

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8809200103 DOC.DATE: 88/09/09 NOTARIZED: NO DOCKET #
 FACIL:50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261
 AUTH.NAME AUTHOR AFFILIATION
 LEGETTE,F.L. Carolina Power & Light Co.
 MORGAN,R.E. Carolina Power & Light Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 88-017-00:on 880812,inadvertent maint requiring LCO for
 post-maint testing.

W/8 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 5
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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	PD2-1 LA	1 1	PD2-1 PD	1 1
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INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	ACRS WYLIE	1 1	AEOD/DOA	1 1
	AEOD/DSP/NAS	1 1	AEOD/DSP/ROAB	2 2
	AEOD/DSP/TPAB	1 1	ARM/DCTS/DAB	1 1
	DEDRO	1 1	NRR/DEST/ADS 7E	1 0
	NRR/DEST/CEB 8H	1 1	NRR/DEST/ESB 8D	1 1
	NRR/DEST/ICSB 7	1 1	NRR/DEST/MEB 9H	1 1
	NRR/DEST/MTB 9H	1 1	NRR/DEST/PSB 8D	1 1
	NRR/DEST/RSB 8E	1 1	NRR/DEST/SGB 8D	1 1
	NRR/DLPQ/HFB 10	1 1	NRR/DLPQ/QAB 10	1 1
	NRR/DOEA/EAB 11	1 1	NRR/DREP/RAB 10	1 1
	NRR/DREP/RPB 10	2 2	NRR/DRIS/SIB 9A	1 1
	NUDOCS-ABSTRACT	1 1	<u>REG FILE</u> 02	1 1
	RES TELFORD,J	1 1	RES/DSIR DEPY	1 1
	RES/DSIR/EIB	1 1	RGN2 FILE 01	1 1
EXTERNAL:	EG&G WILLIAMS,S	4 4	FORD BLDG HOY,A	1 1
	H ST LOBBY WARD	1 1	LPDR	1 1
	NRC PDR	1 1	NSIC HARRIS,J	1 1
	NSIC MAYS,G	1 1		

TOTAL NUMBER OF COPIES REQUIRED: LTTR 46 ENCL 45

NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) H. B. ROBINSON STEAM ELECTRIC PLANT UNIT NO. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 6 1	PAGE (3) 1 OF 0 4
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TITLE (4) INADVERTENT MAINTENANCE REQUIRING LCO FOR POST-MAINTENANCE TESTING

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0	8	12	8	8	017	0	9	09		0 5 0 0 0
					00					0 5 0 0 0

OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)							
POWER LEVEL (10) 1 0 0		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)	
		20.405(a)(1)(i)		50.38(c)(1)		50.73(a)(2)(v)		73.71(c)	
		20.405(a)(1)(ii)		50.38(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)			
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)			
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)	
NAME F. L. Legette, Senior Reactor Operator	TELEPHONE NUMBER AREA CODE 8 0 3 3 8 3 - 1 2 5 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract

On August 12, 1988, at approximately 1030 hours, Unit No. 2 was operating at 100 percent power. Licensee Maintenance personnel were prematurely authorized by shift Operations personnel to adjust the packing on one of two redundant containment vessel spray (CS) isolation valves (SI-845B). This was done prior to determining the post-maintenance testing condition and requirements. Post-maintenance testing required the Spray Additive portion of the CS system be isolated prior to stroke testing SI-845B after a packing adjustment. As a result, the CS system had to be declared inoperable, and the Plant was placed in an eight-hour LCO (Technical Specification 3.0) for the short period required to perform post-maintenance testing. At 1748 hours, the licensee notified the NRC via the ENS pursuant to 10CFR50.72b.1.ii.B. At 1757 hours the Spray Additive Tank was isolated to complete maintenance. At 1805 hours, the valve was tested satisfactorily and the CS system was returned to service.

