedures were utilized to stabilize the plant, and all systems performed as designed. The NRC was notified of this event at 0256 hours via the ENS pursuant to 10CFR50.72(b)(2)(ii).

II. Cause of Event

This event is attributed to personnel error on the part of the Control Operator. The switch on the Reactor Turbine Generator Board (RTGB) utilized to shut the SDAFW valve MS-VI-8C is located to the immediate left of the Main Steam Isolation Valve VI-3A, is similar in appearance, and is similar in labeling nomenclature. The Operator mistakenly manipulated the incorrect switch to secure the SDAFW Pump. A contributing cause was that the procedures used to perform the surveillance tests contained discrepancies regarding the length of time the SDAFW pump could run on minimum flow recirculation. While OST-202 stated that, under this condition, the pump run time shall not exceed thirty (30) minutes, EST-013 stated that the pump should be run for at least thirty (30) minutes. This placed additional pressure on the Operator to ensure the SDAFW pump run time was not in violation of either procedure, which may have diverted his attention to proper switch operation.

III. Analysis of Event

A. Reason Reportable

This event is reportable pursuant to 10CFR50.73(a)(2)(iv) as an event which resulted in the automatic actuation of an engineered safety feature.

B. Safety Assessment

All plant systems performed as designed throughout the event. Other than causing an unnecessary challenge to the Reactor Protection System, the reactor trip had no impact on plant safety.

¹/H. B. Robinson Steam Electric Plant, Unit No. 2 is a Westinghouse Pressurized Water Reactor Nuclear Power Plant in commercial operation since March, 1971.

NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
H. B. ROBINSON STEAM ELECTRIC PLANT,		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
UNIT NO. 2	0 5 0 0 0 2 6 1 8 9	-	0 0 5	-	0 0	0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Subsequent to the event, the valve manufacturer was consulted for guidance, and the "A" MSIV externals were inspected. The valve was tested and found to be operating properly.

IV. Corrective Action

The Control Operator was disciplined for inattention to detail. The human factors aspects associated with the proximity of the affected switches on the RTGB are being reviewed as part of the Corrective Action Program process. The surveillance procedures used for SDAFW pump testing are being revised to ensure compatibility when they are performed concurrently. In addition, as a precautionary measure and after discussion with the MSIV vendor, the MSIV internals are scheduled to be inspected during a future outage.

V. Additional Information

A. Failed Component Identification

There were no failed components applicable to this event.

B. Previous Similar Events

No previous events have occurred similar to the event described by this report.



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
POST OFFICE BOX 790
HARTSVILLE, SOUTH CAROLINA 29550

APR. 18 1989

Robinson File No: 13510C

Serial: RNP/89-1301
(10 CFR 50.73)


United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 88-005-00

Gentlemen:

The enclosed Licensee Event Report (LER) is submitted in accordance with
10 CFR 50.73 and NUREG-1022 including Supplements No. 1 and 2.

Very truly yours,


for R. E. Morgan
General Manager

H. B. Robinson S. E. Plant

RDC:bah

Enclosure

cc: Mr. S. D. Ebnetter
Mr. L. W. Garner
INPO

IER2
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