

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8708100496 DOC. DATE: 87/08/05 NOTARIZED: NO DOCKET #
 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261
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 SAYRE, D. Carolina Power & Light Co.
 MORGAN, R. E. Carolina Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-021-00: on 870712, orderly reactor shutdown from 100%
 reactor power initiated causing unidentified primary sys
 leakage. Caused by failure in pressurizer steam space sample
 valve, RC-586. Valve backseated & replaced. W/870805 ltr.

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 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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	AEOD/DOA	1 1	AEOD/DSP/NAS	1 1
	AEOD/DSP/ROAB	2 2	AEOD/DSP/TPAB	1 1
	DEDRO	1 1	NRR/DEST/ADE	1 0
	NRR/DEST/ADS	1 0	NRR/DEST/CEB	1 1
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	NRR/DLPQ/QAB	1 1	NRR/DOEA/EAB	1 1
	NRR/DREP/RAB	1 1	NRR/DREP/RPB	2 2
	NRR/PMAS/ILRB	1 1	<u>REG FILE</u> 02	1 1
	RES DEPY GI	1 1	RES TELFORD, J	1 1
	RES/DE/EIB	1 1	RGN2 FILE 01	1 1
EXTERNAL:	EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
	NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)		DOCKET NUMBER (2)	PAGE (3)
H. B. Robinson Steam Electric Plant, Unit No. 2		0 5 0 0 0 2 6 1	1 OF 0 2
TITLE (4)			

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)	
0	7	1	2	8	7	8	7	0	2	1	0	0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)												
OPERATING MODE (9)			20.402(b)			20.406(c)			50.73(a)(2)(iv)			73.71(b)
POWER LEVEL (10)			20.406(a)(1)(i)			50.38(a)(1)			50.73(a)(2)(v)			73.71(c)
1 0 0			20.406(a)(1)(ii)			50.38(a)(2)			50.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)
			20.406(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)			
			20.406(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)			
			20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER
NAME	AREA CODE	
Don Sayre, Senior Specialist - Regulatory Compliance	8 1 0 3	3 1 8 3 1 1 2 4 2

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	
X	A	B	5	M	V	E	10	9	15	Y

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)		X NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 12, 1987, an orderly reactor shutdown from 100% reactor power was initiated at Unit 2 due to unidentified primary system leakage greater than one gallon per minute (gpm). Technical Specification 3.1.5.1 requires that the source of a primary system leak be identified within twelve hours or the reactor placed in a hot shutdown condition utilizing normal operating procedures. It was decided to go to cold shutdown to allow personnel entry into containment to identify the source of the leak.

A primary system leak of about 2.4 gpm had been found earlier in the day during the daily leak rate test. The size of the leak had created a Limiting Condition for Operation (LCO) pending identification of the source. A team of personnel had been sent into containment prior to shutdown and had determined the source of the leak to be somewhere at the top of the Pressurizer. The cubicle temperature prohibited personnel access at the time.

An Unusual Event was declared at 2145 hours since the source of the leak had yet to be identified after twelve hours. At the time the Event had been declared, total unidentified primary system leakage had reached approximately 8 gpm.

The Plant was stabilized at 2305 hours, July 13, with a primary coolant temperature of 250 degrees Fahrenheit. Personnel entered the Pressurizer cubicle and identified the source of the leak as the packing on a Pressurizer steam space sample valve. The valve was backseated, repacked, and returned to service, with the Unusual Event being terminated at 0400 hours, July 14.

The Unit was returned to power operation at 1517 hours, July 15.

Review is underway to determine the root cause of the packing failure and to develop an action plan to prevent recurrence or a similar occurrence on like valves.

NRC Form 388A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
H. B. Robinson S. E. Plant, Unit 2	0 5 0 0 0 2 6 1 8 7	- 0	2 1	- 0	0 0 2	OF 0 2	

TEXT (If more space is required, use additional NRC Form 388A's) (17)

EVENT DESCRIPTION

On July 12, 1987, at 1656 hours, an orderly shutdown from 100% power was initiated at Unit 2 due to unidentified primary system leakage greater than one gallon per minute. Technical Specification 3.1.5.1 requires that, if the primary system leakage exceeds one gallon per minute (gpm) and the source of leakage is not identified within twelve hours, the reactor shall be placed in the hot shutdown condition utilizing normal operating procedures.

The NRC was notified of the shutdown decision in accordance with 10 CFR 50.72.

A primary system leak of approximately 2.4 gpm had been discovered earlier that day at 0955 hours during the daily leak rate test. Since the leak exceeded one gpm, the Plant entered a Limiting Condition for Operation (LCO) requiring the leak to be identified within 12 hours. Personnel dispatched into containment to identify the source of the leak reported the leak was coming from somewhere at the top of the Pressurizer cubicle; however, high temperatures in the cubicle precluded a more detailed inspection. Indications of the source of the leak included condensation dripping down the wall of the cubicle, a loud roar from the top of the cubicle, and excessive heat near the Pressurizer relief and safety valves. Reactor shutdown was initiated to reduce cubicle temperature and allow personnel entry into the cubicle. The Unit was taken off-line at 1932 hours.

An Unusual Event was declared at 2145 hours and the NRC notified in accordance with 10 CFR 50.72. Total unidentified primary system leakage had reached approximately 8 gpm.

The Unit was stabilized at 2305 hours with primary reactor coolant temperature at 250 degrees Fahrenheit on July 13.

Personnel re-entered containment and the Pressurizer cubicle and identified the source of the leak as the packing on steam space sample valve, RC-586.

CAUSE

The unidentified primary system leak was caused by a packing failure in Pressurizer steam space sample valve, RC-586.

CORRECTIVE ACTION

The valve was backseated and repacked. The Unusual Event was terminated at 0400 hours, July 14, and the Unit was returned to power operation at 1517 hours, July 15.

The NRC was notified of the Unusual Event termination in accordance with 10 CFR 50.72.

The cause of the packing failure has yet to be determined. Maintenance engineering review of the event has been initiated to identify the root cause and to develop an action plan to preclude recurrence or a similar occurrence on like valves.



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
POST OFFICE BOX 790
HARTSVILLE, SOUTH CAROLINA 29550
AUG 5 1987

Robinson File No: 13510C

Serial: RNP/87-3404
(10 CFR 50.73)

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 87-021

Dear Sir:

The enclosed Licensee Event Report (LER) is submitted in accordance with the Licensee Event Report System of 10 CFR 50.73. The format of the LER follows the recommendations of NUREG-1022, September 1983.

Very truly yours,

R. E. Morgan
for R. E. Morgan
General Manager

H. B. Robinson S. E. Plant

DAS:lko

Enclosure

cc: J. N. Grace
H. E. P. Krug
INPO

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