Operations procedures are being revised to require post-maintenance testing determination prior to starting maintenance on Plant equipment. In addition, post-maintenance testing requirements will be addressed at the planning level prior to a Maintenance Work Request (WR) being issued for scheduling. Involved Operations and Maintenance personnel were interviewed and counseled by the Operations Manager.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
H. B. ROBINSON STEAM ELECTRIC PLANT UNIT NO. 2	0 5 0 0 0 2 6 1 8 8	—	0 1 7	— 0 0	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 368A's) (17)

I. Description of Event

On August 12, 1988, at approximately 1030 hours, Unit No. 2 was operating at 100 percent power.¹ The Licensee's senior control operator (SCO) had given Maintenance personnel authorization to adjust the packing on one of two redundant Containment Vessel Spray (CS) Isolation Valves (SI-845B). While maintenance was in progress, the SCO, after conferring with the Shift Foreman, realized that the required post-maintenance stroke testing rendered the CS System inoperable. A non-automatic valve in series with the two redundant valves would require closing in order to prevent gravity flow from the Spray Additive Tank to the Refueling Water Storage Tank (See attached diagram). The SCO notified Maintenance personnel to stop the work; however, the valve packing had already been adjusted. A 24-hour LCO was declared at 1045 hours on the inoperable valve while a surveillance test was revised to stroke test the valve. At 1748 hours, the licensee notified the NRC pursuant to 10CFR72.b.1.ii.(B) that the Plant would be entering an eight-hour LCO (Technical Specifications 3.0) to test and verify operability of SI-845B. At 1757 hours, the Spray Additive Tank (SAT) was manually isolated (at Valve SI-892C) with an operator designated to unisolate the SAT in the event of a CV spray signal.

The valve (SI-845B) was stroke tested per the revised surveillance procedure following completion of maintenance.² At 1805 hours, the SAT was restored to service and the LCO terminated. At 1808 hours, the licensee notified the NRC that the test was performed satisfactorily and that the Plant was no longer in the LCO.

II. Cause of Event

The SCO failed to adequately research post-maintenance testing requirements for the work to be done on (SI-845B), in that he relied on his memory instead of reviewing the operation work procedure testing requirements prior to authorizing the maintenance.³ Although the operations procedure does not require testing requirements to be reviewed prior to the start of maintenance, had the SCO done so, he would have realized that testing required the plant to be in cold shutdown.

¹H. B. Robinson Steam Electric Plant, Unit No. 2 is a 2300 megawatt thermal Westinghouse pressurized water reactor, in commercial operation since March 1971.

²OST-157, Revision 8 Safety Injection and Containment Spray Systems Valve Test

³OWP-004, Revision 5 Containment Spray

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) H.B. ROBINSON STEAM ELECTRIC PLANT UNIT NO. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 6 1 8 8 — 0 1 7 — 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 8	0 1 7	0 0	0 3	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

III. Analysis of Event

The Containment Vessel Spray (CS) system in conjunction with the Containment Air Handling (HVH) units is designed to cool and depressurize the containment vessel (CV) following a steam break in containment or a loss-of-coolant-accident. In addition to the heat removal function, the CS system uses the spray additive tank (SAT) to provide a means of adding sodium hydroxide to the spray lines for the absorption of CV free iodine following a LOCA. Although, the SAT was isolated for approximately eight minutes, sodium hydroxide would have been available via the designated operator whose sole function was to unisolate the SAT in the event of a containment spray signal. Therefore, at no time was there a threat to the health and safety of the plant or the public.

IV. Corrective Action

Immediately upon recognizing what had occurred, the Operations Manager met with involved Maintenance and Operations personnel and their supervision. This meeting was intended to emphasize the significance of the event, to determine the circumstances of the event and to identify corrective action. As a result of this meeting the Local Clearance and Test Request (LCTR) procedure is being revised to require formal review and documentation of testing requirements prior to the start of maintenance on safety related equipment. This revision will be completed by October 30, 1988.

Maintenance Planners/Analysts have been provided with a list of valves that require a specific plant configuration for maintenance. This list will inform the Planners/Analysts of specific requirements on these valves when they plan work requests (WR). Therefore WRs will include conditions required for maintenance at the planning level prior to being processed for scheduling.

V. Additional Information

- A. Failed Component: None
- B. Previous Similar Event: No similar LER reported.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)

H.B. ROBINSON STEAM ELECTRIC PLANT
UNIT NO. 2

DOCKET NUMBER (2)

0 5 0 0 0 2 6 1 8 8

LER NUMBER (6)

