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ACCESSION NBR:8812200146 DOC.DATE: 88/12/09 NOTARIZED: NO DOCKET #
 FACIL:50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261
 AUTH.NAME AUTHOR AFFILIATION
 LEGETTE,F.L. Carolina Power & Light Co.
 MORGAN,R.E. Carolina Power & Light Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 88-026-00:on 881114,inadvertent safeguard actuation.
 W/8 ltr.

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) H. B. Robinson Steam Electric Plant Unit No. 2										DOCKET NUMBER (2) 0 5 0 0 0 2 6 1				PAGE (3) 1 OF 0 3									
TITLE (4) Inadvertent Safeguard Actuation																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)										
1	1	1	4	8	8	8	8	0	2	6	0	0	1	2	0	9	8	8	0	5	0	0	0
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 0 0		20.402(b)				20.405(c)				X 50.73(a)(2)(iv)				73.71(b)									
		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)									
		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME F. L. Legette - Senior Reactor Operator										TELEPHONE NUMBER AREA CODE 8 0 3 3 8 3 - 1 2 5 3													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs													
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input type="checkbox"/> NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

On Monday, November 14, 1988, the Plant was in cold shutdown for refueling. At about 1257 hours, a safeguard actuation was received from an inadvertent Safety Injection (SI) signal. An investigation into the event revealed the SI signal resulted from removal of the Low Pressurizer (PZR) Pressure SI blocked permissive. The block was removed when the licensee's operating personnel deenergized two of three reactor protection channels as part of initial actions in preparation for a plant modification. The NRC was notified via the ENS pursuant to 10CFR50.72(b)(2)ii at 1423 hours. The event resulted due to lack of procedural guidance in either the modification procedures or operating procedures to restrict the number of channels deenergized without safeguards defeated.

The operating procedures will be revised to require that only one train of Reactor Protection, only one train of Safeguards, or their power supplies can be deenergized at a time unless the Reactor is in Cold Shutdown and Safeguards is defeated. This LER has been reviewed by Operations' Training Staff and will be included in the upcoming license operator retraining program.

8812200146 881209
PDR ADOCK 05000261
S PDC

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
H. B. Robinson S. E. Plant, Unit 2	0 5 0 0 0 2 6 1	8 8	— 0 2 6	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 368A's) (17)

DESCRIPTION OF EVENT

On Monday, November 14, 1988, the Plant was in cold shutdown for refueling. The licensee had completed surveillance testing of its Emergency Diesel Generators and Safety Injection System.^{1,2} The restoration line-up for these systems was in progress. At about 1257 hours, a safeguard actuation was received from an inadvertent Safety Injection (SI) signal. This safeguard actuation did not result in actual injection into the Reactor Coolant System (RCS) since, at the time, the SI pumps breakers were open and their discharge valves shut. The remaining safeguard components functioned as designed. An investigation into the event revealed the SI signal resulted from removal of the Low Pressurizer (PZR) Pressure SI blocked permissive. The block was removed when the licensee's operating personnel deenergized two of three reactor protection channels as part of initial actions in preparation for a plant modification. The NRC was notified via the ENS pursuant to 10CFR50.72(b)(2)ii at 1423 hours.

II. CAUSE OF EVENT

The Modification procedure was originally written with a precaution that specified that only one protection channel was to be deenergized at a time during Cold Shutdown.³ This would be necessary to prevent the two of three (2/3) logic from being satisfied which will actuate Safeguards. However, during the modification review/approval process, this precaution was removed since Safeguards is normally defeated when the Plant is in Cold Shutdown. Eliminating this precaution was intended to allow implementation of work on more than one channel at a time, thus expediting the Modification. However, when this precaution was removed no precaution or initial condition was inserted to ensure that Safeguards was defeated prior to deenergizing multiple channels, thus preventing an inadvertent Safety Injection (SI) actuation. No similar guidance, restricting the number of channels deenergized without safeguards defeated, appeared in the operating procedures. At the time, Safeguards was not defeated due to a system line-up for unrelated surveillance testing. Additionally, miscommunication occurred between the Operations Coordinator and the Clearance Center Operators in that only a partial breaker line up was intended to be performed. This partial breaker line-up would have prevented the inadvertent SI. However, due to a misunderstanding and with no requirement that SI be defeated, a full breaker line-up was initiated and the inadvertent SI signal was actuated.

1/ OST-162 Revision 8, Emergency Diesel Generator Auto Start on Loss of Power and Safety Injection - Emergency Diesel Trips Defeat.

2/ OST-163 Revision 8, Safety Injection Test.

3/ Mod-959, Revision 0, Unit 2: RCS Bypass RTDs.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
H. B. Robinson S. E. Plant, Unit 2	0 5 0 0 0 2 6 1 8 8	—	0 2 6	—	0 0	0 3	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

III. ANALYSIS OF EVENT

The safeguard system is designed to mitigate the consequences of certain postulated accidents. However, with the Plant in cold shutdown, these accidents are not credible. The inadvertent safeguards actuation, therefore, created no condition that jeopardized Plant safety.

IV. CORRECTIVE ACTIONS

The Operations Coordinator and the Clearance Center Operators have met and discussed the event. This meeting placed particular emphasis on communication, with the conclusion that whenever issuing clearances that effect Reactor protection or Safeguards, a full understanding of what is to be done is essential. The operating procedures will be revised by March 3, 1989, to require that only one train of Reactor Protection, only one train of Safeguards or their power supplies can be deenergized at a time unless the Reactor is in Cold Shutdown and Safeguards is defeated. Additionally, this LER has been reviewed by Operations' Training Staff and will be included in the 1989 license operator retraining program.

A significant condition report has been initiated in accordance with PLP-026, Corrective Action Program, to further investigate the circumstances surrounding the event and to determine if any additional corrective actions are appropriate.

V. ADDITIONAL INFORMATION

A. Failed Components

None.

B. Previous Similar Event

LER 86-006-00 Engineering Safety Features Actuation During Cooldown

LER 84-006-00 Inadvertent SI - Containment Pressure

LER 84-010-00 Safeguards Actuation - Lo Pressurizer Pressure



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
POST OFFICE BOX 790
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DEC 09 1988

Robinson File No: 13510C

Serial: RNP/88-5836
(10 CFR 50.73)

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 88-026-00

Gentlemen:

The enclosed Licensee Event Report (LER) is submitted in accordance with
10 CFR 50.73 and NUREG-1022 including Supplements No. 1 and 2.

Very truly yours,

R. E. Morgan
General Manager
H. B. Robinson S. E. Plant

FLL:jch

Enclosure

cc: Mr. M. L. Ernst
Mr. L. W. Garner
INPO

Handwritten initials/notes in bottom right corner