YEAR

SEQUENTIAL
NUMBER

REVISION
NUMBER

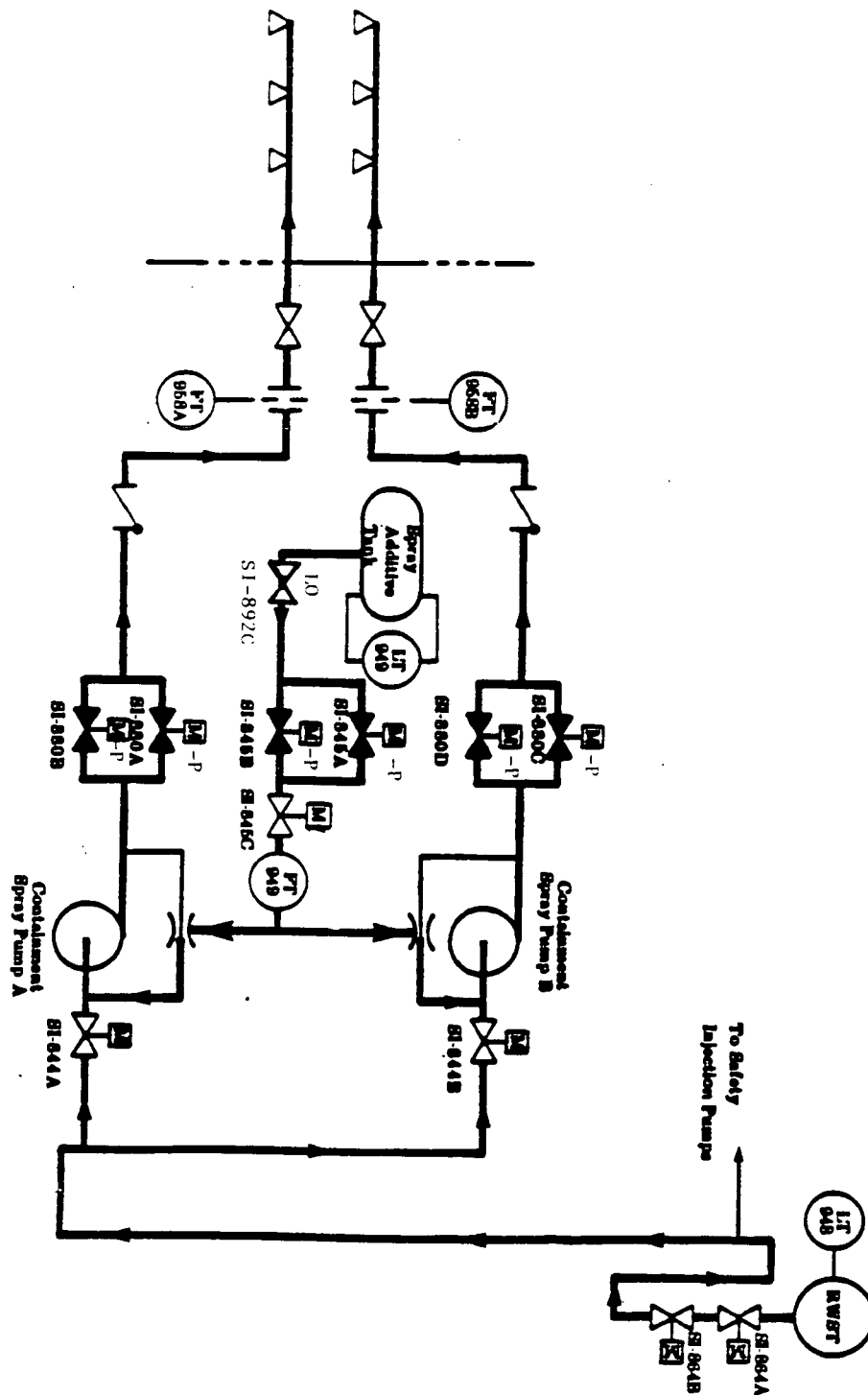
0 1 7

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PAGE (3)

0 4 OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)



P - Open on CV Spray Signal
LO - Locked Open



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
POST OFFICE BOX 790
HARTSVILLE, SOUTH CAROLINA 29550
SEP 9 1988

Robinson File No: 13510C

Serial: RNP/88-4072
(10 CFR 50.73)

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 88-017-00

Gentlemen:

The enclosed Licensee Event Report (LER) is submitted in accordance with
10 CFR 50.73 and NUREG-1022 including Supplements No. 1 and 2.

Very truly yours,

R. E. Morgan
General Manager
H. B. Robinson S. E. Plant

FLL:jch

Enclosure

cc: Dr. J. N. Grace
Mr. L. W. Garner
INPO

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